Seismic Evaluation Guidance

Screening, Prioritization and Implementation Details (SPID) for the Resolution of Fukushima Near-Term Task Force Recommendation 2.1: Seismic

This document does **NOT** meet the requirements of 10CFR50 Appendix B, 10CFR Part 21, ANSI N45.2-1977 and/or the intent of ISO-9001 (1994).

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Product Description

Following the accident at the Fukushima Daiichi nuclear power plant resulting from the March 11, 2011, Great Tohoku Earthquake and subsequent tsunami, the NRC Commission established a Near Term Task Force (NTTF) to conduct a systematic review of NRC processes and regulations and to determine if the agency should make additional improvements to its regulatory system. The NTTF developed a set of recommendations intended to clarify and strengthen the regulatory framework for protection against natural phenomena. Subsequently, the NRC issued a 50.54(f) letter that requests information to assure that these recommendations are addressed by all U.S. nuclear power plants. This report provides guidance for conducting seismic evaluations as requested in Enclosure 1 of the 50.54(f) letter [1]. This 50.54(f) letter requests that licensees and holders of construction permits under 10 CFR Part 50 reevaluate the seismic hazards at their sites against present-day NRC requirements and guidance. Based upon this information, the NRC staff will determine whether additional regulatory actions are necessary.

Objectives

The objective of the work reported in this document is to provide guidance on the performance of plant seismic evaluations, and in particular those intended to satisfy the requirements of NTTF Recommendation 2.1: Seismic.

Approach

The approach taken was to formulate guidance for the seismic evaluations through a series of expert meetings, supplemented by analytical research to evaluate selected criteria. Previous seismic evaluations are described and applied, to the extent applicable. Screening methods are described for evaluating newly calculated seismic hazards against previous site-specific seismic evaluations, as well as to determine the structures, systems, and components (SSCs) that are appropriate to be modeled in a seismic probabilistic risk assessment (SPRA).

A number of public meeting were also held with the NRC during development of the guidance to discuss evaluation criteria and to ensure the guidance met the requirements of NTTF Recommendation 2.1: Seismic.

Results and Findings

This report outlines a process and provides guidance for investigating the significance of new estimates of seismic hazard and, where necessary, performing further seismic evaluations. This guidance is primarily designed for use in responding to the U.S. Nuclear Regulatory Commission's Near Term Task Force (NTTF) Recommendation 2.1: Seismic evaluations. The guidance includes a screening process for evaluating updated site-specific seismic hazard and ground motion response spectrum (GMRS) estimates against the plant safe shutdown earthquake (SSE) and High Confidence of Low Probability of Failure (HCLPF) capacities. It also provides a selected seismic risk evaluation criteria as well as spent fuel pool evaluation criteria.

Applications, Value and Use

The guidance in this report is intended primarily for use by all U.S. nuclear power plants to meet the requirements of NTTF Recommendation 2.1: Seismic. The primary value in this guidance is that it has been reviewed with the NRC and can be applied by all plants to provide a uniform and acceptable industry response to the NRC. Furthermore, the guidance related to seismic evaluations is of value for any seismic risk assessment.

Keywords

Earthquakes
Fukushima
Seismic hazard
Fragilities
SPRA

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List of Acronyms

AEF	annual exceedance frequency
ANS	American Nuclear Society
AOV	air-operated valve
ASME	American Society of Mechanical Engineers
BWR	boiling water reactor
CAV	Cumulative Absolute Velocity
CDF	core damage frequency
CDFM	Conservative Deterministic Failure Margin
CENA	Central and Eastern North America
CEUS	Central and Eastern United States
COLA	Combined Operating License Application
COV	coefficient of variation
ESP	early site permit
FSAR	Final Safety Analysis Report
FRMF	Filtered Random Multi-Frequency
GI	Generic Issue
GIP	Generic Issues Program
GL	Generic Letter
GMPE	ground motion prediction equations
GMRS	ground motion response spectra
HCLPF	High Confidence of Low Probability of Failure
IEEE	Institute of Electrical and Electronics Engineers
IHS	IPEEE HCLPF Spectrum
IPEEE	Individual Pant Examination of Individual Events
ISG	Interim Safety Guide
ISRS	in-structure response spectra
LERF	large-safety release frequency
LHT	Low Hazard Threshold
LMSM	Lumped-mass stick models
MCC	motor control center
MCCB	Molded case circuit breakers

MOV	motor-operated valve
MSVH	Main Steam Valve House
NGA	Next Generation Attenuation
NPP	nuclear power plant
NRC	Nuclear Regulatory Commission
NTTF	Near Term Task Force
PGA	Peak Ground Acceleration
PSD	power spectral density
PSHA	probabilistic seismic hazard analysis
RLE	review level earthquake
RLME	Repeated Large Magnitude Earthquake
RMF	Random Multi-Frequency
RVT	Random Variation Theory
SAMG	Severe Accident Management Guidance
SCDF	seismic core damage frequency
SER	Safety Evaluation Report
SFP	spent fuel pool
SMA	seismic margin assessment
SPID	Screening, Prioritization, and Implementation Details
SPRA	seismic probabilistic risk assessment
SQUG	Seismic Qualification Utilities Group
SSC	structures, systems, and components
SSE	safe-shutdown earthquake
SSHAC	Senior Seismic Hazard Analysis Committee
SSI	soil-structure interaction
TS	Time Series
UHRS	uniform hazard response spectrum
UHS	uniform hazard spectrum
USNRC	United States Nuclear Regulatory Commission
WNA	Western North America
WUS	Western United States
ZPA	zero period acceleration

Section 1: Purpose and Approach

Following the accident at the Fukushima Daiichi nuclear power plant resulting from the March 11, 2011 Great Tohoku Earthquake and subsequent tsunami, the United States Nuclear Regulatory Commission (NRC) established the Near Term Task Force (NTTF) in response to Commission direction. The NTTF issued a report that made a series of recommendations, some of which were to be acted upon "without unnecessary delay." Subsequently, the NRC issued a 50.54(f) letter that requests information to ensure that these recommendations are addressed by all U.S. nuclear power plants (NPPs). The principal purpose of this report is to provide guidance for responding to the request for information in the 50.54(f) Letter, Enclosure 1, Recommendation 2.1: Seismic [1].

Although the guidance in this document is specifically directed at supporting responses to the 50.54(f) letter, much of the guidance is appropriate for elements of any seismic risk evaluation.

Section 1 of this report provides the background on two past seismic programs (IPEEE and GI 199) that are particularly relevant to the 2.1 seismic assessment, and summarizes both the NTTF recommendations and the technical approach intended to support the response to the 2.1 seismic requests. Section 2 characterizes the seismic hazard elements of the response to the information requests. Section 3 contains the ground motion response spectra (GMRS) screening criteria associated with the resolution of the 2.1 seismic issue. Section 4 describes the elements of the recommended seismic hazard and screening report to be submitted to the NRC. Section 5 describes the schedule prioritization for completion of the seismic risk part of the 2.1 seismic program. Section 6 contains the seismic risk evaluation methods for those plants required to conduct these assessments. Finally, Section 7 documents an approach to the evaluation of the seismic integrity of spent fuel pool integrity assessment.

1.1 Background on Seismic Risk Evaluations in the U.S.

The risk posed by seismic events to plants operating in the United States was previously assessed in the mid-1990s as part of the response to the request for an Individual Plant Examination of External Events [2].

Further efforts to understand seismic risks, particularly in light of increased estimates of seismic hazard for some sites, led to the initiation of the Generic Issue 199 program [6]. An understanding of these two programs provides valuable background for the discussion of seismic evaluations related to the current 50.54(f) letter.

1.1.1 Individual Plant Examination of External Events – Seismic

On June 28, 1991, the NRC issued Supplement 4 to Generic Letter (GL) 88-20, "Individual Plant Examination of External Events (IPEEE) for Severe Accident Vulnerabilities," [2]. This supplement to GL 88-20, referred to as the IPEEE program, requested that each licensee identify and report to the NRC all plant-specific vulnerabilities to severe accidents caused by external events. The IPEEE program included the following four supporting objectives:

1. Develop an appreciation of severe accident behavior.
2. Understand the most likely severe accident sequences that could occur at the licensee's plant under full-power operating conditions.
3. Gain a qualitative understanding of the overall likelihood of core damage and fission product releases.
4. Reduce, if necessary, the overall likelihood of core damage and radioactive material releases by modifying, where appropriate, hardware and procedures that would help prevent or mitigate severe accidents.

The following external events were to be considered in the IPEEE: seismic events; internal fires; high winds; floods; and other external initiating events, including accidents related to transportation or nearby facilities and plant-unique hazards. The IPEEE program represents the last comprehensive seismic risk/margin assessment for the U.S. fleet of NPPs and, as such, represents a valuable resource for future seismic risk assessments.

EPRI conducted a research project to study the insights gained from the seismic portion of the IPEEE program [3]. The scope of that EPRI study was to review the vast amounts of both NRC and licensee documentation from the IPEEE program and to summarize the resulting seismic IPEEE insights, including the following:

- Results from the Seismic IPEEE submittals
- Plant improvements/modifications as a result of the Seismic IPEEE Program
- NRC responses to the Seismic IPEEE submittals

The seismic IPEEE review results for 110 units are summarized in the EPRI Report [3]. Out of the 75 submittals reviewed, 28 submittals (41

units) used seismic probabilistic risk assessment (PRA) methodology; 42 submittals (62 units) performed seismic margin assessments (SMAs) using a methodology developed by EPRI [39]; three submittals (three units) performed SMAs using an NRC developed methodology; and two submittals (four units) used site-specific seismic programs for IPEEE submittals.

In addition to the EPRI review of seismic IPEEE insights, the NRC conducted a parallel study. NUREG-1742, "Perspectives Gained from the Individual Plant Examination of External Events (IPEEE) Program," issued April 2002 [4], provides insights gained by the NRC from the seismic part of the IPEEE program. Almost all licensees reported in their IPEEE submittals that no plant vulnerabilities were identified with respect to seismic risk (the use of the term "vulnerability" varied widely among the IPEEE submittals). However, most licensees did report at least some seismic "anomalies," "outliers," or other concerns. In the few submittals that did identify a seismic vulnerability, the findings were comparable to those identified as outliers or anomalies in other IPEEE submittals. Seventy percent of the plants proposed improvements as a result of their seismic IPEEE analyses.

1.1.2 Generic Issue 199

In support of early site permits (ESPs) and combined operating license applications (COLAs) for new reactors, the NRC staff reviewed updates to the seismic source and ground motion models provided by applicants. These seismic updates included new EPRI models to estimate earthquake ground motion and updated models for earthquake sources in the Central and Eastern United States (CEUS), such as those around Charleston, South Carolina, and New Madrid, Missouri. These reviews produced some higher seismic hazard estimates than previously calculated. This raised a concern about an increased likelihood of exceeding the safe-shutdown earthquake (SSE) at operating facilities in the CEUS. The NRC staff determined that, based on the evaluations of the IPEEE program, seismic designs of operating plants in the CEUS do not pose an imminent safety concern. At the same time, the NRC staff also recognized that because the probability of exceeding the SSE at some currently operating sites in the CEUS is higher than previously understood, further study was warranted. As a result, the NRC staff concluded on May 26, 2005 [5] that the issue of increased seismic hazard estimates in the CEUS should be examined under the Generic Issues Program (GIP).

Generic Issue (GI)-199 was established on June 9, 2005 [6]. The initial screening analysis for GI-199 suggested that estimates of the seismic hazard for some currently operating plants in the CEUS have increased. The NRC staff completed the initial screening analysis of GI-199 and held a public meeting in February 2008 [7], concluding that GI-199 should proceed to the safety/risk assessment stage of the GIP.

Subsequently, during the safety/risk assessment stage of the GIP, the NRC staff reviewed and evaluated the new information received with the ESP/COLA submittals, along with NRC staff estimates of seismic hazard produced using the 2008 U.S. Geological Survey seismic hazard model. The NRC staff compared the new seismic hazard data with the earlier seismic hazard evaluations conducted as part of the IPEEE program. NRC staff completed the safety/risk assessment stage of GI-199 on September 2, 2010 [8], concluding that GI-199 should transition to the regulatory assessment stage of the GIP. The safety/risk assessment also concluded that (1) an immediate safety concern did not exist, and (2) adequate protection of public health and safety was not challenged as a result of the new information. NRC staff presented this conclusion at a public meeting held on October 6, 2010 (ADAMS Accession No. ML102950263). Information Notice 2010-018, "Generic Issue 199, Implications of Updated Probabilistic Seismic Hazard Estimates in Central and Eastern United States on Existing Plants," dated September 2, 2010 [9], summarizes the results of the GI-199 safety/risk assessment.

For the GI-199 safety/risk assessment, the NRC staff evaluated the potential risk significance of the updated seismic hazards using the risk information from the IPEEE program to calculate new seismic core damage frequency (SCDF) estimates. The changes in SCDF estimate calculated through the safety/risk assessment performed for some plants lie in the range of 10^{-4} per year to 10^{-5} per year, which meet the numerical risk criteria for an issue to continue to the regulatory assessment stage of the GIP. However, as described in NUREG-1742 [4], there are limitations associated with utilizing the inherently qualitative insights from the IPEEE submittals in a quantitative assessment. In particular, the NRC staff's assessment did not provide insight into which structures, systems, and components (SSCs) are important to seismic risk. Such knowledge is necessary for NRC staff to determine, in light of the new understanding of seismic hazards, whether additional regulatory action is warranted. The GI 199 issue has been subsumed into Fukushima NTTF recommendation 2.1 as described in subsequent sections.

1.2 NRC NTTF Recommendations

The NRC issued an information request on March 12, 2012 related to the Fukushima NTTF recommendations 2.1, 2.3, and 9.3 [1]. The requested seismic information associated with Recommendation 2.1 is stated to reflect:

- Information related to the updated seismic hazards at operating NPPs
- Information based on a seismic risk evaluation (SMA or seismic probabilistic risk assessment (SPRA)), as applicable
- Information that would be obtained from an evaluation of the spent fuel pool (SFP)

The basic seismic information requested by the NRC is similar to that developed for GI-199 as presented in the draft GL for GI-199 [10]. The NRC has identified an acceptable process for responding to the 2.1 seismic requests, which is documented in Attachment 1 to the March 12, 2012 10CFR 50.54(f) letter [1]. The NRC asks each addressee to provide information about the current hazard and potential risk posed by seismic events using a progressive screening/evaluation approach. Depending on the comparison between the re-evaluated seismic hazard and the current design basis, the result is either no further risk evaluation or the performance of a seismic risk assessment. Risk assessment approaches acceptable to the staff, depending on the new hazard estimates, include a SPRA or an "NRC"-type of SMA that was described in NUREG-1407 [11] for IPEEEs, with enhancements.

1.3 Approach to Responding to Information Request for NTTF Recommendation 2.1

The approach described in this report has been developed by EPRI, working with experts from within the nuclear industry, with the intent of identifying reasonable measures that can be employed to reduce the resources that might be required to complete an effective seismic evaluation. More specifically, the approach was designed to constitute a specific path to developing a response to the request for information made in connection with NTTF Recommendation 2.1. This approach reflects careful consideration of the NRC's description of an acceptable approach for the seismic elements of Recommendation 2.1 (documented in Attachment 1 to Seismic Enclosure 1 of the March 12, 2012 Request for Information [1]). In general, the approach described in this report is intended to conform to the structure and philosophy of the nine steps suggested by the NRC and outlined in that attachment. Key elements of the approach are designed to streamline several of these nine steps (summarized below) while still yielding an appropriate characterization of the impact of any change in hazard for the plant being evaluated. Figure 1-1 illustrates the process for employing this approach; it is based on a progressive screening approach and is broken down into four major task areas:

- Seismic Hazard and Site Response Characterization
- GMRS Comparisons and Plant Screening
- Prioritization of Risk Assessments
- Seismic Risk Evaluation

The following paragraphs provide a brief discussion about each individual step in Figure 1-1. The subsequent sections of this guide contain the detailed descriptions of the methods and the documentation associated with this approach.

Step 1. Develop site-specific control point elevation hazard curves over a range of spectral frequencies and annual exceedance frequencies determined from a probabilistic seismic hazard analysis (PSHA).

Step 2. Provide the new seismic hazard curves, the GMRS, and the SSE in graphical and tabular format. Provide soil profiles used in the site response analysis, as well as the resulting soil amplification functions.

Step 3. Utilize a screening process to eliminate certain plants from further review. If the SSE is greater than or equal to the GMRS at all frequencies between 1 and 10 Hz, then addressees may terminate the evaluation (Step 4) after providing a confirmation, if necessary, that SSCs which may be affected by high-frequency ground motion, will maintain their functions important to safety. A similar screening review based on the IPEEE High Confidence of Low Probability of Failure (HCLPF) Spectrum comparison to the GMRS can also be conducted. Diamonds 3a thru 3f outline the overall screening process, and Section 3 provides additional guidance.

Step 4. This step demonstrates termination of the process for resolution of NTTF Recommendation 2.1 for plants whose SSE is greater than the calculated GMRS.

Step 5. Based on criteria described in Section 6.2, perform a SPRA (steps 6a and 7a) or a SMA (steps 6b and 7b). Step 5 also describes the prioritization process for determining completion schedules for the seismic risk assessments.

Step 6a. If a SPRA is performed, it needs to be technically adequate for regulatory decision making and to include an evaluation of containment performance and integrity. This guide is intended to provide an acceptable approach for determining the technical adequacy of a SPRA used to respond to this information request.

Step 6b. If a SMA is performed, it should use a composite spectrum review level earthquake (RLE), defined as the maximum of the GMRS and SSE at each spectral frequency. The SMA should also include an evaluation of containment performance and integrity. The American Society of Mechanical Engineers/American Nuclear Society (ASME/ANS) RA-Sa-2009 [12] provides an acceptable approach for determining the technical adequacy of a SMA used to respond to this information request. In addition, the NRC is generating an Interim Safety Guide (ISG) on the NRC SMA approach that will be acceptable for this 2.1 application [15]~~(*Reference to be added when ISG published*)~~.

Step 7a. Document and submit the results of the SPRA to the NRC for review. The "Requested Information" Section in the main body of Enclosure 1 [1] identifies the specific information that is requested. In

addition, addressees are requested to submit an evaluation of the SFP integrity.

Step 7b. Document and submit the results of the SMA to the NRC for review. The "Requested Information" Section in the main body of Enclosure 1 [1] identifies the specific information that is requested. In addition, addressees should submit an evaluation of the SFP integrity.

Step 8. Submit plans for actions that evaluate seismic risk contributors. NRC staff, EPRI, industry, and other stakeholders will continue to interact to develop acceptance criteria in order to identify potential vulnerabilities.

Step 9. The information provided in Steps 6 through 8 will be evaluated in Phase 2 to consider any additional regulatory actions. ~~*(Note – Phase 2 placeholder, further description to be provided)*~~

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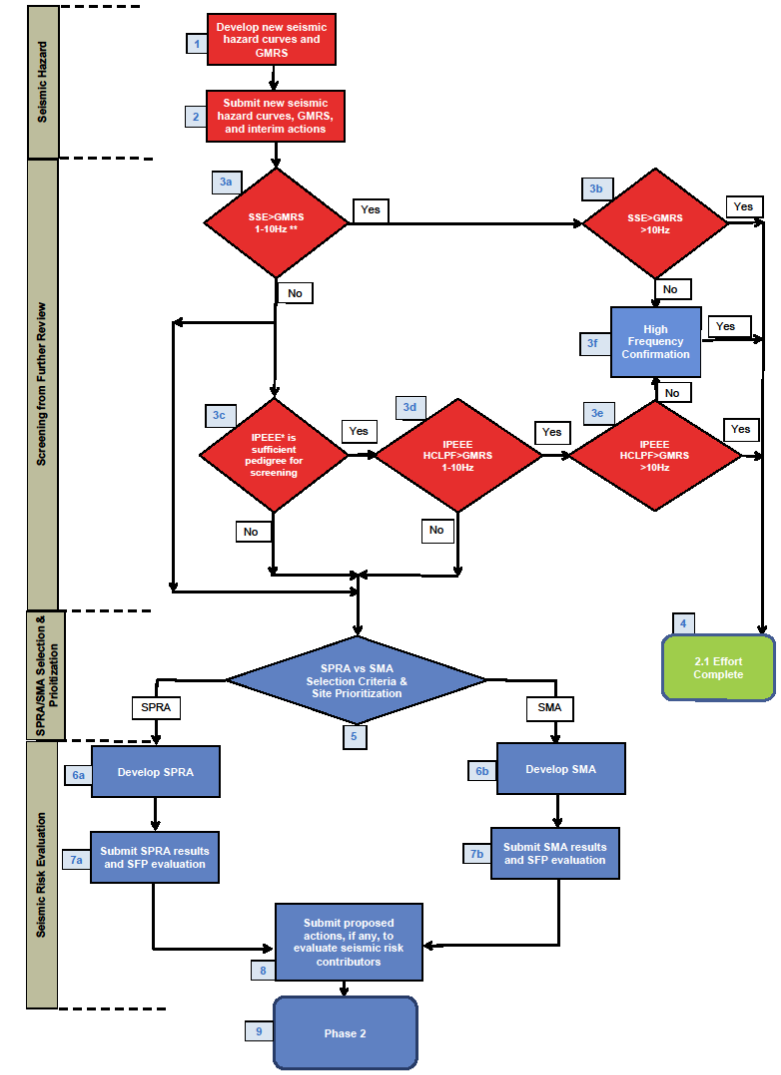

Figure 1-1
Recommended Approach to Respond to Information Request 2.1

Section 2: Seismic Hazard Development

2.1 Introduction and Background

Seismic hazard analysis and the calculation of up-to-date seismic response spectra is the first step to informed evaluations on priorities to mitigate seismic risk. To determine if a reevaluation of seismic risk for a nuclear power plant is appropriate, the comparison of the up-to-date seismic response spectra with the existing plants' seismic design spectra is the next step. Such a comparison should account for both relative and absolute differences between up-to-date seismic response spectra and the existing plants' seismic ruggedness, as characterized by the seismic design spectra.

