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Docket No.: 50-424

NL-12-2384

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555-0001

Vogtle Electric Generating Plant-Unit 1
Licensee Event Report 2012-004-00
Unplanned Auxiliary Feedwater Actuation

Ladies and Gentlemen:

In accordance with the requirements of 10 CFR 50.73 (a)(2)(iv)(A), Southern Nuclear Operating Company (SNC) is submitting the enclosed Licensee Event Report. This letter contains no NRC commitments. If you have any questions, please contact Doug McKinney at (205) 992-5982.

Respectfully submitted,

A handwritten signature in black ink, appearing to read "T. E. Tynan".

T. E. Tynan
Vice President – Vogtle

TET/GWG

Enclosure: Unit 1 Licensee Event Report 2012-004-00

cc: Southern Nuclear Operating Company
Mr. S. E. Kuczynski, Chairman, President & CEO
Mr. D. G. Bost, Executive Vice President & Chief Nuclear Officer
Mr. B. L. Ivey, Vice President – Regulatory Affairs
Mr. B. J. Adams, Vice President – Fleet Operations
Mr. M. J. Ajluni, Nuclear Licensing Director
Mr. D. A. Jackson, Vogtle OE Coordinator
RType: CVC7000

U. S. Nuclear Regulatory Commission
Mr. V. M. McCree, Regional Administrator
Mr. R. E. Martin, NRR Senior Project Manager - Vogtle
Mr. L. M. Cain, Senior Resident Inspector – Vogtle

**Vogtle Electric Generating Plant Plant – Unit 1
Licensee Event Report 2012-004-00
Unplanned Auxiliary Feedwater Actuation**

Enclosure

Unit 1 Licensee Event Report 2012-004-00

LICENSEE EVENT REPORT (LER)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA/Privacy Section (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects.resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME Vogtle Electric Generating Plant – Unit 1	2. DOCKET NUMBER 05000 424	3. PAGE 1 OF 3
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4. TITLE
Unplanned Auxiliary Feedwater Actuation

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
10	05	2012	2012	- 004 -	00	12	3	2012	FACILITY NAME	DOCKET NUMBER

9. OPERATING MODE 5	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR§: (Check all that apply)										
	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)							
10. POWER LEVEL 0	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)							
	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)							
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)							
	<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)							
<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER								
<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	Specify in Abstract below or in NRC Form 366A								

12. LICENSEE CONTACT FOR THIS LER

FACILITY NAME Vogtle Electric Generating Plant / George Gunn, Nuclear Licensing	TELEPHONE NUMBER (Include Area Code) 706-826-3596
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13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX

14. SUPPLEMENTAL REPORT EXPECTED	15. EXPECTED SUBMISSION DATE	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete 15. EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO			

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On October 5, 2012 with Unit 1 in Mode 5 and the residual heat removal (RHR) system in service for reactor coolant system temperature control, activities were in progress in preparation for Mode 4 entry. One of the activities involved restoration of the steam generator level RPS/ESFAS instrumentation from a bypassed condition to normal alignment. During the restoration a steam generator lo-lo level actuation was received and the motor-driven auxiliary feed water (AFW) pumps started and two turbine-driven AFW discharge valves stroked full open. The AFW system had previously been removed from service per the controlling unit operating procedure; however, following safety system testing during the outage the AFW system was not properly aligned to prevent inadvertent actuations. Neither the safety of the plant nor public health and safety were affected by this event.

**LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET**

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NARRATIVE

A. REQUIREMENT FOR REPORT

Since this actuation was not part of a pre-planned test or evolution this report is required per 10 CFR50.73(a)(2)(iv) due to automatic actuation of the Auxiliary Feedwater System to a valid actuation signal.

B. UNIT STATUS AT TIME OF EVENT

During the period of the event the unit was in Mode 5. Other than described herein, there was no other inoperable equipment that contributed to the event.

C. DESCRIPTION OF EVENT

On October 5, 2012 with Unit 1 in Mode 5 and the residual heat removal (RHR) system in service for reactor coolant system temperature control, activities were in progress in preparation for Mode 4 entry. One of the activities involved restoration of the steam generator level RPS/ESFAS instrumentation from a bypassed condition to normal alignment. During the restoration a steam generator lo-lo level actuation was received and the motor-driven AFW pumps started and two turbine-driven AFW discharge valves stroked full open. Due to the system alignment no water was injected into the steam generators.

During the unit shutdown for refueling outage 1R17, the AFW system had been properly removed from service as required by the controlling procedure. During the refueling outage safety system testing was performed necessitating re-alignment of the system to support testing. At the conclusion of the testing, the AFW system was not restored to the procedurally required configuration. Prior to restoring steam generator level instrumentation, the crew identified that actual steam generator level was below the actuation setpoint but assumed the system was in the proper alignment and did not recognize the AFW system was not in the procedurally required configuration to prevent inadvertent actuations.

D. CAUSE OF EVENT

Upon returning steam generator level instrumentation from a bypassed condition to a normal alignment the effect on the AFW system was not fully evaluated.

E. SAFETY ASSESSMENT

The automatic actuation occurred in a Mode in which the system is not required to perform a credited safety function. At the time of the event, the Residual Heat Removal System was in service providing reactor coolant system temperature control.

**LICENSEE EVENT REPORT (LER)
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NARRATIVE

F. CORRECTIVE ACTION

An operating order was issued to maintain the AFW pumps in a disabled condition while in Mode 5 unless otherwise procedurally directed. In addition, shift supervisor permission and verification from a second SRO is required prior to restoring instrumentation from a bypassed condition to a normal alignment. Procedural revisions are in progress to prevent recurrence and a training gap/needs analysis will be performed.

G. ADDITIONAL INFORMATION

1) Failed Components:

None

2) Previous Similar Events:

Topical Report TR 4-39, Review of Auxiliary Feedwater System Events

3) Energy Industry Identification System Code:

[BA] - Auxiliary / Emergency Feedwater System (PWR)