The first major part of the Seismic Enclosure 1 of the March 12, 2012 Request for Information [1] is to calculate seismic hazard at existing plant sites by first calculating uniform hazard response spectra (UHRS), using up-to-date models representing seismic sources, ground motion equations, and site amplification. From the UHRS results, GMRS are calculated. Figure 2-1 depicts (for illustrative purposes only) the three basic elements of the seismic hazard analysis (seismic source characterization, ground motion attenuation, and site amplification), which will be described in more detail in the sections below.

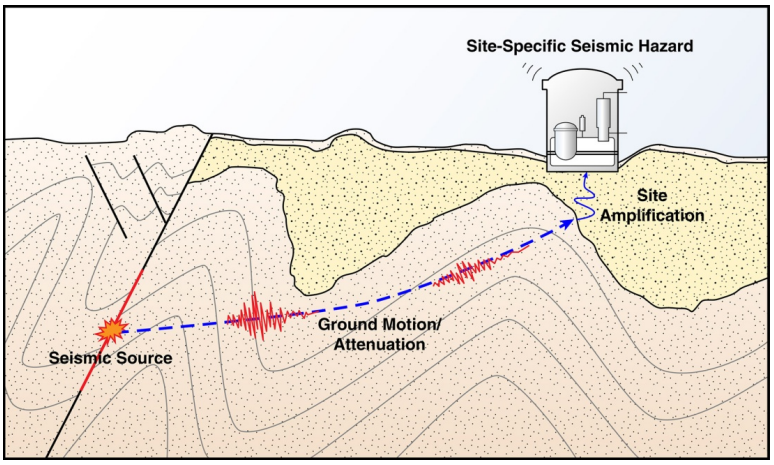

Figure 2-1
Steps to Obtain Site-Specific Seismic Hazard

2.2 Seismic Source Characterization

Seismic Sources for the CEUS – For the region designated the CEUS (United States east of the Rocky Mountains), a regional~~general~~ study was jointly conducted by USNRC, EPRI, and DOE during the period 2009-2011 to develop a comprehensive representation of seismic sources for nuclear plant seismic evaluation purposes. The results were published in 2012 [14] and provide ~~, were reviewed by the USNRC [14], and are~~ an acceptable source characterization model~~set of seismic sources~~ to use for seismic hazard studies [23, p. 115]. This study was conducted as a Senior Seismic Hazard Analysis Committee (SSHAC) Level 3 study [13, 23], meaning that a detailed step-by-step process was used to evaluate data and ~~theories~~ interpretations on earthquake occurrences, their potential locations and sizes, and the rates with which they might occur, and that process was documented and reviewed in a structured way. This ensured that all credible data and interpretations ~~theories~~ were appropriately considered. Specifically, detailed workshops were held that addressed the fundamental technical bases upon which models of seismic sources could be developed, and alternative models, with their technical bases, were defined. This applied to the geometries of seismic sources, as well as to the parameters of the sources (earthquake magnitude distributions, rates of activity, maximum magnitudes, and characteristics of faulting within the earth's crust). Alternative models and parameters were quantitatively weighted to express the credibility of each alternative. A Technical Integration team conducted these analyses and documented the derivation of weights so that a logic-tree approach (alternatives with weights) could be used to characterize the interpretations and their uncertainties. This set of interpretations forms the basis for characterizing the distribution of future earthquake occurrences in the CEUS. Because of

the large regional study area of the CEUS Seismic Source Characterization project, detailed evaluations of geology, topography, and other data in the vicinity of NPPs was not undertaken.

Seismic sources were defined in the CEUS Seismic Source Characterization project in two categories. First were Repeated Large Magnitude Earthquake (RLME) sources, which represent sources where there is evidence of repeated, large-magnitude earthquakes. The two major RLME sources in the CEUS are the New Madrid seismic zone and the Charleston seismic zone. However, the CEUS Seismic Source Characterization project identified additional RLME sources on the basis of paleo-earthquake and other evidence.

The second category of seismic sources were background sources, which are large regions within which earthquakes are modeled as occurring according to an exponential magnitude distribution but where specific faults or causative structures have not been identified. Two sets of background sources were identified based on alternative methods to estimate maximum magnitude, and each set of background sources covers the entire CEUS (and surrounding territory). An updated earthquake catalog was created and used to estimate rates of activity within the sources, the rate of activity varying spatially to reflect the historical occurrences of small and moderate earthquakes. Thus, for example, sub-regions of the CEUS that have experienced relatively many historical earthquakes would have a higher rate of activity than sub-regions that have experienced relatively few historical earthquakes.

For site-specific licensing applications or site-specific safety decisions, these seismic sources would be reviewed on a site-specific basis to determine if they need to be updated. Such evaluations would be appropriate in a licensing application, where focus could be made on site-specific applications. However, for a screening-level study of multiple plants for the purpose of setting priorities, the use of these seismic sources as published is appropriate.

In addition, for applications in a regional study, it is sufficient to include background sources within 320 km (200 miles) of a site, and specifically to include only parts of those background sources that lie within 320 km of the site. This follows the guidance in [18] regarding examination of sources within the “site region” defined as the surrounding 320 km. For RLME sources, it is sufficient to include the New Madrid, Charlevoix, and the Charleston seismic zones if they lie within 1,000 km of a site. Beyond 1,000 km, ground motion equations have not been well-studied, and such distant earthquakes do not generally cause damage to modern engineered facilities. For other RLME sources and sub-regions of background sources with higher rates of activity, it is sufficient to include them in the analysis if they lie within 500 km of a site, based on test hazard results published in the CEUS Seismic Source Characterization project.

Seismic Sources for the WUS– For Western United States (WUS) plants, characterizing ~~designation~~ of seismic sources is much more site-specific. These sites are Diablo Canyon and San Onofre in California, Palo Verde in Arizona, and Columbia in Washington. For the California sites, local faults dominate the seismic hazard; for the Columbia site, local faults, background sources, and subduction zone earthquakes are a consideration. For the Arizona site, background sources and distant faults (including the San Andreas Fault) are important. The development of seismic sources should be made on a site-specific basis for these four sites by conducting a SSHAC Level 3 study [13, 23].

2.3 Ground Motion Attenuation

Ground Motion Estimates for the CEUS – In 2004, EPRI [16] published a set of ground motion prediction equations (GMPEs) for the CEUS, which included both aleatory and epistemic uncertainties. In 2006, EPRI [17] published an updated set of aleatory uncertainties to use with the 2004 equations. These GMPEs estimate the aleatory and epistemic uncertainty in ground motion for the mid-continent region of the CEUS and for the Gulf of Mexico region.

Beginning in 2012, EPRI has been evaluating the 2004-2006 GMPEs in light of new ground motion models published in the technical literature and in light of recorded ground motion data obtained during earthquakes in the CEUS and south-eastern Canada. The overall goals of the project are to determine (a) if the 2004-2006 GMPEs should be updated in light of the new models and data, and (b) if so, how to quantitatively update those GMPEs so they reflect the new information. A decision to update the 2004-2006 GMPEs was confirmed on August 14, 2012, and the updated models are expected in mid-February 2013.

It is anticipated that, as in EPRI 2004-2006, multiple models with weights will be determined for the 2013 updated GMPEs and for the aleatory uncertainties. It is also anticipated that equations will be developed for the two regions (mid-continent and Gulf of Mexico). In cases where the travel path of seismic waves between a potential earthquake source and a site is predominantly in one region, equations for that region should be used. In cases where the travel path crosses from one region to the other, with a substantial fraction of the total travel path of seismic waves in each region, hazard calculations can be made using either the more conservative equations, or using a weighted average of hazard results based on the approximate fraction that seismic waves travel through each region.

Because the EPRI 2012 ground motion update project is proceeding with updating the EPRI 2004-2006 GMPEs, those updated equations, if approved by the NRC, should be used to calculate ground motions for seismic hazard calculations for all CEUS sites for Step 2 “Submit new

seismic hazard curves, GMRS, and interim actions." Otherwise the EPRI 2004-2006 GMPEs should be used.

Currently some CEUS NPPs are developing SPRAs. Consistent with the current SPRA standard requirement of using the most recent seismic hazard information, they are using the EPRI 2004-2006 ground motion attenuation model with the CEUS Seismic Source Characterization model for the seismic hazard portion of their SPRAs. These CEUS NPPs should, in Step 7a, address the effect of the new site hazard based on the updated EPRI 2004-2006 GMPEs.

Ground Motion Estimates for the WUS – In the WUS, earthquake ground motions can be estimated using recorded motions, and the seismic hazard is often dominated by the possible occurrence of a moderate-to-large earthquake at close distances. There are published GMPEs available, the "Next Generation Attenuation," or NGA, equations, but these will be updated in the next several years by the NGA-2 equations. Nuclear plant sites in the WUS should perform a SSHAC Level 3 study [13, 23] in order to make site-specific decisions on which equations are appropriate for their sites or to develop site-specific relationships.

2.4 Site Seismic Response

Every site that does not consist of hard rock should conduct an evaluation of the site amplification that will occur as a result of bedrock ground motions traveling upward through the soil/rock column to the surface. Critical parameters that determine which frequencies of ground motion might experience significant amplification (or de-amplification) are the layering of soil and/or soft rock, the thicknesses of these layers, the initial shear modulus and damping of these layers, their densities, and the degree to which the shear modulus and damping change with increasing ground motion. The methods to calculate possible site amplification are well-established, but at some sites the characterization of the profile and layering is limited. For these sites, analyses must be conducted, as described below, that account for uncertainties in soils and layer properties, and this often results in significant uncertainties in site amplification. This Section also provides a method for defining the elevation(s) for the SSE to GMRS comparison for use in the 2.1 seismic screening.

2.4.1 Site Response for Sites with Limited Data

Many sites, particularly those licensed in the early 1970s, do not have detailed, measured soil and soft-rock parameters to extensive depths. These sites will be handled using the following guidelines (see Appendix B for a more detailed discussion).

Shear-wave Velocity (V_s) – For soil sites where V_s is estimated from compression-wave measurements, or was measured only at shallow

depths, template profiles will be used based on experience with other, well-documented sites. The template profiles will be adjusted and/or truncated to be consistent with measured or estimated V_s in the upper 30 m of soil, called V_{s30}, to obtain a reasonable profile to use for analysis that includes the potential effects on ground motion of soils at large depths.

For firm rock sites (typically underlain by sedimentary rocks) that have little measured V_s data, a V_s profile will be adopted that is consistent with shallow estimates or measurements and that increases with depth using a gradient typical of sedimentary rocks. A consistent gradient has been documented for sedimentary rock sites in various locations around the world, and a profile developed in this way will give reasonable results for the potential effects on ground motion of sedimentary rock at large depths.

For sites with limited, or indirect data on V_s, multiple profiles or base cases should be developed to account for the epistemic uncertainty. Typically three base cases should be developed. To account for the variability in V_s over the scale of the footprint of a NPP, which is treated as an aleatory uncertainty, randomization about the base cases should be implemented. Additional discussion regarding the methodology to incorporate the various types of uncertainty is provided in Appendix B.

Dynamic Soil and Soft-rock Properties – Other soil and soft-rock properties such as dynamic moduli, hysteretic damping, and kappa (a measure of inherent near surface site damping) will be adopted using published models. The same will be done for soil and soft-rock densities, if they have not been measured and reported.

2.4.2 Horizons and SSE Control Point

This Section provides a method for defining the elevation(s) for the SSE to GMRS comparison for use in the 2.1 seismic screening. The SSE to GMRS comparison for 2.1 screening per the 50.54(f) letter are recommended to be applied using the licensing basis definition of SSE control point. The SSE is part of the plant licensing basis which is typically documented in the FSAR. Three specific elements are required to fully characterize the SSE:

- Peak Ground Acceleration
- Response Spectral Shape
- Control Point where the SSE is defined

The first two elements of the SSE characterization are normally available in the part of the Final Safety Analysis Report (FSAR) that describes the site seismicity (typically Section 2.5). The control point for the SSE is not always specifically defined in the FSAR and, as such, guidance is required to ensure that a consistent set of comparisons are made. Most plants have

a single SSE, but several plants have two SSEs identified in their licensing basis (e.g., one at rock and one at top of a soil layer).

For purposes of the SSE-to-GMRS comparisons as part of the 50.54(f) 2.1 seismic evaluations, the following criteria are recommended to establish a logical comparison location:

1. If the SSE control point(s) is defined in the FSAR, use as defined.
2. If the SSE control point is not defined in the FSAR then the following criteria should be used:
   a. For sites classified as soil sites with generally uniform, horizontally layered stratigraphy and where the key structures are soil-founded (Figure 2-2), the control point is defined as the highest point in the material where a safety-related structure is founded, regardless of the shear wave velocity.
   b. For sites classified as a rock site or where the key safety-related structures are rock-founded (Figure 2-3), then the control point is located at the top of the rock.
   c. The SSE control point definition is applied to the main power block area at a site even where soil/rock horizons could vary for some smaller structures located away from the main power block (e.g., an intake structure located away from the main power block area where the soil/rock horizons are different).

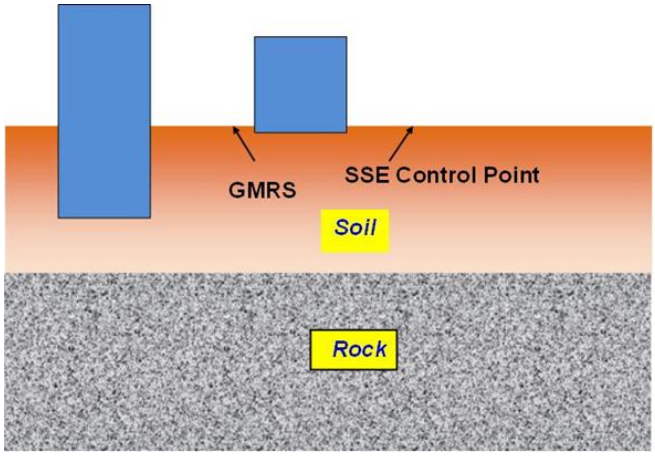

Figure 2-2
Soil Site Example

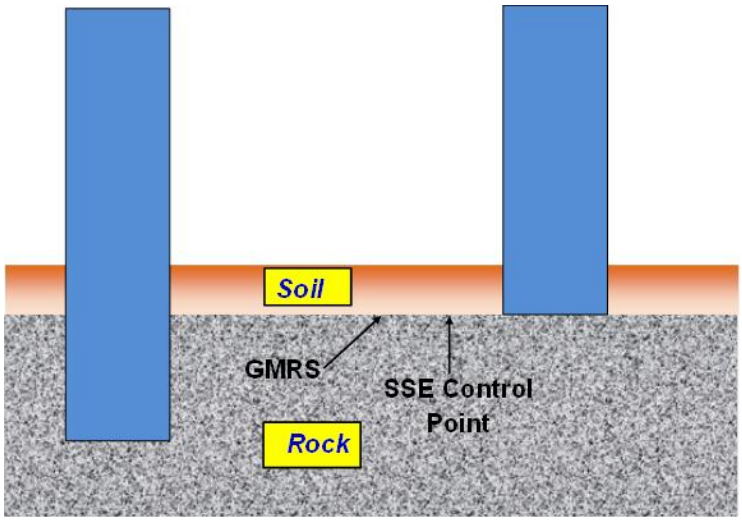

Figure 2-3
Rock Site Example

The basis for the selected control point elevation should be described in the submittal to the NRC. Deviations from the recommendations described above~~below~~ should also be documented.

2.5 Hazard Calculations and Documentation

2.5.1 PSHA and Hazard Calculations

The PSHA will proceed with (1) the CEUS Seismic Source Characterization models [14] or a regional WUS seismic source characterization (Section 2.2 above), with (2) GMPEs for the CEUS or the WUS (Section 2.3 above), and with (3) a site seismic response analysis (quantified as described in Section 2.4 and Appendix B). Several assumptions are appropriate regarding the PSHA calculations as follows:

For CEUS sites, seismic sources should be included for the range of distances indicated in Section 2.2. For WUS sites, the Technical Integration team for the SSHAC Level 3 study with input from the Participatory Peer Review Panel should determine which seismic sources should be included in the PSHA.

As indicated in Section 2.3, for the CEUS the updated EPRI GMPEs should be used for purposes of the 50.54(f) 2.1 seismic evaluations, if approved by the NRC; otherwise, the EPRI 2004-2006 ground motion models [16, 17] should be used. In addition, estimates of ground motion for source-site configurations with seismic wave travel paths across both the mid-continent and Gulf of Mexico regions should be handled as

described in Section 2.3. For the WUS, a SSHAC Level 3 study should be performed to select or develop appropriate GMPEs.

For the purposes of responding to the Seismic Enclosure 1 of the March 12, 2012 Request for Information [1], updates to seismic sources to account for historical seismicity since 2008 (the last year of the earthquake catalog in the CEUS Seismic Source Characterization study) are not required. Similarly, updates to seismic sources to account for more recent earthquakes are not necessary.

The CAV (Cumulative Absolute Velocity) filter developed by EPRI [19] may be applied to account for the damageability of ground motions from small magnitude earthquakes. However, if the CAV filter is applied, the lower-bound magnitude for the PSHA should be set at **M** 4.0, and the CAV model should not be applied for **M** greater than 5.5 (see Attachment 1 to Seismic Enclosure 1 of Reference [1]). In place of the CAV filter a minimum magnitude of **M** 5.0 may be used.

Site amplification factors should be calculated as described in Section 2.4. As discussed in that section, multiple models of site amplification factors (and associated uncertainties) should be developed, indicating the log-mean and log-standard deviation of control-point motion divided by input rock motion, for various spectral frequencies. For input to site hazard calculations, these multiple models should be combined, with weights, to derive the overall log-mean and log-standard deviations of site amplification for each spectral frequency, as described in Appendix B. The soil uncertainties should be incorporated into the seismic hazard calculations using a formulation similar to Eq. (6-5) in [24], wherein the site amplifications (with uncertainties) are incorporated into the hazard integral to estimate the distribution of site amplitudes given earthquake magnitude and distance. The implementation should estimate the distribution of rock amplitude as a function of M and R, and the site amplification (given the rock amplitude) for the value of M at which site amplifications were calculated. This is sufficiently accurate since site amplifications are not highly dependent on M and R.

The control-point elevation seismic hazard curves should be used to calculate a GMRS for the site, using the method of [21]. The GMRS depends, in this calculation, on the 10^{-4} and 10^{-5} spectral accelerations at each spectral frequency. The control point should be defined at the same elevation as the design basis SSE. Given that the site amplification factors are calculated assuming free-surface conditions above the control point, the GMRS will be consistent with that assumption.

2.5.2 Seismic Hazard Data Deliverables

Soil Profile and Properties – A description of the development of the base case profile as it relates to the local geology should be described. In addition, for each base case, the soil profile used to calculate site

amplification factors should be described, including layer boundaries, properties (Vs and density), modulus and damping curves used for each layer, and uncertainties in these properties.

Site Amplification Factors – Site amplification factors should be documented as log-mean amplification factors and log-standard deviations of amplification factors as a function of input rock acceleration, for the spectral frequencies at which GMPEs are defined.

2.5.3 Seismic Hazard Data at Control Points and Base-Rock

Hazard Data at Control Point – Seismic hazard curves should be documented for the control-point elevation corresponding to the mean hazard and common fractiles. These curves should represent seismic hazard at the spectral frequencies for which GMPEs are available. The control-point elevation hazard curves should be represented for annual exceedance frequencies from 10^{-3} to 10^{-7}. Hazard curves should be provided in graphical and tabular format along with the site response amplification function, SSE and GMRS.

The majority of the discussion in Sections 2.4 and 2.5 is based on using site amplification with Method 3 from NUREG/CR-6728 [24] as described in Appendix B. For plants using Method 2 for site amplification in accordance with NUREG/CR-6728, the hard rock seismic hazard curves and the site amplification factors to the control-point elevation should be reported in addition to the control-point elevation hazards noted above.

Section 3: GMRS Comparisons and Screening of Plants

3.1 Background on Screening

Following completion of the updated seismic hazard as described in Section 2, a screening process is needed to determine which plants are required to perform new seismic risk evaluations. The horizontal GMRS calculation discussed/defined in Section 2 is being used to characterize the amplitude of the new seismic hazard at each NPP site, as defined by the NRC [1]. The GMRS should be compared to the horizontal 5% damped SSE as shown in Diamonds 3a and 3b of Figure 1-1. If the SSE is exceeded, then licensees may have the option to perform the screening of the GMRS to the IPEEE HCLPF spectrum (IHS). The IHS is the response spectrum corresponding to the HCLPF level documented from the seismic IPEEE program, as shown in Diamonds 3c through 3e in Figure 1-1. The use of the IHS for screening is contingent upon satisfying specific adequacy criteria, as described in Section 3.3. This screening process, along with examples, is described in more detail in the Sections below.

3.2 SSE Screening Task (SSE-to-GMRS Comparison)

The SSE is the plant licensing basis earthquake and is uniquely defined for each NPP site. The SSE consists of:

- A PGA value which anchors the response spectra at high frequencies (typically 33 Hz for the existing fleet of NPPs),
- A response spectrum shape which depicts the amplified response at all frequencies below the PGA (typically plotted at 5% damping), and
- The control point applicable to the SSE (described in Section 2 of this report). It is essential to ensure that the control point for both the SSE and for the GMRS is the same.

The first step in the SSE screening process is to compare the SSE to the GMRS in the 1 to 10 Hz part of the response spectrum (see Diamond 3a in Figure 1-1). If the SSE exceeds the GMRS in the 1 to 10 Hz region, then a check of the greater than 10 Hz part of the spectrum is performed as shown in Diamond 3b. If the SSE exceeds the GMRS in the greater than 10 Hz region, then no further action is required for NTTF Recommendation 2.1 seismic review (Box 4 in Figure 1-1). If there are

exceedances in the greater than 10 Hz region, then a high-frequency confirmation should be performed (Box 3f in Figure 1-1) as described in Section 3.4.

An example comparison of an SSE with a GMRS is shown in Figure 3-1. In this example, only a high frequency confirmation is needed.

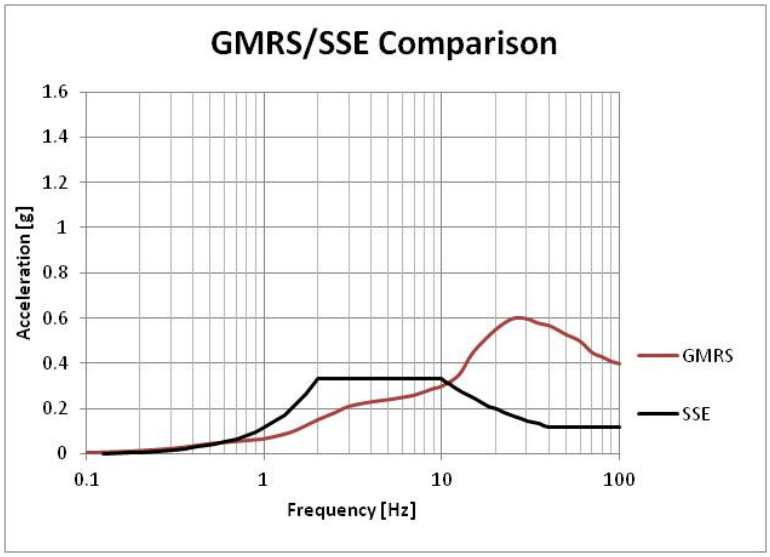

Figure 3-1
Example Comparison of GMRS to SSE (5% Damping)

If the initial review of the SSE to GMRS (Diamond 3a in Figure 1-1) does not demonstrate that the SSE envelops the GMRS in the 1 to 10 Hz region, then, depending upon the nature of the exceedance, the licensees have the option of:

1) Conducting a screening evaluation for narrow band exceedances as described in Section 3.2.1, or
2) Conducting a screening evaluation using the IPEEE HCLPF capacity as described in Section 3.3, or
3) Bypassing the screening evaluations and performing the seismic risk evaluation using either an SPRA or SMA approach, as appropriate, as described in Section 6 of this report.

3.2.1 Special Screening Considerations

There are two special screening considerations:

- GMRS Comparisons and Screening of Plants at Low Seismic Hazard Sites
- Narrow Band Exceedances in the 1 to 10 Hz Range

3.2.1.1 GMRS Comparisons and Screening of Plants at Low Seismic Hazard Sites

A low seismic hazard site is defined herein to be a site where the GMRS peak 5% damped spectral acceleration (SA_p) at frequencies between 1 and 10 Hz do not exceed 0.4g, which is shown in Figure 3-2 as the Low Hazard Threshold (LHT). Because of the low likelihood of any seismically designed SSC being damaged by ground motion with an SA_p less than this LHT, the following relief from having to perform a full SMA or SPRA is considered to be warranted for plants at sites where the GMRS is less than this LHT in the 1 to 10 Hz range.

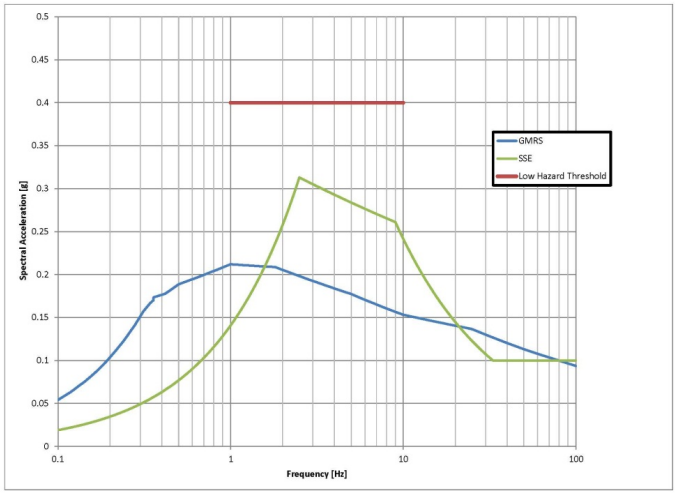

Figure 3-2
Example Comparison of GMRS to SSE and LHT (5% Damping)

Figure 3-2 shows an example where the SSE spectral accelerations exceed the GMRS spectral accelerations at frequencies below 10 Hz except for low frequencies. Because the SSE response spectral accelerations reduce rapidly as frequencies reduce below 2.5 Hz, the situation shown in

Figure 3-2 can occur at low seismic hazard sites. For most SSCs, such exceedance below 2.5 Hz is non-consequential because the fundamental frequency of these SSCs exceeds 2.5 Hz.

Low-frequency exceedances (below 2.5 Hz) at low seismic hazard sites (SA_p less than LHT) do not require a plant to perform a full SMA or SPRA. Instead, it is sufficient to first identify all safety-significant SSCs that are potentially susceptible to damage from spectral accelerations at frequencies below which the highest frequency f_L (f_L < 2.5 Hz) acceleration exceeds the SSE spectral acceleration. Examples of SSCs and failure modes potentially susceptible to damage from spectral accelerations at low frequencies are:

1) Liquid sloshing in atmospheric pressure storage tanks
2) Very flexible distribution systems with frequencies less than f_L
3) Sliding and rocking of unanchored components
4) Fuel assemblies inside the reactor vessel
5) Soil liquefaction

After identifying all safety-significant SSCs that are potentially susceptible to lower frequency accelerations, new HCLPF capacities using the GMRS shape can be computed for these potentially low-frequency susceptible SSCs. T~~t~~he HCLPF~~I~~HS to GMRS seismic margin needs to be computed and reported. As long as the HCLP~~I~~HSF is greater than the ~~/~~GMRS ~~ratio between 1 and 10 Hz~~ for all of these potentially low-frequency susceptible SSCs ~~exceeds unity~~, the plant is screened out from having to perform additional seismic evaluations.

If the IPEEE HCLPF[1] capacity evaluations are considered to be sufficient for screening (as described in Section 3.3.1), the IPEEE HCLPF response spectral accelerations may be used for this HCLPF/GMRS comparison for screening potentially low-frequency susceptible SSCs at low seismic hazard sites. The IPEEE HCLPF response spectral accelerations also reduce rapidly as frequencies reduce below 2.5 Hz so that the GMRS spectral accelerations might also exceed the HCLPF spectral accelerations at low frequencies. In this case, new HCLPF capacities can be computed for these potentially low-frequency susceptible SSCs using the GMRS response spectrum shape instead of the IPEEE response spectrum.

3.2.1.2 Narrow Band Exceedances in the 1 to 10 Hz Range

If the GMRS exceeds the SSE in narrow frequency bands anywhere in the 1 to 10 Hz range, the screening criterion is as follows: In the 1 to 10 Hz range, a point on the GMRS may fall above the SSE by up to 10% provided

[1] IPEEE based screening is not applicable to Spent Fuel Pools because they were not included in the IPEEE evaluations. See Section 7 for Spent Fuel Pool evaluation criteria.

the average ratio of GMRS to SSE in the adjacent 1/3 octave bandwidth (1/6 on either side) is less than unity. There may be more than one such exceedance point above the SSE in the 1 to 10 Hz range provided they are at least one octave apart. Figure 3-3 shows an example of this narrow-band criterion. If the GMRS meets the criteria, no SMA or SPRA is required for the NTTF Recommendation 2.1 seismic review.

If the IPEEE HCLPF[11] capacity evaluations are considered to be of sufficient quality for screening, the IPEEE HCLPF response spectral accelerations may be used for a HCLPF/GMRS comparison in narrow frequency bands. In this case, the SSE is replaced by the IPEEE-HCLPF spectrum to determine if a plant can be screened-out from further seismic review.

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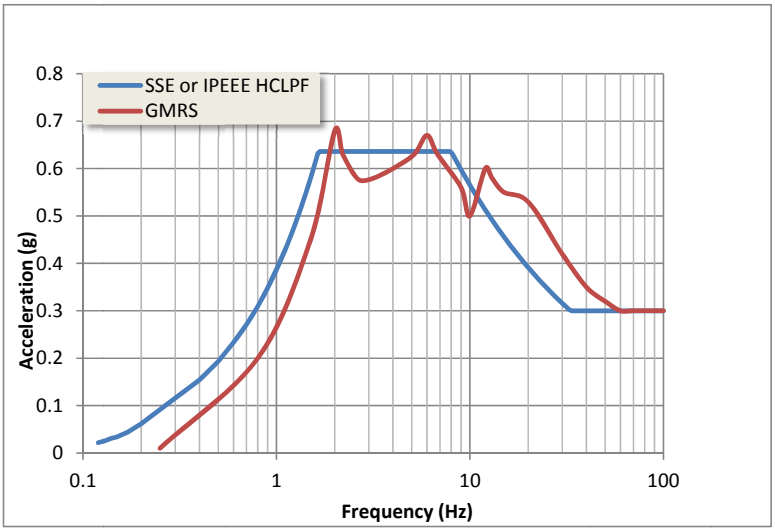

Figure 3-3
Screening – Example Narrow Exceedances at 2 Hz and 6 Hz (5% Damping)

3.3 IPEEE Screening Task

The second method to demonstrate plant seismic adequacy based on screening from further review consists of a comparison of the GMRS to the IPEEE HCLPF spectrum, which is described in Section 3.3.2 below. The use of the IPEEE HCLPF spectrum in the screening process is depicted in Boxes 3c, 3d, and 3e in Figure 1-1. Note that IPEEE screening is not applicable to SFPs because SFPs were not included in the scope of IPEEE evaluations [11]. See Section 7 for SFP evaluation criteria.

For plants that conducted an SPRA, focused scope SMA, or full scope SMA during the IPEEE, the screening is an optional approach that

consists of the comparison of the IPEEE HCLPF spectrum (IHS) to the new GMRS. If the IPEEE HCLPF is used for screening, the IPEEE will be required to pass an adequacy review (Diamond 3c in Figure 1-1). If the IPEEE demonstrates sufficient quality, the next step in this screening process is to compare the IHS to the GMRS in the 1 to 10 Hz part of the response spectrum (see Diamond 3d in Figure 1-1). If the IHS exceeds the GMRS in the 1 to 10 Hz region, then a check of the greater than 10 Hz part of the spectrum is performed, as shown in Diamond 3e. If the IHS exceeds the GMRS in the greater than 10 Hz region, then no further action is required for the NTTF 2.1 seismic review (Box 4 in Figure 1-1). If there are exceedances in the greater than 10 Hz region, then a high-frequency confirmation should be performed (Box 3f in Figure 1-1) as described in Section 3.4.

3.3.1 IPEEE Adequacy

Background

Seismic risk assessments performed as part of the Individual Plant Examination of External Events (IPEEE) for Severe Accident Vulnerabilities (Generic Letter 88-20, Supplement 4) [2] that demonstrate plant capacity to levels higher than the new GMRS can be used to "screen out" plants, provided they meet certain criteria, in which case these plants would not need to perform new seismic risk analyses. IPEEE submittals using either SPRA or SMA analyses can be considered for screening, but in either case the analysis must have certain attributes to be considered for review by the NRC staff.

Use of IPEEE Results for Screening

Certain criteria are necessary if licensees choose to screen a facility based on IPEEE results. The criteria for screening have been grouped into four categories:

- General Considerations
- Prerequisites
- Adequacy Demonstration
- Documentation

Responses to the items in the Prerequisite and Adequacy Demonstration categories should be provided in the hazard submittal to the NRC.

General Considerations

IPEEE reduced scope margin assessments cannot be used for screening. Focused scope margin submittals may be used after having been enhanced to bring the assessment in line with full scope assessments. The enhancements include (1) a full scope detailed review of relay chatter for

components such as electric relays and switches, and (2) a full evaluation of soil failures, such as liquefaction, slope stability, and settlement.

The spectrum to be compared to the GMRS for screening purposes should be based on the plant-level HCLPF actually determined by the IPEEE and reported to the NRC. If this is less than the review level earthquake (RLE) spectrum, then the RLE must be shifted appropriately to reflect the actual achieved HCLPF. In cases where modifications were required to achieve the HCLPF submitted in the IPEEE, verify that the changes were implemented (and describe the current status) in the submittal. This information is also required as part of the Recommendation 2.3 seismic walkdown. Similarly, the uniform hazard spectrum (UHS) for IPEEE SPRA should be anchored at the plant-level HCLPF.

Prerequisites

Responses to the following items should be provided with the hazard evaluation. In order to use the IPEEE analysis for screening purposes and to demonstrate that the IPEEE results can be used for comparison with the GMRS:

1) Confirm that commitments made under the IPEEE have been met. If not, address and close those commitments.
2) Confirm whether all of the modifications and other changes credited in the IPEEE analysis are in place.
3) Confirm that any identified deficiencies or weaknesses to NUREG-1407 [11] in the plant specific NRC Safety Evaluation Report (SER) are properly justified to ensure that the IPEEE conclusions remain valid.
4) Confirm that major plant modifications since the completion of the IPEEE have not degraded/impacted the conclusions reached in the IPEEE.

If any of the four above items are not confirmed and documented in the hazard submittal to the NRC, then the IPEEE results may not be adequate for screening purposes even if responses are provided to the adequacy criteria provided below.

Adequacy Demonstration

The following items, and the information that should be provided, reflect the major technical considerations that will determine whether the IPEEE analysis, documentation, and peer review are considered adequate to support use of the IPEEE results for screening purposes.

With respect to each of the criteria below, the submittal should describe the key elements of (1) the methodology used, (2) whether the analysis was conducted in accordance with the guidance in NUREG-1407 [11] and other applicable guidance, and (3) a statement, if applicable, as to

whether the methodology and results are adequate for screening purposes. Each of the following should be addressed in the submittal to the NRC.

1) Structural models and structural response analysis (use of existing or new models, how soil conditions including variability were accounted for)
2) In-structure demands and ISRS (scaling approach or new analysis)
3) Selection of seismic equipment list or safe shutdown equipment list
4) Screening of components
5) Walkdowns
6) Fragility evaluations (generic, plant-specific analysis, testing, documentation of results)
7) System modeling (diversity of success paths, development of event and fault trees, treatment of non-seismic failures, human actions)
8) Containment performance
9) Peer review (how peer review conducted, conformance to guidance, peer review membership, peer review findings and their disposition)

Documentation

Licensees that choose to implement the use of the IPEEE results for screening purposes should provide a response for each of the criteria in the Prerequisite and Adequacy Demonstration categories in their hazard submittal to the NRC. Licensees should also provide an overall conclusion statement asserting that the IPEEE results are adequate for screening and that the risk insights from the IPEEE are still valid under current plant configurations. The information used by each licensee to demonstrate the adequacy of the IPEEE results for screening purposes should be made available at the site for potential staff audit.

3.3.2 Development of HCLPF Spectrum

The IHS is developed directly from the plant HCLPF capacity established in the IPEEE program. The IPEEE-reported HCLPF values were typically calculated by each plant during the 1990s and documented in the IPEEE submittal reports sent to the NRC by the licensees. These HCLPF values for many of the plants are also documented in NUREG-1742, "Perspectives Gained from the Individual Plant Examination of External Events (IPEEE) Program," April 2002 [4]. For those plants that performed an SMA, the IHS is anchored to the lowest calculated HCLPF of any SSC, and the shape of the IHS is consistent with the RLE used for the SMA (typically the NUREG/CR- 0098 shape). For those plants that conducted an SPRA as part of the IPEEE program, a plant HCLPF value was typically calculated (or can be calculated) from the plant core damage frequency (CDF) and the IHS should be anchored at that value. The shape

of the IHS should correspond to the UHS associated with the seismic hazard utilized within the SPRA. Typically, the shapes of the UHS are similar between the 10^{-4} and the 10^{-5} return period UHS and, thus, either shape could be used for the purpose of generating the IHS. These two return periods are considered to be the appropriate ones for use in the generation of the IHS since the cumulative distribution of the contribution to the CDF has typically been shown to be centered in this return period range.

3.3.3 Comparison of IPEEE HCLPF Spectrum to GMRS

An example of the comparison of a GMRS to the IHS is shown in Figure 3-4. The IHS exceeds the GMRS in the 1 to 10 Hz range, and thus the lower frequency criteria (Diamond 3d of Figure 1-1) have been met. However, for this example, the higher frequency criteria (Diamond 3e in Figure 1-1) have not been met since the GMRS exceeds the IHS in this range. It is noted that (a) the control point for the IHS will typically be defined in a similar way as for the SSE, which is described in Section 2.4.1, and (b) the treatment of Narrow Band Exceedance is the same as discussed in Section 3.2.1 for SSE.

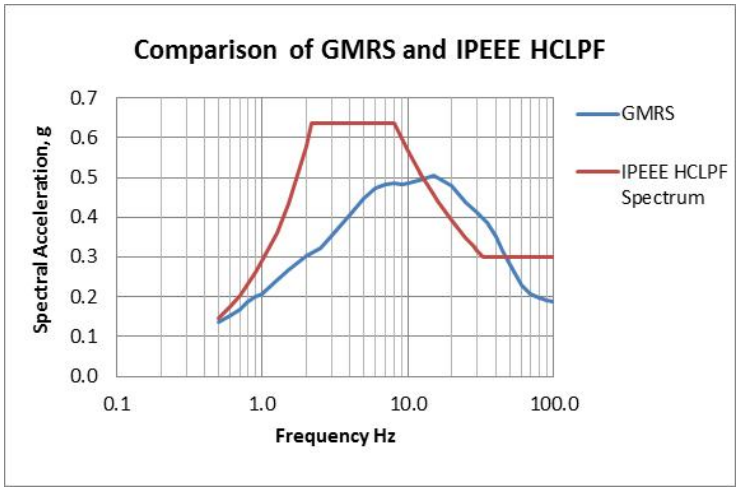

Figure 3-4
Example Comparison of GMRS to IHS (5% Damping)

3.4 Treatment of High-Frequency Exceedances

Equipment important to safety within operating NPPs has been seismically qualified for the SSE defined for each plant. The equipment has also been evaluated, in general, for a RLE under each plant's IPEEE program. The SSE and RLE ground motions, however, do not typically include significant frequency content above 10 Hz. Seismic hazard studies

conducted in the late 1990s developed UHS that had spectral peaks occurring in the 20 to 30 Hz range. EPRI Report NP-7498, "Industry Approach to Severe Accident Policy Implementation," November 1991 [26], included an appendix titled "Recommended Procedures to Address High-Frequency Ground Motions in Seismic Margin Assessment for Severe Accident Policy Resolution." This appendix reviewed the bases for concluding that high-frequency motions were, in general, non-damaging to components and structures that have strain- or stress-based potential failures modes. It concluded that components, such as relays and other devices subject to electrical functionality failure modes, have unknown acceleration sensitivity for frequencies greater than 16 Hz. Thus, the evaluation of high-frequency vulnerability was limited to components that are subject to intermittent states.

In the IPEEE program, the consideration of high-frequency vulnerability of components was focused on a list of "bad actor" relays mutually agreed to by the industry and the NRC, with known earthquake or shock sensitivity. These specific model relays, designated as low ruggedness relays were identified in EPRI Report 7148, "Procedure for Evaluating Nuclear Power Plant Relay Seismic Functionality," December 1990 [27]. Rather than considering high-frequency capacity vs. demand screening, relays on this list were considered program outliers and were evaluated using circuit analysis, operator actions, or component replacements.

EPRI published the following reports during initial new plant licensing activities to provide additional information regarding the potential high-frequency vulnerability of NPP SSCs:

- EPRI Report 1015108, "Program on Technology Innovation: The Effects of High-Frequency Ground Motion on Structures, Components, and Equipment in Nuclear Power Plants," June 2007 [28].
- EPRI Report 1015109, "Program on Technology Innovation: Seismic Screening of Components Sensitive to High-Frequency Vibratory Motions," October 2007 [29].

Report 1015108 [28] summarized a significant amount of empirical and theoretical evidence, as well as regulatory precedents, that support the conclusion that high-frequency vibratory motions above about 10 Hz are not damaging to the large majority of NPP structures, components, and equipment. An exception to this is the functional performance of vibration sensitive components, such as relays and other electrical and instrumentation devices whose output signals could be affected by high-frequency excitation. Report 1015109 [29] provided guidance for identifying and evaluating potentially high-frequency sensitive components for plant applications that may be subject to possible high-frequency motions.

In response to the current NTTF activities, EPRI has established a test program to develop data to support the high frequency confirmation in Step 3f of Figure 1-1 as well as fragility data for a SPRA (Step 6a) or SMA (Step 6b) of Figure 1-1 for potential high-frequency sensitive components. The test program will use accelerations or spectral levels that are sufficiently high to address the anticipated high-frequency in-structure and in-cabinet responses of various plants. Therefore, it will not be necessary for those plants where GMRS > SSE or IHS only above 10 Hz to perform dynamic analysis of structures to develop ISRS.

3.4.1 Scope of High-Frequency Sensitive Components

The following types of failure modes of potentially high-frequency sensitive components and assemblies have been observed in practice:

- Inadvertent change of state
- Contact chatter
- Change in output signal or set-point
- Electrical connection discontinuity or intermittency (e.g., insufficient contact pressure)
- Mechanical connection loosening
- Mechanical misalignment/binding (e.g., latches, plungers)
- Cyclic strain effects (e.g., cracks in solder joints)
- Wiring not properly restrained
- Inadequately secured mechanical fasteners and thumb screw connections

These failure modes are considered below to determine the appropriate scope of potentially high-frequency sensitive components requiring additional information to perform the NTTF 2.1 seismic screening in Figure 1-1, Step 3f.

3.4.1.1 EPRI 1015109 Potentially High Frequency Sensitive Components

EPRI Report 1015109 [29] reviewed potentially high-frequency sensitive components and recommended change of state, contact chatter, signal change/drift, and intermittent electrical connections as the most likely failure modes. These are the first four failure modes highlighted in the above list.

Failures resulting from improper mounting design, inadequate design connections and fasteners, mechanical misalignment/binding of parts, and the rare case of subcomponent mechanical failure, are associated with the same structural failure modes as those experienced during licensing

basis qualification low frequency testing conducted in accordance with the Institute of Electrical and Electronics Engineers (IEEE) Standard 344 [25]. Because the equipment experiences higher stresses and deformations when subjected to low-frequency excitation, these failure modes are more likely to occur under the low-frequency qualification testing.

The evaluation of potentially high-frequency sensitive components in new plants was therefore directed to mechanically actuated bi-stable devices, such as relays, contactors, switches, potentiometers and similar devices, and those components whose output signal or settings (set-points) could be changed by high-frequency vibratory motion. Table 3-1 shows the components identified in EPRI Report 1015109 [29] as being potentially sensitive to high-frequency motion.

3.4.1.2 AP1000 Potentially High Frequency Sensitive Equipment

During licensing reviews for the AP1000, Westinghouse and the NRC identified a broader list of potentially high-frequency sensitive components and assemblies (Table 3-2) to be evaluated in the AP1000 Design Control Document [30].

Table 3-1
EPRI 1015109 Potentially High Frequency Sensitive Items

▪ Electro-mechanical relays (e.g., control relays, time delay relays, protective relays) ▪ Circuit breakers (e.g., molded case and power breakers – low and medium voltage) ▪ Control switches (e.g., benchboard, panel, operator switches) ▪ Process switches and sensors (e.g., pressure, temperature, flow, limit/position)	▪ Electro-mechanical contactors (e.g., MCC starters) ▪ Auxiliary contacts (e.g., for MCCBs, fused disconnects, contactors/starters) ▪ Transfer switches (e.g., low and medium voltage switches with instrumentation) ▪ Potentiometers (without locking devices) ▪ Digital/solid state devices (mounting and connections only)

The primary difference between the list of components in EPRI 1015109 [29] and the AP1000 list [30] is that the EPRI 1015109 list is focused on potentially sensitive subcomponents, and the AP1000 list is focused on assemblies that would include those subcomponents. For example, the potentially sensitive parts of a Battery Charger or a 250 Vdc Motor Control Center are the relays, switches, and contactors noted in the EPRI 1015109 component list [29]. Therefore, evaluating the potential

sensitivity of the items in the EPRI 1015109 list would also address the items in the AP1000 list.

Three key exceptions on the AP1000 list [30] are transformers, batteries, and valves (motor-operated valves (MOVs), air-operated valves (AOVs), solenoid valves (SVs). Transformers are primarily passive systems with strain- or stress-based potential failures modes. Some transformers may include subcomponents on the EPRI 1015109 list [29], but they would be addressed as noted above.

Battery cells have a material aging phenomenon that occurs over time. There is no indication that cell electrical degradation is influenced by the frequency content of the cell support motion being either high-frequency or low-frequency. Batteries do not fail during support motion, but rather fail to produce the rated amp-hour capacity following the support motion. It is judged that the post-earthquake electrical capacity is a function of cell age and the RMS acceleration level of the input motion rather than the frequency content of the motion. Batteries that are less than ten years in age would not experience post-earthquake degradation due to cell shaking.

Valves have been subjected to significant high-frequency test motions due to Boiling Water Reactor (BWR) hydrodynamic loads and have not demonstrated high frequency unique sensitivities. EPRI Report 1015108 [29] provides an example of previous MOV operator combined seismic and BWR hydrodynamic qualification testing with inputs up to 100 Hz. This example valve operator is the same as used in other plant designs. These types of tests also show that additional high frequency content does not affect equipment function. In addition, line mounted valves and operators are subjected to 5-100 Hz sine sweep vibration testing as part of normal valve qualification to simulate normal plant induced vibration environments.

Table 3-2
AP1000 Potentially High Frequency Sensitive Items

▪ 125V Batteries ▪ 250Vdc Distribution Panels ▪ Fuse Panels ▪ Battery Disconnect Switches ▪ 250Vdc Motor Control Centers ▪ Regulating Transformers ▪ 6.9KV Switchgear ▪ Level Switches (Core Makeup Tank, Containment Flood) ▪ Radiation Monitors (Containment High Range Area, Control Room Supply Air) ▪ Transmitters (Flow, Level, Pressure, Differential Pressure) ▪ Control Room (Workstations, Switch Station, Display Units) ▪ Motor Operated Valves (Motor Operators, Limit Switches) ▪ Air Operated Valves (Solenoid Valves, Limit Switches)	▪ Battery Chargers ▪ 120Vdc Distribution Panels ▪ Fused Transfer Switches ▪ Termination Boxes ▪ 250Vdc Switchboard ▪ Inverters ▪ Reactor Trip Switchgear ▪ Neutron Detectors (Source Range, Intermediate Range, Power range) ▪ Speed Sensors (Reactor Coolant Pump) ▪ Protection and Safety Monitoring Systems (System Cabinets, Transfer Switches, Neutron Flux Preamplifiers, High Voltage Distribution Boxes) ▪ Other Valves (Squib [Explosive Opening] Operators, Limit Switches)

3.4.1.3 Component Types to be Evaluated

The list of component types to be evaluated in the above noted high frequency test program was developed based on the reviews described in Sections 3.4.1.1 and 3.4.1.2 and is provided in Table 3-3. A subset of these component types are used in the Phase 1 testing effort described below. The complete list of component types in the table will be considered in a follow-on Phase 2 testing effort.

Test samples will be selected from the component types in Table 3-3 to represent the components installed in operating nuclear power plants. A review of selected utility components lists will be used to inform the test sample selection.

Table 3-3
High Frequency Confirmation Component Types

▪ Electro-mechanical relays (e.g., control relays, time delay relays,	▪ Electro-mechanical contactors (e.g., MCC starters)

protective relays) ▪ Circuit breakers (e.g., molded case and power breakers – low and medium voltage) ▪ Control switches (e.g., benchboard, panel, operator switches) ▪ Process switches and sensors (e.g., pressure, temperature, flow, limit/position)	▪ Auxiliary contacts (e.g., for MCCBs, fused disconnects, contactors/starters) ▪ Transfer switches (e.g., low and medium voltage switches with instrumentation) ▪ Potentiometers (without locking devices)

3.4.2 Phase 1 Testing

The high-frequency test program consists of two phases. The first phase pilot effort has focused on (1) developing a recommended high-frequency test protocol to be used in the full test program, and (2) acquiring sufficient data to allow development of criteria for comparison of fragility levels obtained using high-frequency wide-band and narrow-band motions.

3.4.2.1 Phase 1 Test Samples

The components included in the Phase 1 test program were selected to provide a representative sample of the types of components listed in Section 3.4.1.3, as well as a variety of expected seismic capacity levels. The list of components used for Phase 1 testing is provided in Table 3-4.

Table 3-4
Phase 1 Test Samples

▪ Electro-mechanical relay (600V industrial control relay) ▪ Electro-mechanical relay (pneumatic timing relay) ▪ Electro-mechanical contactor (with auxiliary and overload contacts) ▪ Electro-mechanical relay (lockout relays, two configurations considered) ▪ Electro-mechanical relay (auxiliary relay - hinged armature)	▪ Electro-mechanical relay (socket mounted control relay) ▪ Electro-mechanical relay (300V industrial control relay) ▪ Electro-mechanical relay (600V control relay – prior HF testing history) ▪ Electro-mechanical relay (induction disk protective relay) ▪ Process switch (pressure switch)

3.4.2.2 Phase 1 Testing Protocol

A number of test parameters were investigated in Phase 1, as described below.

Primary Frequency Range of Interest

For the component types listed in Section 3.4.1.3, licensing basis seismic qualification testing is typically performed over a frequency range up to 33 Hz. For floor- or wall-mounted components, in-structure response spectra typically peak in the 4 to 10 Hz range and reach the Zero Period Acceleration (ZPA) in the 15 to 20 Hz range. For in-cabinet mounted components, IEEE C37.98 [31] recommends a response spectrum shape with peak spectral accelerations in the 4 to 16 Hz range and a ZPA at 33 Hz.

Some of the new ground motion estimates have peak accelerations in the 25 to 30 Hz range, which may produce significant in-structure or in-cabinet motions in the 20 to 40 Hz range. Figure 3-5 shows an example ground motion where in-structure and in-cabinet high-frequency motions may be significant.

Because licensing basis seismic qualification testing adequately addresses the lower frequency range, the high-frequency test program will focus on this higher frequency range. The primary focus of the high-frequency testing program is the 20 to 40 Hz frequency range. Phase 1 testing initially considered a broader frequency range of 16 to 64 Hz to insure that the focus on the 20-40 Hz range is sufficient.

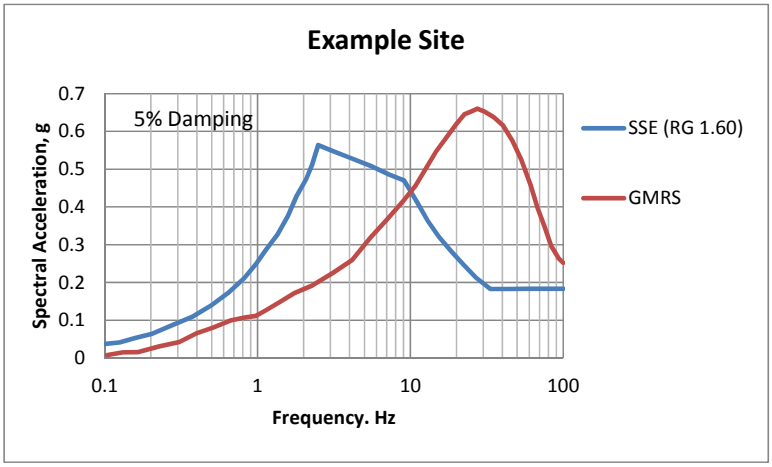

Figure 3-5
Example High Frequency Ground Motion Response Spectrum

Test Input Motions

Three types of test input motions were investigated in Phase 1: sine sweeps, random multi-frequency (RMF) motions and filtered RMF motions. In each case, the input motions were increased in amplitude until either the components failed the acceptance criteria (typically 2 ms contact chatter per ANSI C37.98 [31]), or had anomalous behavior, or the test machine limits were reached.

Sine Sweep Input Motions – This test series used single-axis sine sweep inputs with constant acceleration levels over the 16 to 64 Hz range. The components were tested in each primary direction (e.g., front-to-back, side-to-side, vertical) in the de-energized (non-operate) state with subsequent tests in the energized (operate) state. The objective of this test series was to develop a plot of chatter threshold frequency vs. peak input motion acceleration as a means of displaying the regions of high-frequency sensitivity for each component.

RMF Input Motions – This test series used wide-band multi-frequency tri-axial independent random motions with response spectra covering three separate amplified frequency ranges as shown in Figure 3-6. The three frequency ranges were 16 to 32 Hz, 24 to 48 Hz, and 20 to 40 Hz. The general shape of the amplified spectral region was patterned after the normalized test shape from IEEE C37.98 [31] with the peak acceleration region being 2.5 times the ZPA, but with the frequency ranges shifted as shown in Figure 3-6. A set of three motions was generated for each frequency range. Each axis of motion of each set was independent but had the same general response spectrum shape and amplitude. The purpose of these tests was to determine the fragility level of each device associated with each set of RMF motions for a given frequency range.

Filtered Random Multi-Frequency (FRMF) Input Motions – This test series used wide-band multi-frequency independent random input motions along two primary axes with a set of narrow-band filtered motions along the third axis as depicted in Figure 3-7. The narrow-band motions were applied along the third axis, one at a time, at the indicated 1/6 octave frequencies between 17.8 and 44.9 Hz (21.2 Hz, 23.8 Hz, 26.7 Hz, 30.0 Hz, 33.6 Hz, 37.8 Hz). The RMF motions applied to the other two axes had strong motion frequency range of 17.8 Hz to 44.9 Hz. The FRMF motions were applied separately in the component front-to-back direction and the side-to-side direction. Note that the FRMF testing was intended to simulate either in-structure response or high frequency local panel in-cabinet response, which is expected to be dominated by front-to-back or side-to-side responses; therefore, the filtered motions were limited to those two directions. Each filtered motion had the appearance of multiple sine-beat motions superimposed on a wide band random backbone motion. The purpose of these tests was to determine the fragility level of each device associated a given FRMF motion. The comparison of the fragility response spectra for both the FRMF and RMF

motion allows a 'clipping factor' to be defined that can be used to convert an in-structure or in-cabinet demand (response spectrum) to an effective wide band motion for comparison to a RMF fragility test spectrum.

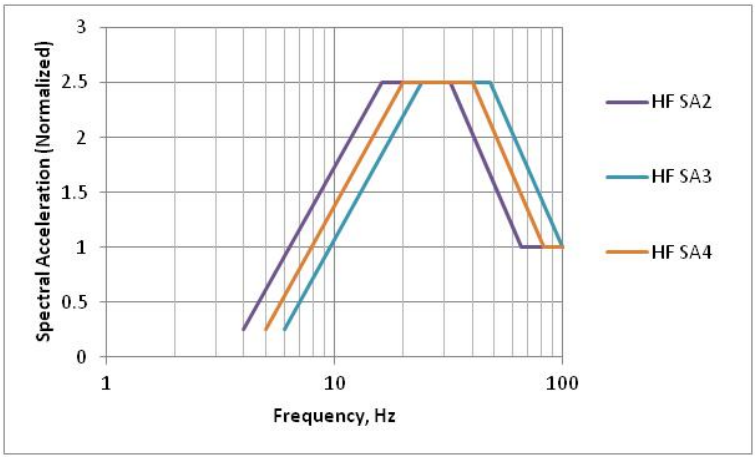

Figure 3-6
Random Multi-Frequency Test Input Motions

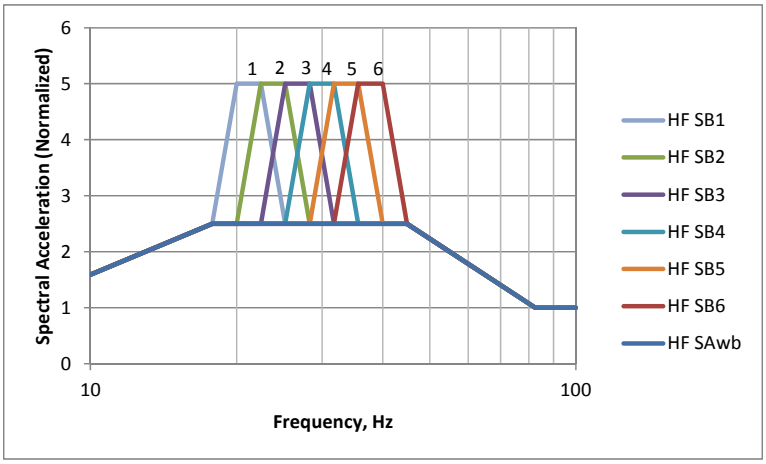

Figure 3-7
Filtered Random Multi-Frequency Test Input Motions

3.4.2.3 Phase 1 Test Results

Component High Frequency Sensitivity

No chatter or change of state occurred for any Phase 1 relay in the energized state for any input motion, thus a relay in the energized state is considered to not be frequency sensitive

Four devices did not have any chatter in the de-energized state for the highest input levels tested. These devices may be considered as not frequency sensitive. Two devices were expected to not have any high-frequency sensitivity based on the high demonstrated low frequency ruggedness (12.5 g spectral), however, these models had anomalous behavior in the greater than 30 Hz range. The remaining 5 relay models demonstrated various high-frequency sensitivities for the de-energized state.

Test Input Motion Results

The general conclusions from each of the three test input motion types are described below.

Sine Sweep Input Motions – The sine sweep tests were primarily exploratory tests but they did not appropriately simulate earthquake motions at the component mounting locations and at high amplitudes, they over predicted component sensitivity.

RMF Input Motions – The RMF tests over the three peak frequency ranges (16 to 32 Hz, 20 to 40 Hz, and 24 to 48 Hz) proved to be efficient to perform, and effective at identifying high frequency component sensitivity. Some of the Phase 1 test components were a little more sensitive to the test motions from 16 to 32 Hz than the other two test ranges; however, this was due to the lower frequency energy included in those tests. That lower frequency energy is already included standard relay testing performed using the IEEE relay qualification standard [31]; therefore, it is more indicative of lower frequency sensitivities than high frequency sensitivities. Between the remaining two RMF frequency ranges, the 20 to 40 Hz range is more consistent with the expected high frequency ground motions shown in Figure 3-5. The Phase 1 components were also slightly more sensitive to that frequency range input than the 24 to 48 Hz RMF motion.

FRMF Input Motions – The FRMF tests may be the most accurate at simulating the kinds of earthquake motions at the component mounting locations, but they were very time consuming to perform. Comparisons of the FRMF peak spectral accelerations that produced component chatter with the RMF peak spectral accelerations confirmed that previous narrow band clipping factors (e.g. [39], Appendix Q) are also generally applicable to high frequency motions.

Phase 1 Overall Conclusions

The Phase 1 study indicates that the high-frequency sensitivity of contact devices is generally device specific. Thus, the best means to identify such frequency sensitivity is to test the devices. Additional testing in Phase 2 may facilitate more general conclusions for some categories of potentially sensitive devices (e.g. miniature relays, potentiometers).

The use of the 20-40 Hz multi-frequency random input motion appears to be the best compromise for determining frequency sensitivity. The use of other input motions requires considerable effort and do not appear to provide any better resolution for determining high-frequency sensitivity.

Filtered multi-frequency narrow-band inputs resulted in peak spectral fragility values that were 2-3 times the spectral fragility values obtained using the wide-band multi-frequency inputs. Thus, it appears that the clipping factors used for low frequency fragility are valid for high-frequency fragility. (This is still under study)

3.4.3 Phase 2 Testing

Phase 2 testing will be performed to address the component types identified in Section 3.4.1.3. The complete test results will be compiled as appropriate to support utility high-frequency confirmation screening in Figure 1-1, Step 3f, as well as SPRA or SMA evaluations in Figure 1-1, Steps 6a and 6b.

3.4.3.1 Phase 2 Test Protocol

Based on the Phase 1 testing, the 20 to 40 Hz RMF response spectrum shape will be used to develop the test motions for the Phase 2 test protocol. These motions will be used to determine the fragility spectra for each component.

3.4.3.2 Expanded Sample

The test sample list for Phase 2 testing will be selected to address the range of component types identified in Section 3.4.1.3. Components will be selected to represent a distribution of manufacturers and specific model numbers. Components will also be selected to address a variety of contact mechanical motions (e.g., plunger- and clapper-type relays) and physical forms (e.g., socket and bolted mounting configurations). The number of components in any component type category may be adjusted depending on the expected degree of high-frequency sensitivity. In addition, the specific model numbers selected may be adjusted depending on the component availability. To the extent practical, the distribution of test samples will be selected to achieve the broadest possible conclusions.

Section 4: Seismic Hazard and Screening Report

The NRC 50.54(f) information request associated with NTTF Recommendation 2.1 seismic is delineated in [1]. Within 1.5 years of the March 12, 2012 date of the information request, each CEUS addressee is requested to submit information related to the seismic hazard and the screening portions of the program (see Boxes 1, 2, and 3a-3e of Figure 1-1). An example of the type of information that has been requested, which could form the table of contents for that report, is listed below.

- Introduction
  1. Responding to 50.54(f) letter
  2. Brief history of seismic licensing basis (summary of SSE and which codes, standards, and methods were used in the design of Seismic Category I SSCs)
  3. Brief description of method used to develop GMRS and outcome of screening comparisons
- Seismic Hazard Results: GMRS
  1. Regional and Local Geology
     a. Regional Geology
        i. 1-2 paragraphs describing tectonic setting and history
     b. Local Geology
        i. 1-2 paragraphs described any prominent geologic features, complexity of geologic features (folding and faulting)
  2. Probabilistic Seismic Hazard Analysis
     a. Probabilistic Seismic Hazard Analysis
        i. Summary of sources used (sub-set of CEUS Seismic Source Characterization sources or site-specific for WUS sites)
        ii. Ground Motion Prediction Equations used or developed
     b. Base Rock Seismic Hazard Curves (if hard rock site)
        i. Common fractiles and mean for spectral frequencies for which GMPEs are available

3. Site Response Evaluation (if not a hard rock site)
    a. Description of Subsurface Materials and Properties
        i. Soil/rock types, layering, and properties
    b. Development of Base Case Profiles and Nonlinear Material Properties
        i. Resources used and basis for base case profiles
            1. Base case shear wave velocity profiles
            2. Selected Shear Modulus and Damping curves
    c. Randomization of Profiles
        i. Randomization method and parameters
        ii. Constraints applied on layer thicknesses and velocities
        iii. Kappa values
    d. Input Spectra
        i. Fourier amplitude spectra and response spectra including input elevation
        ii. Any modifications to input spectra (kappa correction)
    e. Methodology
        i. Brief description of Random Vibration Theory (RVT) or time series approach
        ii. Parameters used in RVT or time histories used
    f. Amplification Functions
        i. Amplification functions
        ii. Amplification versus Input Amplitude including uncertainty bands for each of the spectral frequencies
    g. Control Point Seismic Hazard Curves
        i. Common fractiles and mean for spectral frequencies for which GMPEs are available
4. Ground Motion Response Spectrum
    a. Uniform Hazard Response Spectra
        i. 10-4 and 10-5 UHRS
    b. GMRS
        i. Table of 10-4 and 10-5 UHS, Design Factor values, and GMRS

- Safe Shutdown Earthquake Response Spectra
    1. Spectral Shape and Anchor Point PGA for 5% critical damping
        a. Brief description from FSAR
    2. Control Point Elevation(s)

a. Description from FSAR or assumptions used to determine control point elevation

- Special Screening Considerations
  1. GMRS and SSE Comparison
     a. Discussion of results
     b. High-frequency, Narrow Band Exceedance (if applicable)
  2. Evaluation of IPEEE Submittal
     a. see Section 3.3-1
  3. GMRS and IHS Comparison
     a. If applicable, discussion of results (narrow-band exceedance if applicable)
  4. Screening for Risk Evaluation (SPRA or SMA see Section 6.2)
     a. If applicable, discussion of results
- Interim Actions*
  1. Any interim actions taken or planned while risk evaluation is being performed
- Conclusions
  1. Summary of results
  2. Path forward based on Screening Evaluations

*The NRC has requested that each addressee provide information on "any actions taken or planned to address the higher seismic hazard relative to the design basis, as appropriate, prior to the risk evaluation." Examples of the types of information which could be included in this response are:

- Modifications or upgrades that the addressee decides to undertake prior to the seismic risk evaluation described in Section 6 of this report.
- Addressee intentions relative to conducting an SPRA or SMA.
- Description of the types of exceedances (low-frequency range, high-frequency range, narrow-frequency band, etc.) and the types of SSCs which may be affected by that exceedance (e.g., high-frequency exceedance could affect chatter sensitive devices which are going to be addressed by the EPRI testing program described in Section 3.4 of this report).

Section 5: Prioritization (Schedule)

The resolution for the 2.1 seismic information requests [1] consists of first generating the new seismic hazard information for all sites, followed by the screening assessments described in the previous sections. Those plants required to perform the seismic risk evaluation are then required to be prioritized in terms of their schedule for submittals (Diamond 5, Figure 1-1). This prioritization occurs after seismic hazard and screening submittals described in Section 4 of this report are submitted to the NRC. That report is scheduled to be completed by Fall 2013 for CEUS sites and Spring 2015 for the WUS sites.

The intent of the prioritization is to take into account:

- the amount of the seismic hazard exceedance
  - GMRS to SSE Ratios
  - GMRS to IHS
- the available resources (industry-wide and individual utility multi-unit fleets), and
- available plant mitigation strategies

Consideration should be given to plants that are already conducting SPRAs. Lessons learned from these early SPRA implementations will be valuable resources to the industry and NRC.

The exact criteria/methods to be used for this prioritization are being discussed between the NRC and the nuclear utility industry as part of ongoing discussions on the resolution of the 2.1 seismic program.

Section 6: Seismic Risk Evaluation

6.1 Background on SPRA and SMA

SPRA and SMA studies have been conducted for many of the U.S. NPPs over the last twenty years. Initially they were conducted to answer safety concerns in heavily populated areas. The next widespread application was for satisfaction of the USNRC request for information regarding severe accident vulnerabilities in Generic Letter 88-20, Supplement 4 [2]. The USNRC is currently encouraging the use of PRA for making risk-informed decisions and has developed a Risk-Informed Regulation Implementation Plan [32] and associated regulatory guides. The Licensees in turn are using PRAs for Changes to Licensing Basis, Changes to Technical Specifications, Graded Quality Assurance, Significance Determination Processes, etc. Seismic PRAs are also required for each new NPP one year prior to fuel load. SPRAs and SMAs are now also being recommended as paths to conduct the seismic risk evaluations within Tasks 6 and 7 of Figure 1-1.

6.1.1 SPRA Methods and Procedures

Current U.S. NPPs were designed to withstand a conservatively selected large earthquake ground motion (the SSE) with adequate margins introduced at different stages of design, analysis, qualification, and construction. However, it is understood that larger earthquake ground motions (although rare) could occur. The basic objective of the SPRA is to estimate the probability of occurrence of different levels of earthquake ground shaking that may affect the plant, and to assess the plant response to such ground motions. Following the historical PRA practice, the results of this plant seismic assessment are presented in terms of seismically induced CDF and large-early release frequency (LERF). SPRAs completed to date, have shown that the seismic contribution to the overall CDF and LERF at some NPPs could be significant and occasionally can even be dominant. Therefore, a quantitative assessment of the seismic risk (e.g., SPRA) can be an important component of the overall risk-informed decision making process.

The key elements of a SPRA can be identified as:

- **Seismic Hazard Analysis:** Used to assess the seismic hazard in terms of the frequency of exceedance for selected ground motion parameters during a specified time interval. The analysis involves the characterization of earthquake sources, evaluation of the regional earthquake history, and an estimation of the intensity of the earthquake-induced ground motion at the site (Figure 6-1).

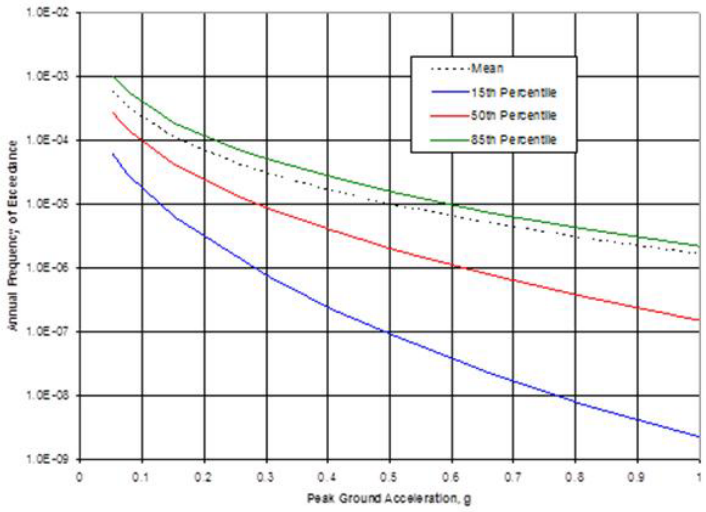

Figure 6-1
Example Seismic Hazard Curve

- **Seismic Fragility Analysis**: Estimates the conditional probability of SSC failures at a given value of a seismic motion parameter, such as PGA, peak spectral acceleration, floor spectral acceleration, etc. Seismic fragilities used in a seismic PRA are realistic and plant-specific based on actual current conditions of the SSCs in the plant, as confirmed through a detailed walkdown of the plant. The fragilities of all the systems that participate in the accident sequences are included (Figure 6-2).

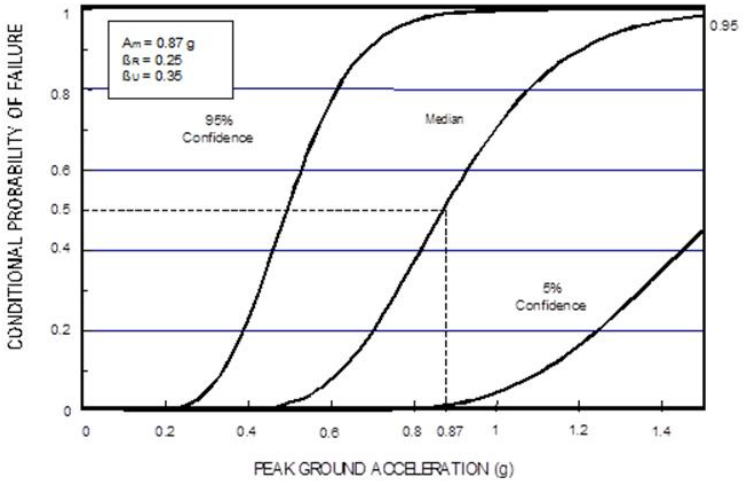

Figure 6-2
Example Seismic Fragility Curve

- **Systems/Accident Sequence Analysis**: Modeling of the various combinations of structural and equipment failures that could initiate and propagate a seismic core damage sequence.
- **Risk Quantification**: Calculates the frequencies of severe core damage and radioactive release to the environment by using the plant logic model and accident sequences for which the SSC fragilities are integrated with the seismic hazard. The analysis is usually carried out by adding some earthquake-related basic events to the PRA internal events model, as well as eliminating some parts of the internal events model that do not apply or that can be screened-out.

The overall SPRA process is characterized in Figure 6-3.

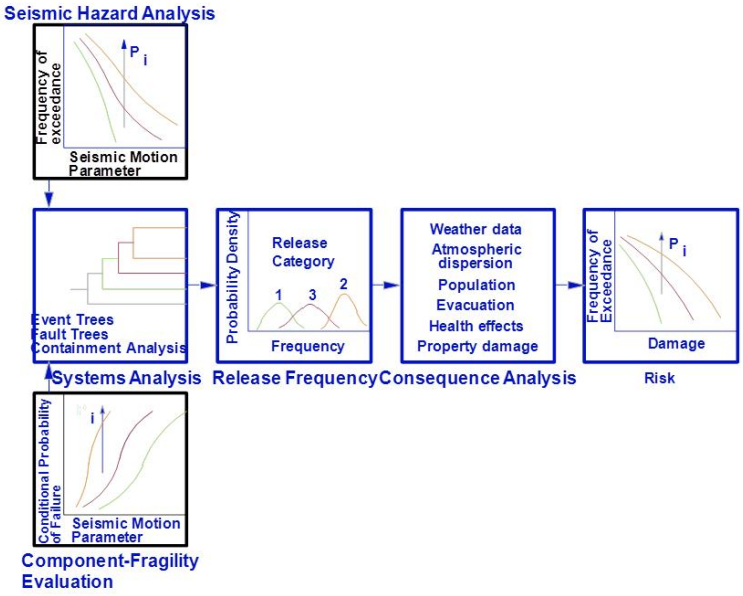

Figure 6-3
Overview of the SPRA Methodology

The detailed methods and criteria to develop the seismic fragility, seismic hazard, and seismic plant logic models are well beyond the scope of this guide. Fortunately, there are many technical references which document these methods. Table 6-1 is intended to provide a good list of references on these topics, while there are obviously many more in the literature.

Table 6-1
Partial List of SPRA Technical References

SPRA Topic	Recommended Document Title	Reference
SPRA	Seismic Probabilistic Risk Assessment Implementation Guide	EPRI 1002989 (Dec 2003) [33]
	Seismic Evaluation of Existing Nuclear Power Plants	Safety Report Series No. 28 [34]
	Probabilistic Safety Assessment for Seismic Events	IAEA Tecdoc-724 (Oct 1993) [35]
Seismic Fragility	Seismic Fragility Applications Guide Update	EPRI Report 1019200 (Dec 2009) [36]
	Seismic Fragility Application Guide	EPRI 1002988 (Dec 2002) [37]
	Methodology for Developing Seismic Fragilities	EPRI TR-103959 (June 1994) [38]
	A Methodology for Assessment of Nuclear Plant Seismic Margin	EPRI NP 6041 (Oct 1988) [39]
Seismic Hazard	PRA Procedures Guide: A Guide to the Performance of Probabilistic Risk Assessments for Nuclear Power Plants	NUREG/CR-2300 (1983) [54]
	Recommendations for Probabilistic Seismic Hazard Analysis: Guidance on Uncertainty and Use of Experts	NUREG/CR-6372 (1997) [13]
	Practical Implementation Guidelines for SSHAC Level 3 and 4 Hazard Studies	NUREG-2117 (2012) [23]
	Technical Basis for Revision of Regulatory Guidance on Design Ground Motions: Hazard- and Risk-Consistent Ground Motion Spectra Guidelines	NUREG/CR-6728 (Oct 2001) [24]

6.1.2 NRC SMA Methods and Procedures

The Nuclear Regulatory Commission (NRC) staff developed interim staff guidance (ISG) [100] on an acceptable method for performing a Seismic Margin Assessment (SMA) as referred to in the March 12, 2012 NRC letter [1]. This SMA method includes enhancements to the NRC SMA method originally described in NUREG/CR-4334 that the NRC deemed necessary to meet the objectives of the 50.54(f) letter. The NRC ISG approach for SMA is specifically intended be used to respond to the 50.54(f) letter. The level of effort to perform a SMA to meet this ISG is nearly equal to that required for a SPRA. The primary difference is that the SMA reports results in terms of HCLPF values, rather than risk metrics such as CDF or LERF.

A list of the high level features and enhancements to an SMA that are documented in the draft ISG are listed below. Some of these topics are similar to staff positions taken during the IPEEE program, and others are additional enhancements.

- The SMA should use a systems-analysis approach that begins by following the NRC SMA methodology, using event trees and fault trees, with enhancements; an EPRI SMA approach using success-path systems logic is not acceptable.
- The SMA should be a full-scope SMA, not a focused-scope or reduced-scope SMA (as described in NUREG-1407).
- The systems model should be enhanced over what was contained in either the original NRC SMA guidance (in NUREG/CR-4334 and NURE/CR-5076) or the NRC's IPEEE guidance (in NUREG-1407).
- The scope should include certain containment and containment systems, so as to enable analysis of the plant-level HCLPF for large early release.
- The "mission time" should extend to when the plant reaches a stable state.
- The use of the so-called "Min-Max" method must be justified and, if used, should follow certain guidance provided in [100]. The Convolution Method is stated to be the NRC's preferred method.
- When developing sequence-level and plant-level HCLPF capacities, the analysis should differentiate between those sequences that lead to core damage and those that lead to a large early release.
- Separately report HCLPF capacities for those sequences with non-seismic failures and human actions and HCLPF capacities for those sequences without them.
- A formal peer review of the SMA is required. The ISG peer review requirements are not consistent with the peer review requirements of RG 1.200 or the ASME/ANS PRA Standard.

The details for each of these features and enhancements are described in detail within the draft ISG [56].

Licensees may propose other methods for satisfying SMA requirements of the 50.54(f) NRC letter. The NRC staff will review such methods and determine their acceptability on a case-by-case basis.

6.2 Criteria for Selection of Risk Evaluation Method (SPRA vs. SMA)

As shown in Figure 1-1, plants that do not meet the screening criteria outlined in Section 3 of this report need to proceed to a seismic risk evaluation. Reference [1] describes two different approaches for performing the seismic risk evaluation, an SPRA, or an NRC SMA. The

NRC SMA is appropriate for sites where the re-evaluated seismic hazard is not considerably higher than the design basis seismic hazard or for sites that have a relatively low seismic hazard level. The SPRA could be used for any of the plants proceeding to the seismic risk evaluation phase.

The NRC criteria for requiring the use of the SPRA consists of the following:

- If the GMRS exceeds the response spectra between 1 and 10 Hz represented by the higher of the following two spectra, then an SPRA should be conducted:
  1. 1.3 times the SSE
  2. Low Hazard Threshold of 0.4g

Figure 6-4 shows an example of a GMRS exceeding the 1.3 SSE and the LHT spectra in the 1 to 10 Hz range.

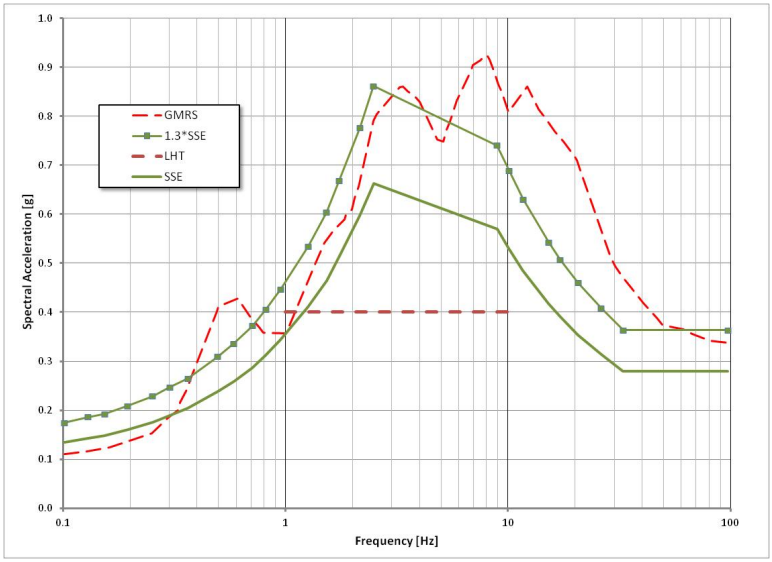

Figure 6-4
Example for Selection of SPRA vs. SMA

6.3 Key Elements of Seismic Structural and SSI Response

6.3.1 Structure Modeling

Many existing structural models (i.e., those used for design basis, USI-A-46 or IPEEE studies) could be used in structural dynamic analyses that are performed to support SPRAs or SMAs required as part of the response

to the 50.54(f) letter on 2.1, provided that their adequacy is demonstrated for this purpose. This requires a review of the existing models to be performed by an experienced structural engineer(s) (and a peer reviewer) to determine the adequacy of the models for dynamic analysis for application in risk assessments for 2.1 using the criteria provided below. If necessary, the existing structural models can be enhanced to meet the structural modeling criteria.

The existing structural models that have been used in dynamic analyses to develop seismic responses for the design, licensing and qualification of plant SSCs (e.g., lumped-mass stick models (LMSM)), were reasonably complex for their original intended purpose at the time they were developed. These models were used to capture the overall structural frequencies, mode shapes, and seismic responses. Typically, if a model complexity is increased, the contribution of the modes identified within the simpler model is decreased as modal mass is shifted to other modes, often resulting in lower spectral peaks for the significant modes of the structure. However, more recent experience has shown that, for some structures, additional complexity of the numerical model may lead to the identification of important higher modes that may be important for some systems and components.

Using the existing structural models, in either their current or enhanced state, will facilitate the completion of the SPRA/SMA effort with the desired accuracy required as part of the response to the 50.54(f) letter on 2.1.

The criteria against which structural engineer(s) and peer reviewer(s) should review the existing models are listed below.

1. The structural models should be capable of capturing the overall structural responses for both the horizontal and vertical components of ground motion.
2. If there is significant coupling between the horizontal and the vertical responses, one combined structural model should be used for analyzing all three directions of the earthquake. See ASCE 4-98 Section 3.1.1.1 “Models for Horizontal and Vertical Motions” [40].
3. Structural mass (total structural, major components, and appropriate portion of live load) should be lumped so that the total mass, as well as the center of gravity, is preserved. Rotational inertia should be included if it affects response in the frequency range of interest. See ASCE 4-98 Section 3.1.4.1 “Discretization of Mass” Part (b) 1 [40].
4. The number of nodal or dynamic degrees of freedom should be sufficient to represent significant structural modes. All modes up to structural natural frequencies of about 20 Hz in all directions should be included (vertical floor slab flexibility will generally not be considered because it is expected to have frequencies above 15 Hz, but this should be verified by the structural engineer). This will ensure

that the seismic responses and in-structure response spectra (ISRS) developed in the 1 to 10 Hz frequency range are reasonably accurate. See ASCE 4-98 Section 3.1.4.1 "Discretization of Mass" Part (b) 2 [40].

5. Torsional effects resulting from eccentricities between the center of mass and the center of rigidity should be included. The center of mass and the center of rigidity may not be coincident at all levels, and the torsional rigidity should be computed. See ASCE 4-98 Section 3.1.8.1.3 "Requirements for Lumped-mass Stick Models" Parts (b) and (c) [40]. Alternatively, a multiple LMSM may be used if the stiffness elements are located at the centers of rigidity of the respective groups of element and the individual models are properly interconnected.
6. The analyst should assess whether or not a "one-stick" model sufficiently represents the structure. For example, two-stick models could be more appropriate for the analysis of internal and external structures of the containment founded on a common mat.
7. The structural analyst should review whether in-plane floor flexibility (and subsequent amplified seismic response) has been captured appropriately for the purposes of developing accurate seismic response up to the 15 Hz frequency. Experience has shown that, for nuclear structures with floor diaphragms that have length to width ratios greater than about 1.5, the in-plane diaphragm flexibility may need to be included in the LMSM. The use of this 1.5 aspect ratio should be reviewed by the structural engineer since some structures are affected by the in-plane diaphragm flexibility by aspect ratios lower than the 1.5. As with all these recommendations, alternate approaches can be used when justified.

The use of existing models must also be justified in the submission to the NRC using the above criteria.

6.3.2 Seismic Response Scaling

Scaling of ISRS to account for higher ground motions levels is considered a technically sound approach and has been used in previous SPRAs and SMAs. Using scaling approaches, where appropriate, will reduce the effort involved in performing detailed soil-structure interaction (SSI) analyses for the new hazard/UHS, facilitating the completion of the SPRA or SMA effort for those plants that are screened-in.

Scaling of responses will be based on

- previously developed ISRS,
- shapes of the previous UHS/RLE,
- shapes of the new UHS/RLE, and
- structural natural frequencies, mode shapes, and participation factors.

Example guidance on scaling methods is provided in EPRI documents EPRI NP-6041-SL Rev. 1 [39] and EPRI 103959 [38].

Scaling can be used in developing ISRS for those cases where the new UHS or RLE shape is approximately similar to the spectral shape previously used to generate the ISRS. An example of two response spectra with similar shapes is shown below in Figure 6-5.

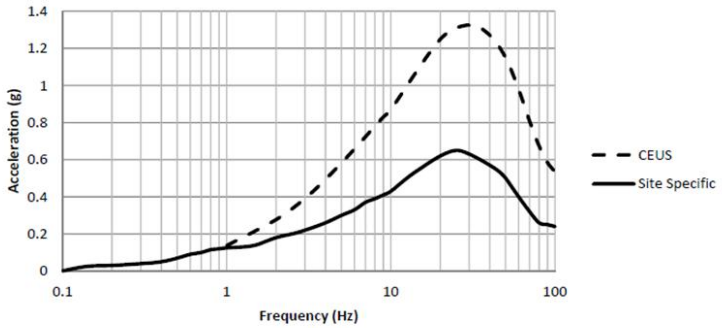

Figure 6-5
Example of Ground Response Spectra that are Similar

Scaling of rock or soil sites where the shape of the new hazard spectrum is not similar to the previous spectrum will require a rigorous justification that demonstrates the validity of the scaling approach. An example of spectra that are not similar is shown in Figure 6-5 below. The peak response of these two spectra is significantly shifted in frequency.

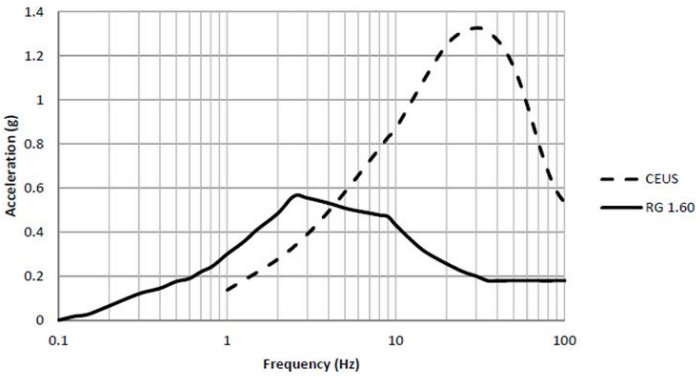

Figure 6-6
Example of Ground Response Spectra that are not Similar

Scaling of "non-similar" shapes would need to be technically justified on a case specific basis and would need to appropriately consider any non-linear effects to the structure or to the soil/rock profile resulting from the new response spectra shape and amplitude.

6.3.3 Fixed-Based Analysis of Structures for Sites Previously Defined as "Rock"

For nuclear safety-related structures founded on the commonly used definition of rock as defined in the design documentation for many operating plants, i.e., shear wave velocity (V_s) > about 5000 ft/sec, past experience has shown that the amplified response spectra in the 1 to 10 Hz are generally about the same from a fixed-based analysis of the structure as from a model that uses SSI analysis. Therefore, it is reasonable to use fixed-base dynamic analyses for rock-founded structures even when the rock shear wave velocities are not as high as 9200 ft/sec, which is the definition of hard rock for new reactors licensed by the NRC. The original definition of rock (V_s > about 5000 ft/sec) that was used by some plants in the past can still be used as the criterion for performing a fixed-base dynamic analysis to develop ISRS that are needed to perform fragility or HCLPF calculations.

The validity of the above criterion was reviewed using two examples of existing structures at a nuclear power plant [55]. The first example describes the analysis of a structure with a fundamental frequency of about 5 Hz, and the second example used a structure with a fundamental frequency of about 10 Hz in one horizontal direction. These examples considered fixed-base analysis and SSI analyses with different V_s values and are discussed in Appendix C. The results from this study show that there is a slight shift of frequency to the left, and some changes in spectral peak amplitudes occur when the fixed base is compared to an SSI analysis with V_s of about 3500 ft/second; however, the comparison of fixed-base analysis is much better with an SSI analysis using V_s of about 5000 ft/sec or higher.

Therefore, it is appropriate to model a rock-founded structure as fixed base if the best estimate of V_s is greater than about 5000 ft/sec. For structures founded on rock with V_s between 3500 ft/sec and 5000 ft/sec, peak-broadening or peak-shifting of the ISRS in fragility analyses can potentially alleviate the effect of a slight frequency/amplitude shift between the SSI and fixed-base analyses. The determination whether a fixed-base model can be used for rock sites with V_s values in this range should be made by an experienced structural engineer and justified in the submittal report to NRC. This assessment should also be peer reviewed by an experienced structural engineer as part of the peer review process.

6.4 Key Elements of Fragility/Capacity for the Resolution of NTTF Recommendation 2.1

6.4.1 Hybrid Approach for Fragilities

There are two well-known methods to calculate fragilities of SSCs for use in a seismic PRA model [36, 37, 38, 39]. These are: (a) the Conservative Deterministic Failure Margin (CDFM) approach [39] where the HCLPF capacity is calculated first and the median capacity with an assumed (experience-based) composite variability (typically in the 0.35 to 0.45 range) is then calculated from the HCLPF; and (b) the fragility or separation of variables approach [36, 37, 38] where the median capacity is calculated, and the randomness and uncertainty variabilities (logarithmic standard deviations) are then calculated in a detailed manner for various parameters.

The CDFM approach for developing fragilities is a simpler method that can be performed consistently by more analysts and is an acceptable approach for generating fragilities within an SPRA for the majority of components for which a less detailed assessment is necessary. Because only a handful of components are risk-significant enough to justify the additional effort required by the separation of variables method, the CDFM method can provide efficiencies in the overall effort. Therefore, use of the CDFM approach is useful and beneficial for calculating fragilities of SSCs for use in seismic PRAs conducted to address the 50.54(f) letter.

In the CDFM fragility approach (also referred to as the Hybrid Method), the 1% failure probability capacity $C_{1\%}$ is computed along with an estimate of the composite logarithmic standard deviation β_C and its subdivision into random variability β_R and uncertainty β_U, which, are used to estimate the corresponding fragility curve. As noted in [51], typically β_C lies within the range of 0.3 to 0.6. In fact, if all of the sources of variability discussed in [38] are appropriately considered, it is not possible to obtain an estimated β_C less than approximately 0.3.

The Hybrid Method is based on the observation that the annual probability of unacceptable performance (P_F) for any SSC is relatively insensitive to β_C. This annual probability (seismic risk) can be computed with adequate precision from the CDFM Capacity (C_{CDFM}) and an estimate of β_C. It is shown in [51] that the computed seismic risk at $\beta = 0.3$ is approximately 1.5 times that at $\beta = 0.4$, while at $\beta = 0.6$ the computed seismic risk is approximately 60% of that at $\beta = 0.4$.

Table 6-2 provides recommended values for β_C, β_R, β_U, and the ratio of the median capacity $C_{50\%}$ to the $C_{1\%}$ capacity computed by the CDFM Method. The recommended β_C values are based on Ref. [51] recommendations and on average are biased slightly conservative (i.e., slightly low β_C on average). Because random variability β_R is primarily due to ground motion variability, a constant β_R value of 0.24 is recommended

irrespective of the SSC being considered. The recommended β_U values are back-computed from the recommended β_C and β_R values. The Beta values for Table 6-2 apply to fragilities tied to ground motion parameters (e.g., PGA or Peak Spectral Acceleration at 5 Hz). Appendix D contains a sensitivity study on the computed probability of failure, P_F, to the logarithmic standard deviation used in the hybrid method. The results of the study in Appendix D demonstrates a lack of sensitivity of the computed seismic risk exists over the full practical range of seismic hazard curve slopes.

Table 6-2
Recommended β_C, β_R, β_U, and $C_{50\%}/C_{1\%}$ Values to Use in Hybrid Method for Various Types of SSCs

Type SSC	Composite β_C	Random β_R	Uncertainty β_U	$C_{50\%}/C_{1\%}$
Structures & Major Passive Mechanical Components Mounted on Ground or at Low Elevation Within Structures	0.35	0.24	0.26	2.26
Active Components Mounted at High Elevation in Structures	0.45	0.24	0.38	2.85
Other SSCs	0.40	0.24	0.32	2.54

Following the generation of the fragilities using the hybrid approach, the fragility parameters are then used in the systems model to convolve with the hazard. For those SSCs that are determined to be the dominant risk contributors or are risk-significant in the seismic accident sequences, estimates of median capacity ($C_{50\%}$) and variabilities (β_u and β_r) should be done using the fragility or separation of variables approach and then used in the integration.

6.4.2 High-Frequency Capacities

Section 3.4 discusses the treatment of high frequency responses of SSCs that may result from new seismic hazards and/or GMRS shapes developed for CEUS plants, particularly for the plants on rock sites. In general, the seismic testing described in Section 3.4 will produce results that can be divided into three categories. Some components will have high capacities when subjected to the high frequency motions while others will have moderate capacities, and still others may have low capacities.

The flow chart in Figure 6-7 provides a general process for performing the High Frequency Confirmation in Figure 1-1, Step 3f. Components that demonstrate a high capacity can be screened out from further evaluations.

Components that demonstrate a moderate or low capacity will require further evaluations. The recommended relay evaluation methods are derived from the evaluation methods used in the SQUG program [52].

For relays or contactors, some applications can be accepted by performing circuit analysis to show that contact chatter does not lead to inappropriate plant operation. Examples include relays that provide control room annunciations or contactors that turn on or off motors. In the case of the annunciation, following an earthquake, operators will reset control room annunciations and then evaluate those annunciations that do not properly reset. In the case of motors, contact chatter does not typically provide enough power to the motor to drive it to move. These are examples where relay or contactor chatter is considered acceptable.

Other relay chatter affects can be resolved by Operator Actions. Examples of this strategy include resetting lock-out or seal-in relays that lead to undesired plant conditions. Care should be exercised to avoid overloading the Operators by crediting too many Operator Actions.

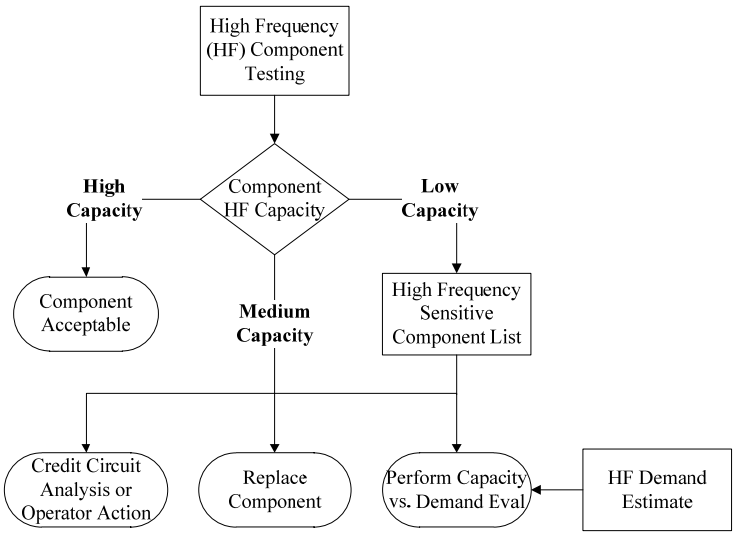

Figure 6-7
Potentially High-Frequency Sensitive Component Screening

High frequency sensitive components with moderate or low capacities can also be replaced with comparable components with higher seismic capacities.

In general, the above approach ensures the adequacy of potentially high frequency sensitive components and specific demand to capacity

comparisons or calculations of their fragilities for inclusion in the SPRA logic models will typically not be required.

If needed, a detailed capacity to demand evaluation can be done to show that the seismic capacity of the component exceeds the seismic demand at the component mounting location. The ability for utilities performing the High Frequency Confirmation under Figure 1-1 Step 3f to perform high frequency capacity to demand evaluations may be limited because of the difficulty in calculating a reliable and accurate high frequency demand at the component mounting locations. Research activities may be performed as part of the High Frequency Program in Section 3.4 leading to estimation techniques for high frequency in-structure and in-cabinet seismic demands adequate to perform component capacity to demand evaluations. If needed, plants performing SPRAs can calculate fragilities of potentially high frequency sensitive components using the best available estimates of seismic demand.

6.4.3 Capacity-based Selection of SSCs for Performing Fragility Analyses

Capacity-based criteria to determine the SSCs for which fragility analyses should be conducted have been developed to provide uniform guidance to analysts performing seismic PRAs (or margin analyses) and to ensure that proper focus is given to those SSCs that have the potential to be risk-significant. These criteria were developed as a result of a parametric/sensitivity study [42] that was undertaken for that purpose. These criteria establish which SSCs will require explicit calculation of fragility parameters for inclusion in the plant logic models. SSCs with capacities above the screening level calculated using the criteria are not expected to have significant impact on the result of the seismic PRA analyses, the ranking of accident sequences, or the calculated sequence- or plant-level seismic CDF or LERF values.

It is noted that a standard practice for seismic PRA practitioners has been to use insights from logic models to determine the need for fragility calculations and to prioritize SSCs. A preliminary SPRA plant logic model is developed even before the fragility calculation effort begins. Screening or ranking of SSCs from this preliminary SPRA logic model can be done by performing parametric sensitivity analyses with assumed initial fragilities and ranges of fragility values. Those SSCs that do not contribute significantly to the SCDF of an accident sequence may not need detailed fragility calculations. These SSCs may be retained in the model with a screening level capacity value which is described below.

Certain SSCs are inherently seismically rugged and consequently have a very low probability of failing as a result of a seismic event, as shown in Figure 6-7. Consistent with long-standing practice in seismic PRAs, seismic failure of such SSCs need not be included in the PRA logic models. Exclusion of such SSCs from the logic models does not affect the seismic

CDF or the insights derived from the seismic PRA. Guidance in industry documents [39, 41] is available for identifying seismically rugged SSCs.

Other SSCs may be less rugged but would still have sufficient capacity such that their failures would be unlikely to contribute significantly to the SCDF in a seismic PRA. Screening criteria discussed below are developed for these SSCs. Detailed fragility calculations are not warranted for SSCs that meet these criteria. Figure 6-7 illustrates the use of screening level, which is applicable to the SSCs in the middle box.

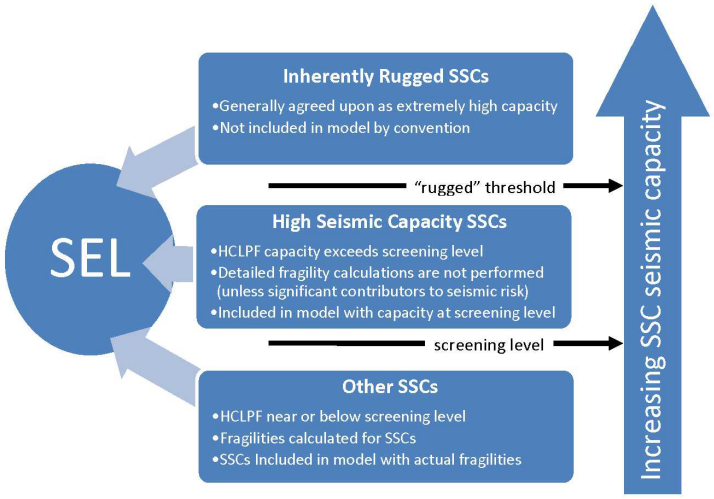

Figure 6-8
Capacity-based Criteria for Fragility Analyses

Based on the results of the sensitivity study conducted to develop this guidance [42], the screening HCLPF value of SSCs for a site should be calculated by convolving the fragility of a single element with the site-specific hazard curve such that the seismic CDF is at most about 5E-7 per year. This can be done with trial and error runs using a quantification code or with a spreadsheet with an assumed composite variability (e.g., β_c= 0.4). Because each site will have a different hazard curve, the screening HCLPF value for each seismic PRA needs to be separately derived. An alternative criterion, equivalent to the above CDF-based HCLPF, is to screen SSCs that have a HCLPF capacity above about 2.5 x GMRS. The results of the sensitivity study do not indicate that the screening criteria would be different for soil and rock sites.

The results of the sensitivity analyses performed indicate that the recommended screening HCLPF capacity derived from a CDF of 5E-7 is conservative for some hazard curves; a more liberal criterion may be

appropriate for some sites and can be developed from an initial quantification of the logic model. Even though certain SSCs can be screened-out from having to perform detailed fragility calculations using the above criteria, their failure should be retained in the seismic PRA logic model with their capacity equal to the screening level or at a higher capacity level, if calculated, to allow for a more efficient ranking of accident sequences. Retention of such failures will ensure that future changes or sensitivities that could increase their importance are not overlooked and also addresses the problem of a potential cumulative effect of screened out components.

The results of the SPRA should be reviewed to determine whether or not an SSC modeled at the screening level could be identified as a significant contributor to CDF or to LERF sequences. If such an SSC is identified, then detailed fragility calculations should be performed for that SSC using the separation of variables method, and the quantification analysis should be rerun with the new fragility values.

If a component modeled at the screening level is risk significant, then the screening level has been set too low. One approach to assess whether the screening level was set at a sufficiently high level is to use a screening level set to three times the screening level originally used in the logic model for that component, quantify the seismic CDF/LERF and ensure that the CDF/LERF estimates are not reduced by more than 20%. It is likely that there will be several SSCs initially modeled at the screening capacity level, and the above approach, if performed for one component at a time, may be cumbersome. The procedure can also be done for multiple SSCs modeled at the screening capacity level simultaneously (i.e., set the screening level for all such SSCs to three times the original level and ensure that the CDF/LERF estimates do not reduce by more than 20%). If either of the estimates change by more than 20%, it may take some effort to pinpoint which component(s) modeled at the screening level is/are risk-significant. The sensitivity analyses in [42] showed that if the ratio of the screening level CDF (i.e., 5E-7 per year) to the plant's seismic CDF is not much greater than about 3% to 4%, the cumulative impact of SSCs modeled at the screening level is not expected to be significant (i.e., none of the screening level SSCs is likely to be risk-significant). This 3% to 4% ratio can be used as a guide by the PRA analyst to determine if the initial screening criterion for SSCs was appropriate or the screening capacity level needs to be adjusted (up or down).

To implement the capacity-based screening criteria, engineers can review previous calculations and reports (e.g., design basis, IPEEE, USI A-46 analyses, shake-table tests, etc.) to determine and judge if the seismic capacity of a component or structure for the new seismic hazard is such that no further calculation of fragility parameters is warranted.

It is expected that the use of the above screening methods will reduce the scope of the fragility or margin calculations required in the SPRA or SMA,

and still meet the objective of identifying and ranking safety-significant SSCs. It is noted that, while the use of the above criteria is optional, engineers should not select a low screening HCLPF level (such as 0.3g) that was used by some plants during the IPEEEs. The above criteria are expected to result in sufficiently high screening levels to minimize the surrogate SCDF contribution (modeled at the screening level). Once the screening level is selected, the list of SSCs can be ordered so that the ones with the highest SCDF impact are calculated first.

6.5 Key Elements of SPRA/SMA Scope and Plant Logic Modeling

6.5.1 Evaluation of LERF

Enclosure 1 to the 50.54(f) letter recommends that a SPRA performed as the evaluation for plants that do not screen based on the change in the assessed seismic hazard address the frequency of large early release (LERF) in addition to CDF. The LERF risk measure for post-core damage accident sequences is evaluated in detail in the internal events PRA that is the foundation for the SPRA. The ASME/ANS Standard [12] appropriately directs use of the internal events LERF analysis as the starting point for the SPRA LERF assessment:

> *The approach to any external-events PRA typically uses as its starting point the internal-events PRA model…both the part of the internal-events model dealing with CDF and the part dealing with LERF are used as starting points.*

Seismic-induced core damage events can contribute to LERF scenarios in three ways:

1. Seismic-induced failures of systems important to containment performance and to preventing a large early release may contribute to LERF.

2. Like other core-damage accidents that can lead to a large early release, some portion of those initiated by a seismic event can contribute to LERF, even if the scenario does not directly include a seismic challenge to containment.

3. Some accident scenarios that would not constitute large early releases could effectively be considered to be such due to the potential that the seismic event could delay or slow evacuation.

The appropriate treatment for these LERF contributors is addressed in the subsequent subsections.

6.5.1.1 Seismic-induced Failures of Systems Important to LERF

The starting point for the seismic LERF evaluation is the internal events LERF model. In the internal events portion of the ASME/ANS PRA Standard, Table 2-2.8-9 identifies the LERF contributors considered depending on the containment design. Table 6-3 lists the potential contributors from the Standard and summarizes the recommended approach to addressing the LERF contributor in the SPRA.

Structures, systems and components beyond those relevant to preventing core damage must be considered for the LERF analysis. This requires that the seismic equipment list (SEL) be expanded to include the following (based on applicability to the particular containment design):

i. The containment structure, including the containment pressure boundary and pressure suppression function, as applicable.

ii. Equipment that plays a role in containment isolation failure modes from the internal event LERF model, including containment isolation valves, containment airlocks relying on inflatable seals to maintain containment integrity, and required support systems including backup air supplies.

iii. Hydrogen igniters and required support systems (PWRs with ice condenser containments and BWRs with Mark III containments).

iv. The isolation condenser (IC) for BWRs that have them, including IC tube failure modes.

v. Containment vacuum breakers, as applicable.

Table 6-3
Consideration of LERF Contributors in SPRA

LERF Contributor	Treatment in SPRA
Containment isolation failure	• Include containment (pressure boundary) in SEL • Include containment isolation failure modes in SEL • Consider containment isolation function in relay chatter evaluation
Containment bypass	
(a) Interfacing-systems LOCA	• Consider possibility due to relay chatter
(b) Steam generator tube rupture (SGTR)	• No significant seismic-induced impact
(c) Induced SGTR	• No significant seismic-induced impact
Energetic containment failure (a) High-pressure melt	• No significant seismic-induced impact

ejection (b) Hydrogen combustion (c) Core-debris impingement	• Include hydrogen igniters and support systems in SEL • No significant seismic-induced impact
Steam explosion	• No significant seismic-induced impact
Shell melt-through	• No significant seismic-induced impact
Bypass of pressure suppression	• Consider as part of containment seismic fragility evaluation
Venting of reactor pressure vessel or containment	• No significant seismic-induced impact
Rupture of isolation condenser tube(s)	• Include isolation condenser tubes in SEL
Vacuum breaker failure	• Include vacuum breakers in SEL
Hydrodynamic loads under severe accident conditions	• No significant seismic-induced impact
Containment flooding	• No significant seismic-induced impact
In-vessel recovery	• No significant seismic-induced impact
Containment failure induced by failure of reactor to scram	• No significant seismic-induced impact

The relay chatter evaluation should be expanded to address the following functions related to seismic-induced LERF contributors, as applicable:

- Containment isolation
- Closure of ISLOCA isolation valves

Seismic-induced failure modes for contributors noted in Table 6-3 should be evaluated and either screened due to high seismic capacity or evaluated by adding appropriate seismic failure events to the LERF logic model.

Other LERF contributors can be addressed in the same manner as addressed in the internal events LERF model.

6.5.1.2 Integration of LERF Model with Seismic Core Damage Model

The seismic-induced core damage scenarios from the Level 1 SPRA should be extended to address the potential for large early release. It is important to preserve the potential dependencies between the Level 1 core damage sequences and the containment response event tree. A typical systems-analysis approach is to add seismic-related basic events (or sometimes entire new "branches") to the internal-events event- and fault-tree models that are adapted from the internal-events-PRA Level 1 and LERF analysis.

In this manner, both the random (non-seismic-induced) failures and the seismic-induced failures described in Section 6.5.1.1 can be accounted for.

6.5.1.3 Definition of LERF Scenario for Seismic Accidents

An accident scenario constitutes a potential large early release if it leads to discharge of a large fraction of the core's inventory of airborne fission products (i.e., they are not effectively scrubbed) and the release occurs before there has been time for protection of the public in the vicinity of the plant (by evacuation or sheltering). Thus, "early" is a relative time.

In the event of core damage due to a seismic event, it is possible that implementation of the emergency plan could be delayed or impeded. For example, evacuation routes might be disrupted due to seismic damage. Thus, a scenario that would not lead to an early release for an internal initiating event could be effectively early if the sequence resulted from an earthquake.

This consideration is noted in Section 5-1.3 of the ASME/ANS PRA Standard, although there are no corresponding high-level or supporting requirements in the Standard.

Addressing this consideration would require a site-specific examination of the potential for seismic failures that could have an impact on the emergency plan (e.g., on systems for notifying the public, on bridges and other structures that could be in evacuation routes, etc.). Such an examination is beyond the scope of the evaluation for NTTF Recommendation 2.1, which is focused on the seismic capacities of plant systems and structures relative to updated estimates of seismic hazard.

6.6 Comparison to ASME/ANS SPRA Standard and RG1.200

6.6.1 Background

The ASME/ANS PRA Standard [12] presents requirements that are intended to specify the capability of a PRA in terms of its scope, level of detail and plant specificity relative to the nature of the intended application for the risk assessment. In any PRA performed for a risk-informed application, the intent is that the analyses meet at least the minimum requirements that could be relevant for the application, at the capability category corresponding to the nature of the application. The application in this case is to gain an updated understanding of the risk of seismic events at NPPs in light of new information about seismic hazard. This includes developing a new or changed understanding of risk outliers due to seismic events. For any future applications, the SPRA conducted for the 50.54(f) evaluation can be used if it is shown to meet the following Standard requirements: it is updated to reflect the then plant configurations and operations; it meets the specific update requirements

of the Standard (e.g., SHA-H1); and it is shown to be adequate for the intended application.

It is expected that these seismic evaluations will be performed with sufficient detail and plant specificity to meet the requirements for Capability Category II, wherever feasible. To meet Capability Category II, the PRA must generally account for plant-specific configuration and design, and reflect plant-specific data where doing so could affect the important risk contributors.

For this application, which is aimed at developing an improved understanding of the impact of new seismic hazard estimates, screening approaches will be used to limit the scope of detailed analyses for some specific elements of the seismic PRA. Where more detailed analyses are essential to achieve an adequate level of understanding (e.g., with respect to "realism"), these analyses will be performed or alternative measures will be taken (such as making plant changes to address the impacts).

The requirements in the Standard are organized according to the three major elements of a seismic PRA:

- Seismic hazard analysis (element SHA in the Standard),
- Seismic fragility analysis (element SFR), and
- Seismic plant-response modeling (element SPR).

The supporting requirements for each of these three elements are discussed in the next section in the context of the guidance provided in this document. For any specific instances in which the simplifications employed to facilitate seismic evaluations for use in responding to the 50.54(f) letter would not produce analyses that meet Capability Category II, a justification is provided.

6.6.2 Comparison of 2.1 Seismic Approach to the SPRA Standard

For this application, the requirements corresponding to Capability Category II of the ASME/ANS PRA Standard will generally be applied in the performance of elements of an SPRA. As noted above, exceptions to the Standard requirements may be taken in limited cases. The intent of the Standard will be met. Each supporting requirement has been reviewed against the technical approach recommended in this document to assess the capability category that applies.

The ASME/ANS PRA Standard has recently been revised, and a new version has been approved by the Joint Committee on Nuclear Risk Management, which is responsible for its development. The new version has not yet been published. To be of maximum benefit to users of either version of the PRA Standard, the comparison has been made to the supporting requirements in both versions. These are as follows:

- ASME/ANS RA-Sa–2009, the currently approved version of the ASME/ANS PRA Standard [12] (commonly referred to as "Addendum A"); and
- Addenda B to ASME/ANS RA-Sa-2009, the version of the Standard that is currently undergoing balloting [44] (commonly referred to as "Addendum B").

The NRC has endorsed Addendum A of the PRA Standard for use in regulatory applications via Regulatory Guide 1.200, Revision 2 [43].

In some cases, Regulatory Guide 1.200 provides further clarification or specification beyond the details in the PRA Standard. In the case of the requirements for seismic PRA elements SFR and SPR, however, there are no such clarifications in Regulatory Guide 1.200.

Comments on the guidance in this document relative to the supporting requirements in the two versions of the PRA Standard are provided in Tables 6-4 through 6-6.

The PRA Standard includes both high-level requirements and more detailed supporting requirements. In Addendum B, the high-level requirements reflect relatively minor wording changes from Addendum A. Therefore, Tables 6-4 through 6-6 include the high-level requirements as they exist in Addendum B , with changes from Addendum A indicated.

This document does not provide explicit guidance for every element of a seismic PRA, relying instead on reference to other resources. It therefore does not address the technical details corresponding to every supporting requirement. In those cases for which this document does not provide guidance, it is expected that the seismic evaluation will incorporate analyses commensurate with Capability Category II.

Table 6-4
Comparison of SPID Guidance to ASME/ANS PRA Standard Supporting Requirements: Element SHA

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SHA-A	The frequency of ~~earthquake~~ seismic ground motions at the site shall be based on a site-specific probabilistic seismic hazard analysis (existing or new) that reflects the composite distribution of the informed technical community. The level of analysis shall be determined based on the intended application and on site-specific complexity.		
SHA-A1	In performing the probabilistic seismic hazard analysis (PSHA), BASE it on, and MAKE it consist of, the collection and evaluation of available information and data, evaluation of the uncertainties in each element of the PSHA, and a defined process and documentation to make the PSHA traceable. *CC II–III*	In performing the probabilistic seismic hazard analysis (PSHA), BASE it on, and MAKE it consist of, the collection and evaluation of available information and data, evaluation of the uncertainties in each element of the PSHA, and a defined process and documentation to make the PSHA traceable. *CC II–III*	The PSHA is based on an evaluation of available information on historical seismicity, paleoseismic data on large-magnitude recurrence rates, and ground motion models, and is based on available information on each site's geologic profile. This information is modeled, including evaluation of uncertainties, using a defined evaluation process. Documentation will be included in each site's hazard report. Licensees in the CEUS will be using the CEUS-SSC, which is documented in NUREG-2115. PSHAs conducted under the SPID will meet this requirement directly.
SHA-A2	As the parameter to characterize both hazard and fragilities, USE the spectral accelerations, or the average spectral acceleration over a selected band of frequencies, or peak ground acceleration. *CC I-II*	As the parameter to characterize both hazard and fragilities, USE the spectral accelerations, or the average spectral acceleration over a selected band of frequencies, or peak ground acceleration. *CC I-II-III*	Spectral accelerations will be used to characterize seismic hazard, and are transmitted for use in fragility evaluations. PSHAs conducted under the SPID will meet this requirement directly.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SHA-A3	In the selection of frequencies to determine spectral accelerations or average spectral acceleration, CAPTURE the frequencies of those structures, systems, or components, or a combination thereof that are significant in the PRA results and insights. *CC I-II-III*	In the selection of frequencies to determine spectral accelerations or average spectral acceleration, INCLUDE the frequencies of SSCs that are significant in the PRA results and insights. *CC I-II-III*	Spectral accelerations will be reported in hazard calculations that capture spectral frequencies from 0.1 Hz to 100 Hz. This is sufficient to capture the frequencies of structures, systems, and components. PSHAs conducted under the SPID will meet this requirement directly.
SHA-A4	In developing the probabilistic seismic hazard analysis results, whether they are characterized by spectral accelerations, peak ground accelerations, or both, EXTEND them to large-enough values (consistent with the physical data and interpretations) so that the truncation does not produce unstable final numerical results, such as core damage frequency, and the delineation and ranking of seismic-initiated sequences are not affected. *CC I-II-III*	In developing the probabilistic seismic hazard analysis results, whether they are characterized by spectral accelerations, peak ground accelerations, or both, EXTEND them to large-enough values (consistent with the physical data and interpretations) so that the truncation does not produce unstable final numerical results, such as core damage frequency, and the delineation and ranking of seismic-initiated sequences are not affected. *CC I-II-III*	The PSHA results will be extended to spectral accelerations corresponding to annual frequencies of exceedence of 1E-8 or less. This should be sufficient to produce stable core damage frequency calculations and to identify and rank failure sequences. PSHAs conducted under the SPID will meet this requirement directly.
SHA-A5	SPECIFY a lower-bound magnitude (or probabilistically defined characterization of magnitudes based on a damage parameter) for use in the hazard analysis, such that earthquakes of magnitude less than this value are not expected to cause significant damage to the engineered structures or equipment. *CC I-II-III*	SPECIFY a lower-bound magnitude (or probabilistically defined characterization of magnitudes based on a damage parameter) for use in the hazard analysis, such that earthquakes of magnitude less than this value are not expected to cause significant damage to the engineered structures or equipment. *CC I-II-III*	A lower-bound magnitude of 5.0 will be used in the PSHA and is sufficient to capture earthquakes that will cause damaging ground motions. PSHAs conducted under the SPID will meet this requirement directly.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SHA-B	To provide inputs to the probabilistic seismic hazard analysis, ~~a~~ comprehensive up-to-date <u>data shall be compiled that include</u> ~~database, including~~ geological, seismological, and geophysical data; local site topography; and surficial geologic and geotechnical site properties~~, shall be compiled~~. A catalog of historical, instrumental, and paleoseismicity information shall also be compiled.		
SHA-B1	In performing the probabilistic seismic hazard analysis (PSHA), BASE it on available or developed geological, seismological, geophysical, and geotechnical databases that reflect the current state of the knowledge and that are used by experts/analysts to develop interpretations and inputs to the PSHA. *CC I-II*	In performing the probabilistic seismic hazard analysis USE it on available or developed geological, seismological, geophysical, and geotechnical data that reflect the current state of the knowledge and that are used by experts/analysts to develop interpretations and inputs to the PSHA. *CC I-II*	Inputs to the PSHA will be based on SSHAC Level 3 studies (including the CEUS Seismic Source Characterization model for sites in the CEUS), which use available geological maps (including tectonic province, crustal thickness, and volcanic maps) and geophysical maps (including gravity, heat flow, and crustal velocity maps). Site response studies will use geotechnical databases (including topographic maps, site cross-sections, borehole data, and seismic velocity measurements). Seismological data used for the SSHAC Level 3 studies, including the CEUS Seismic Source Characterization project, includes an updated historical earthquake catalog that represents a compilation of available earthquake catalogs for the region. PSHAs conducted under the SPID will meet this requirement directly.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SHA-B2	ENSURE that the database and information used are adequate to characterize all credible seismic sources that may contribute to the frequency of occurrence of vibratory ground motion at the site, considering regional attenuation of ground motions and local site effects. If the existing probabilistic seismic hazard analysis (PSHA) studies are to be used in the seismic PRA, ENSURE that any new data or interpretations that could affect the PSHA are adequately incorporated in the existing databases and analysis. *CC I-II*	ENSURE that the database and information used are adequate to characterize all credible seismic sources that may contribute significantly to the frequency of occurrence of vibratory ground motion at the site, considering regional attenuation of ground motions and local site effects. If the existing probabilistic seismic hazard analysis (PSHA) studies are to be used in the seismic PRA, ENSURE that any new data or interpretations that could affect the PSHA are adequately incorporated in the existing data and analysis. *CC I-II*	Under the SSHAC Level 3 process, databases and information are reviewed to ensure that all credible seismic sources are evaluated. Databases and information on ground motion equations are also reviewed to ensure that all credible ground motion models are evaluated. Available site data are reviewed as inputs to local site response calculations to ensure that all data and interpretations are incorporated into site response studies. Existing PSHA studies will not be used. PSHAs conducted under the SPID will meet this requirement through the use of published models used as inputs.
SHA-B3	As a part of the database used, COMPILE a catalog of historically reported, geologically identified, and instrumentally recorded earthquakes. USE reference [5-30] requirements or equivalent. *CC II-III*	INCLUDE an appropriate existing catalog of historically reported earthquakes, earthquakes reported through geological investigations, and instrumentally recorded earthquakes. USE reference [5-30] requirements or equivalent. *CC I-II*	Under the SSHAC Level 3 process, a catalog of historically reported, geologically identified, and instrumentally recorded earthquakes is compiled. PSHAs conducted under the SPID will meet this requirement.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SHA-C	To account for the frequency of occurrence of ~~earthquakes~~ seismic ground motions in the site region, ~~the seismic sources with the hazard model of~~ the probabilistic seismic hazard analysis shall examine all credible sources of potentially damaging earthquakes. ~~For the purpose of characterizing the occurrence rates for those seismic sources, all earthquakes greater than magnitude 3 shall be considered. Both the aleatory and epistemic uncertainties shall be addressed in characterizing the seismic sources.~~		
SHA-C1	In the probabilistic seismic hazard analysis, EXAMINE all potential sources of earthquakes that affect the probabilistic hazard at the site. BASE the identification and characterization of seismic sources on regional and site geological and geophysical data, historical and instrumental seismicity data, the regional stress field, and geological evidence of prehistoric earthquakes. *CC I-II-III*	In the probabilistic seismic hazard analysis, EVALUATE all potential sources of earthquakes that have the potential to contribute significantly to the probabilistic hazard at the site. MAKE the identification and characterization of seismic sources taking into account previous compilations of seismic sources, on regional and site geological and geophysical data, historical and instrumental seismicity data, and geological evidence of prehistoric earthquakes. *CC I-II-III*	Under the SSHAC Level 3 process, all potential sources of earthquakes that might affect a PSHA are examined. The identification and characterization of those sources is based on geological and geophysical data, historical and instrumental earthquakes, crustal stress, and tectonic interpretations. Available evidence of prehistoric earthquakes in the CEUS is evaluated in defining earthquake sources and their characteristics. PSHAs conducted under the SPID will meet this requirement.
SHA-C2	ENSURE that any expert elicitation process used to characterize the seismic sources is compatible with the level of analysis discussed in Requirement HA-A, and FOLLOW a structured approach. *CC I-II-III*	ENSURE that if an expert elicitation process is used to characterize the seismic sources then it needs to be compatible with the level of analysis discussed in Requirement HLR-SHA-A, and USE a structured approach. *CC I-II-III*	A SSHAC Level 3 approach for seismic sources is specified in the 50.54(f) letter to ensure that experts familiar with the regions of the study are contacted and elicited for information regarding potential seismic sources. Under the SSHAC Level 3 process, a structured approach is followed, including formal workshops and review of data and documentation. PSHAs conducted under the SPID will meet this requirement.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SHA-C3	The seismic sources are characterized by source location and geometry, maximum earthquake magnitude, and earthquake recurrence. INCLUDE the aleatory and epistemic uncertainties explicitly in these characterizations. *CC II-III*	The seismic sources are characterized by alternative source representations and source geometry, maximum earthquake magnitude, and earthquake recurrence. INCLUDE the aleatory and epistemic uncertainties explicitly in these characterizations, where significant. *CC II-III*	Under the SSHAC Level 3 process, the aleatory and epistemic uncertainties in seismic sources are characterized for source location and geometry, magnitude, and activity rate. For each source, uncertainty in geometry and location is represented with alternative interpretations, uncertainties in recurrence rates are represented with alternative rates, and uncertainties in maximum magnitude are represented with distributions of values. All of these interpretations ensure that the aleatory and epistemic uncertainties are included explicitly in the characterizations. PSHAs conducted under the SPID will meet this requirement.
SHA-C4	If an existing probabilistic seismic hazard analysis study is used, SHOW that any seismic sources that were previously unknown or uncharacterized are not significant, or INCLUDE them in a revision of the hazard estimates. *CC I-II-III*	If an existing seismic source model is used, DEMONSTRATE that any new seismic sources that have been identified or were uncharacterized when the existing models were developed are not significant, or INCLUDE them in the update of the hazard estimates. *CC I-II-III*	Existing PSHA studies are not used in characterizing seismic hazard. This requirement is not applicable to PSHAs conducted under the SPID.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SHA-D	The probabilistic seismic hazard analysis shall examine ~~credible~~ mechanisms influencing ~~estimates of~~ vibratory ground motion that can occur at a site given the occurrence of an earthquake of a certain type (e.g., strike slip, normal, reverse),magnitude, and at a certain location. ~~Both the aleatory and epistemic u~~ Uncertainties shall be addressed in characterizing the ground motion propagation.		

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SHA-D1	ACCOUNT in the probabilistic seismic hazard analysis for (a) credible mechanisms governing estimates of vibratory ground motion that can occur at a site (b) regional and site-specific geological, geophysical, and geotechnical data and historical and instrumental seismicity data (including strong motion data) (c) current attenuation models in the ground motion estimates *CC I-II-III*	In the vibratory ground motion analysis, INCLUDE (a) credible mechanisms governing estimates of vibratory ground motion that can occur at a site (b) historical and instrumental seismicity data (including strong motion data) (c) current attenuation models for the ground motion estimates *CC I-II-III*	(a) For the CEUS, the Updated Ground Motion Model project results to be used in the PSHA do not account explicitly for slip mechanism, because seismic sources in the CEUS do not distinguish between different slip mechanisms. Therefore, explicit modeling of slip mechanisms is not applicable to CEUS sites. For the WUS, ground motion prediction equations do take slip mechanisms into account and will be used for the PSHAs. (b) Under the SSHAC Level 3 process, empirical data for the region of interest is used to evaluate published ground motion models and derive modifications and weights for those models. The effect of local site conditions will explicitly be taken into account by modeling site amplification using a site-specific profile. PSHAs conducted under the SPID will meet this requirement through the use of published models used as inputs. (c) Ground motion models published for the WUS and CEUS, as appropriate, will be reviewed and evaluated to develop appropriate ground motion prediction models for the PSHAs. PSHAs conducted under the SPID will meet this requirement.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SHA-D2	ENSURE that any expert elicitation process used to characterize the ground motion is compatible with the level of analysis discussed in Requirement SHA-A, and FOLLOW a structured approach. *CC I-II-III*	ENSURE that any expert elicitation process used to characterize the ground motion or any other elements of the ground motion analysis is compatible with the level of analysis discussed in Requirement HLR-SHA-A, and USE a structured approach. *CC I-II-III*	For both the CEUS and WUS, the SSHAC approach is being used to characterize the ground motion. For the CEUS, the Updated Ground Motion Model project followed a SSHAC Level 2 approach for evaluating ground motion equations, which was appropriate for updating a prior ground motion model. For both CEUS and WUS, experts familiar with ground motions, for the respective regions, were or will be contacted and elicited to ensure that information regarding ground motion data and equations is properly characterized. A structured approach has or will be followed, including a formal questionnaire and documentation of responses to the questionnaire for each expert who has or will participate. PSHAs conducted under the SPID will meet this requirement.
SHA-D3	ADDRESS both the aleatory and epistemic uncertainties in the ground motion characterization in accordance with the level of analysis identified for Requirement SHA-A. *CC II-III*	INCLUDE both the aleatory and epistemic uncertainties separately in the ground motion characterization in accordance with the level of analysis identified for Requirement HLR-SHA-A. *CC II-III*	Both aleatory and epistemic uncertainties in ground motion characterization have been or will be addressed to develop ground motion prediction equations. Multiple equations were or will be derived, each with multiple aleatory uncertainties, thereby capturing epistemic uncertainty in ground motions.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SHA-D4	If an existing probabilistic seismic hazard analysis study is used, SHOW that any ground motion models or new information that were previously unused or unknown are not significant, or INCLUDE them in a revision of the hazard estimates. *CC I-II-III*	If existing ground motion models are used, DEMONSTRATE that new information not previously used or unknown would not significantly affect the PSHA results, or INCLUDE it in the update of the hazard estimates. *CC I-II-III*	Existing PSHA studies will not be used to characterize seismic hazard at any site. This requirement is not applicable to PSHAs conducted under the SPID.
HLR-SHA-E	The probabilistic seismic hazard analysis ~~SHALL~~ shall account for the effects of local site response.		
SHA-E1	ACCOUNT in the probabilistic seismic hazard analysis for the effects of site topography, surficial geologic deposits, and site geotechnical properties on ground motions at the site. *CC II-III*	In the probabilistic seismic hazard analysis INCLUDE the effects of site topography, surficial geologic deposits, and site geotechnical properties on ground motions at the site. *CC II-III*	The effects of site topography, surficial geologic deposits, and site geotechnical properties will be taken into account by reviewing available licensing documents and other relevant studies in each site's vicinity. Any surficial geological deposits, and the geotechnical properties of subsurface geologic strata, will be taken into account in calculating the site amplification that might occur from incoming rock motions at the base of a sedimentary profile underlying each site. PSHAs conducted under the SPID will meet this requirement directly.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SHA-E2	ADDRESS both the aleatory and epistemic uncertainties in the local site response analysis. *CC II-III*	INCLUDE both the aleatory and epistemic uncertainties in the local site response analysis. *CC II-III*	Aleatory and epistemic uncertainties in local site response will explicitly be addressed in calculating site response: uncertainties will be included in shear-wave velocity for each sand, clay, and soft-rock layer, in the thickness of each sand and clay layer, in the depth to bedrock, and in the shear modulus and damping curves for each sand, clay, and soft-rock layer. PSHAs conducted under the SPID will meet this requirement directly.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SHA-F	Uncertainties in each step of the hazard analysis shall be propagated and displayed in the final quantification of hazard estimates for the site. ~~The results shall include fractile hazard curves, median and mean hazard curves, and uniform hazard response spectra. For certain applications, the probabilistic seismic hazard analysis shall include seismic source deaggregation and magnitude-distance deaggregation.~~		
SHA-F1	In the final quantification of the seismic hazard, INCLUDE and DISPLAY the propagation of both aleatory and epistemic uncertainties. *CC II-III*	In the final quantification of the seismic hazard, INCLUDE uncertainties through a family of hazard curves. *CC II-III*	The final quantification of seismic hazard will include the propagation of both aleatory and epistemic uncertainties by generating multiple seismic hazard curves, each representing an alternative combination of epistemic uncertainties in input models and parameters, and each with an epistemic weight. The propagation of aleatory and epistemic uncertainties will be displayed through fractile hazard curves showing the effects of alternative assumptions on uncertainty in seismic hazard. PSHAs conducted under the SPID will meet this requirement directly.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SHA-F2	In the probabilistic seismic hazard analysis, INCLUDE appropriate sensitivity studies and intermediate results to identify factors that are important to the site hazard and that make the analysis traceable. *CC I-II-III*	In the probabilistic seismic hazard analysis, INCLUDE appropriate sensitivity studies and intermediate results to identify factors that are important to the site hazard and that make the analysis traceable. *CC I-II-III*	For the CEUS, the CEUS Seismic Source Characterization project report includes sensitivity results at 7 test sites that show the effect on seismic hazard of alternative seismic sources and their parameters. In addition, for the CEUS, the Updated Ground Motion Model project report includes sensitivity results at the same 7 test sites that show the effect on seismic hazard of alternative ground motion models and their uncertainties. The WUS licensees will also conduct sensitivity studies as part of their SSHAC Level 3 studies. These sensitivity studies provide identification and traceability from the seismic source and ground motion inputs to the hazard results. Explicit sensitivity results to alternative site amplification models will not be included for each site.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SHA-F3	DEVELOP the following results as a part of the quantification process, compatible with needs for the level of analysis determined in (HLR-SHA-A): (a) fractile and mean hazard curves for each ground motion parameter considered in the probabilistic seismic hazard analysis, (b) fractile and mean uniform hazard response spectrum *CC II*	CALCULATE the following results as a part of the quantification process, compatible with needs for the level of analysis determined in (HLR-SHA-A): (a) fractile and mean hazard curves for each ground motion parameter considered in the probabilistic seismic hazard analysis, (b) uniform hazard response spectra *CC II*	The following results will be developed and presented in the PSHA report: (a) fractile and mean hazard curves for peak ground acceleration and spectral accelerations, for the spectral frequencies at which seismic hazard is calculated. (a) fractile and mean uniform hazard response spectrum, anchored at the spectral frequencies at which seismic hazard is calculated and interpolated in between. PSHAs conducted under the SPID will meet this requirement directly.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SHA-G	For further use in the seismic PRA, the spectral shape shall be based on a site-specific evaluation taking into account ~~the contributions of deaggregated magnitude-distance~~ results of the probabilistic seismic hazard analysis. ~~Broad-band, smooth spectral shapes, such as those presented in NUREG/CR-0098 [5-5] (for lower-seismicity sites such as most of those east of the U.S. Rocky Mountains) are also acceptable if they are shown to be appropriate for the site. The use of uniform hazard response spectra is also acceptable unless evidence comes to light that would challenge these uniform-hazard spectral shapes.~~		
SHA-G1	BASE the response spectral shape used in the seismic PRA on site-specific evaluations performed for the probabilistic seismic hazard analysis. REFLECT or BOUND the site-specific considerations. *CC II*	For the response spectral shape used in the seismic PRA, USE site-specific evaluations performed for the probabilistic seismic hazard analysis. USE or BOUND the site-specific considerations. *CC II*	The final site spectral shape is based on the site amplification calculations that take into account site-specific soil properties. Thus the final site spectral shape reflects site-specific considerations. PSHAs conducted under the SPID will meet this requirement directly.
HLR-SHA-H	When use is made of an existing study for probabilistic seismic hazard analysis purposes, it shall be confirmed that the basic data and interpretations are still valid in light of current information~~, the study meets the requirements outlined in A through G above, and the study is suitable for the intended application~~.		
SHA-H1	Use of existing studies allowed. *CC II*	Confirm that the basic data and interpretations of any existing studies used remain valid in light of established current information, consistent with the requirements in HLR-SHA-A through G, and DESCRIBE the bases and methodology used. *CC I-II-III*	Existing PSHA studies will not be used to characterize seismic hazard at any site. This requirement is not applicable to PSHAs conducted under the SPID.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SHA-I	A screening analysis shall be performed to assess whether in addition to the vibratory ground motion, other seismic hazards, such as fault displacement, landslide, soil liquefaction, or soil settlement, need to be included in the seismic PRA ~~for the specific application. If so, the seismic PRA shall address the effect of these hazards through assessment of the frequency of hazard occurrence or the magnitude of hazard consequences, or both~~.		
SHA-I1	(There are no supporting requirements here.) *CC I-II-III*	DOCUMENT the basis and methodology used for any screening out of the seismic hazards other than the vibratory ground motion. *CC I-II-III*	Screening evaluations for fault displacements should be performed on a site-specific basis[2]. Screening evaluations for landslides, soil liquefaction, and soil settlement should be performed as necessary in the SPRA.
SHA-I2	Not included in Addendum A	For those hazards not screened out, INCLUDE their effect through assessment of the frequency of hazard occurrence and the magnitude of hazard consequences.	The effects of fault displacements should be included in SPRAs if not screened out. The effects of landslides, soil liquefaction, and soil settlement should be included in the SPRA if not screened out.

[2] Fault displacements significant enough to be considered would be identified in the plant UFSAR. If significant fault displacements are not identified in the plant UFSAR, that should be considered an adequate methodology for screening them out for SPRA impacts.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SHA-J	Documentation of the probabilistic seismic hazard analysis shall be consistent with the applicable supporting requirements.		
SHA-J1	DOCUMENT the probabilistic seismic hazard analysis in a manner that facilitates PRA applications, upgrades, and peer review. *CC I-II-III*	DOCUMENT the probabilistic seismic hazard analysis in a manner that facilitates PRA applications, upgrades, and peer review. *CC I-II-III*	The PSHA will be been documented with plots, tables of digital values, and supporting electronic files to facilitate PRA applications, upgrades, and peer review. PSHAs conducted under the SPID will meet this requirement directly.
SHA-J2	DOCUMENT the process used in the probabilistic seismic hazard analysis. For example, this documentation is typically consistent with reference [5-28] and includes a description of: (a) the specific methods used for source characterization and ground motion characterization, (b) the scientific interpretations that are the basis for the inputs and results, and (c) if an existing PSHA is used, documentation to ensure that it is adequate to meet the spirit of the requirements herein. *CC I-II-III*	DOCUMENT the process used in the probabilistic seismic hazard analysis. For example, this documentation is typically consistent with reference [5-28] and includes a description of: (a) the specific methods used for source characterization and ground motion characterization, (b) the scientific interpretations that are the basis for the inputs and results, and (c) if an existing PSHA is used, documentation to ensure that it is adequate to meet the spirit of the requirements herein. *CC I-II-III*	For the WUS, seismic hazards will be developed using the SSHAC Level 3 process, which is expected to meet the applicable documentation process. For the CEUS, the process used for developing the CEUS seismic sources and ground motion equations has been documented with references to supporting documents, experts contacted (with questionnaires and responses), and explanation/justification of alternative models and weights. Specifically: (a) For seismic source and ground motion characterization, the CEUS-Seismic Source Characterization and Updated Ground Motion Model projects documented the ranges of applicable models and data, to characterize epistemic uncertainties. PSHAs conducted under the SPID will meet this requirement through the use of published models used as inputs.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
			(b) The CEUS-Seismic Source Characterization and Updated Ground Motion Model projects documented the scientific interpretations (including tectonic models, geology, seismology, and empirical ground motion data) that were used to select applicable seismic sources and ground motion equations, and to reflect the range of interpretations that is scientifically justified. (c) Existing PSHA studies will not be used to characterize seismic hazard at any site. This requirement is not applicable to PSHAs conducted under the SPID.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SHA-J3	DOCUMENT the sources of model uncertainty and related assumptions associated with the probabilistic seismic hazard analysis. *CC I-II-III*	DOCUMENT the sources of model uncertainty and related assumptions associated with the probabilistic seismic hazard analysis. *CC I-II-III*	The CEUS-Seismic Source Characterization and Updated Ground Motion Model projects documented the sources of model and parameter uncertainties as follows: (a) Alternative RLME and background seismic sources reflect different interpretations on crustal structure and on the homogeneity of that structure. PSHAs conducted under the SPID will meet this requirement through the use of published models used as inputs. (b) Alternative parameters for RLME and background seismic sources reflect alternative interpretations and assumptions on homogeneity and possible earthquake size. PSHAs conducted under the SPID will meet this requirement through the use of published models used as inputs. (c) Alternative ground motion models reflect uncertainties on seismic ground motion in the CEUS. PSHAs conducted under the SPID will meet this requirement through the use of published models used as inputs. (d) Uncertainties in site profile parameters reflect uncertainties in shear-wave velocities, modulus curves, and damping curves appropriate for sites where direct data and measurements are limited. PSHAs conducted under the SPID will meet this requirement directly.

Table 6-5
Comparison of SPID Guidance to ASME/ANS PRA Standard Supporting Requirements: Element SFR

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SFR-A	The seismic-fragility evaluation shall be performed to estimate ~~plant-specific, realistic~~ seismic fragilities of SSCs ~~structures, or systems, or components, or a combination thereof~~ whose failure may contribute to core damage or large early release, or both.		
SFR-A1	DEVELOP seismic fragilities for all those structures, systems, or components, or a combination thereof identified by the systems analysis (see Requirement SPR-D1). *CC I–II-III*	DEVELOP seismic fragilities for SSCs identified by the systems analysis (see Requirement SPR-D1). *CC I-II-III*	No substantive change from Addenda A to Addenda B. A note in the Standard refers to the screening criteria in HLR-SFR-B. Using these criteria, some SSCs do not require explicit fragility calculations, and others may be assessed using screening values. Note that Section 6.4.3 of this document provides explicit guidance for screening SSCs recognized to be very rugged and for performing more limited fragility calculations for certain other SSCs. The SPID does not provide guidance for identifying SSCs relevant to the seismic evaluation, but relies on other guidance. The intent is that the seismic evaluation be consistent with this requirement.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SFR-A2	BASE the seismic fragilities on plant-specific data, and ENSURE that they are realistic (median with uncertainties). Generic data (e.g., fragility test data, generic seismic qualification test data, and earthquake experience data) MAY be used for screening of certain structures, systems, or components, or a combination thereof and for calculating their seismic fragilities by applying the requirements under (HLR-SFR-F), which permits use of such generic data under specified conditions. However, DEMONSTRATE that any use of such generic data is conservative. *CC II*	CALCULATE the seismic fragilities based on plant-specific data, and ENSURE that they are realistic (median with uncertainties). Generic data (e.g., fragility test data, generic seismic qualification test data, and earthquake experience data) MAY be used for screening of certain SSCs and for calculating their seismic fragilities by applying the requirements under (HLR-SFR-F), which permits use of such generic data under specified conditions. However, DEMONSTRATE that any use of such generic data is applicable. *CC II*	The guidance in the SPID is consistent with this requirement. It entails applying simplified or generic values as a screening measure, and performing detailed calculations for SSCs that are important to risk (refer to Section 6.4).

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SFR-B	If screening of high-seismic-capacity components is performed, the basis for the screening shall be fully described (HLR-SFR-B).		
SFR-B1	If screening of high-seismic-capacity components is performed, DESCRIBE fully the basis for screening and supporting documents. For example, it is acceptable to apply guidance given in EPRI NP-6041-SL, Rev. 1, and NUREG/CR-4334 to screen out components with high seismic capacity. However, CHOOSE the screening level high enough that the contribution to core damage frequency and large early release frequency from the screened-out components is not significant. *CC I-II*	If screening of high-seismic-capacity components is performed, DESCRIBE the basis for screening and supporting documents and CHOOSE the screening level high enough that the contribution to core damage frequency and large early release frequency from the screened-out components is not significant. *CC I-II*	Reference to NP-6041 and NUREG/CR-4334 has been removed in Addendum B but retained in a note, and its removal from the supporting requirement does not constitute a substantive change. The guidance in the SPID is consistent with this requirement. Screening criteria are addressed in detail in Section 6.4.3.
SFR-B2	ASSESS and DOCUMENT the applicability of the screening criteria given in EPRI NP-6041-SL, Rev. 1 [5-3] and NUREG/CR-4334 [5-4] for the specific plant and specific equipment. *CC I-II-III*	Deleted. *CC I-II-III*	The deletion of this supporting requirement in Addendum B is consistent with the general nature of the PRA Standard. The guidance for screening in the SPID does not explicitly address these documents, but the approach is consistent with the guidance they provide.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SFR-C	The seismic-fragility evaluation shall be based on ~~realistic~~ a seismic response that SSCs ~~the structures, or systems, or components, or a combination thereof~~ experience at their failure levels.		
SFR-C1	ESTIMATE the seismic responses that the components experience at their failure levels on a realistic basis using site-specific earthquake response spectra in three orthogonal directions, anchored to a ground motion parameter such as peak ground acceleration over a given frequency band. ENSURE that the spectral shape used reflects or bounds the site-specific conditions. *CC I-II*	ESTIMATE the seismic responses that the components experience at their failure levels using earthquake response spectra in three orthogonal directions, anchored to a ground motion parameter such as peak ground acceleration or average spectral acceleration over a given frequency band or ENSURE that the spectral shape used bounds the site-specific conditions. *CC I-II*	Guidance for seismic-response analyses, and particularly for determining whether existing analyses are adequate, is provided in Section 6.3.2. The guidance provided conforms to each of the aspects of Supporting Requirement SFR-C1. A characterization of site-specific earthquakes is being provided for each NPP site as described in Section 2 of this document. The guidance does not provide specific recommendations as to use of peak ground acceleration (PGA) or average spectral acceleration. It is left to the SPRA analysts to assure appropriate alignment with the fragility parameter.
SFR-C2	If probabilistic response analysis is performed to obtain realistic structural loads and floor response spectra, ENSURE that the number of simulations done (e.g., Monte Carlo simulation and Latin Hypercube Sampling) is large enough to obtain stable median and 85% nonexceedance responses. ACCOUNT for the entire spectrum of input ground motion levels displayed in the seismic hazard curves. *CC I-II*	If probabilistic response analysis is performed to obtain realistic structural loads and floor response spectra, ENSURE that the number of simulations done (e.g., Monte Carlo simulation and Latin Hypercube Sampling) is large enough to obtain stable median and 85% nonexceedance responses. ACCOUNT the entire spectrum of input ground motion levels displayed in the seismic hazard curves. *CC I-II*	No change in Addendum B. This consideration is not addressed explicitly in the SPID.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SFR-C3	If scaling of existing design response analysis is used, JUSTIFY it based on the adequacy of structural models, foundation characteristics, and similarity of input ground motion. *CC I-II*	If scaling of existing response analysis is used, JUSTIFY it based on the adequacy of structural models, foundation characteristics, and similarity of input ground motion. *CC I-II*	The only change in Addendum B was to delete the word "design" from "existing design response analysis". Explicit guidance for scaling of response analysis is provided in Section 6.3.3. This guidance provides an appropriate technical approach for Capability Category II. The guidance also addresses the use of response analysis other than that performed for design. Therefore, this supporting requirement should be satisfied for Addendum B, and the intent is satisfied for Addendum A.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SFR-C4	When the design response analysis models are judged not to be realistic and state of the art, or when the design input ground motion is significantly different from the site-specific input motion, PERFORM new analysis to obtain realistic structural loads and floor response spectra. *CC I-II*	When the existing response analysis models are judged not to be realistic and state of the art, or when the existing input ground motion is significantly different from the site-specific input motion, PERFORM new analysis to obtain realistic structural loads and floor response spectra for use in the seismic PRA. *CC I-II*	The changes in Addendum B were to replace "design" with "existing", and to add the condition that new analysis should be performed "for use in the seismic PRA". This document provides guidance on how to characterize whether the existing seismic input motion is significantly different from the new estimate. It also provides guidance on determining whether an existing structural model is realistic and reflective of the state of the art. This guidance meets the intent Capability Category II. The guidance in Section 6.3 of the SPID is consistent with these requirements, except (as noted above), the response analysis need not be that used in the plant design. Therefore, this supporting requirement should be satisfied for Addendum B, and the intent is satisfied for Addendum A.
SFR-C5	If median-centered response analysis is performed, ESTIMATE the median response (i.e., structural loads and floor response spectra) and variability in the response using established methods. *CC I-II*	If median-centered response analysis is performed, ESTIMATE the median response (i.e., structural loads and floor response spectra) and variability in the response using established methods. *CC I-II*	No change in Addendum B. This consideration is not addressed explicitly in the SPID.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SFR-C6	When soil-structure interaction (SSI) analysis is conducted, ENSURE that it is median centered using median properties, at soil strain levels corresponding to the input ground motions that dominate the seismically induced core damage frequency. ACCOUNT for the uncertainties in the SSI analysis by varying the low strain soil shear modulus between the median value times (1 + Cv) and the median value divided by (1 + Cv), where Cv is a factor that accounts for uncertainties in the SSI analysis and soil properties. If adequate soil investigation data are available, ESTABLISH the mean and standard deviation of the low strain shear modulus for every soil layer. Then ESTABLISH the value of Cv so that it will cover the mean plus or minus one standard deviation for every layer. The minimum value of Cv is 0-5. When insufficient data are available to address uncertainties in soil properties, ENSURE that Cv is taken as no less than 1.0. *CC I-II*	When soil-structure interaction (SSI) analysis is conducted, ENSURE that it is median centered using median properties, at soil strain levels corresponding to the input ground motions that contribute most to the seismically induced core damage frequency. INCLUDE the uncertainties in the SSI analysis. *CC I-II*	Addendum B changes the action verb from "ACCOUNT for" to "INCLUDE" and deletes the "how-to" portion of the Addendum A requirement. This consideration is not addressed explicitly in the SPID.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SFR-D	The seismic-fragility evaluation shall be performed for critical failure modes of SSCs ~~structures, or systems, or components, or a combination thereof~~ such as structural failure modes and functional failure modes identified through the review of plant design documents, supplemented as needed by earthquake experience data, fragility test data, generic qualification test data, and a walk down.		
SFR-D1	IDENTIFY realistic failure modes of structures and equipment that interfere with the operability of equipment during or after the earthquake through a review of the plant design documents and the walkdown. *CC I-II-III*	IDENTIFY realistic failure modes of structures (e.g., sliding, overturning, yielding, and excessive drift), equipment (e.g., anchorage failure, impact with adjacent equipment or structures, bracing failure, and functional failure) and soil (i.e., liquefaction, slope instability, and excessive differential settlement) that interfere with the operability of equipment during or after the earthquake through a review of the plant design documents and the walkdown. *CC I-II-III*	Addendum B added examples of failure modes that needed to be addressed but did not change the substance of the supporting requirement. This consideration is not addressed explicitly in the SPID.
SFR-D2	EXAMINE all relevant failure modes of structures (e.g., sliding, overturning, yielding, and excessive drift), equipment (e.g., anchorage failure, impact with adjacent equipment or structures, bracing failure, and functional failure), soil (e.g., liquefaction, slope instability, and excessive differential settlement), and EVALUATE fragilities for critical failure modes. *CC I-II-III*	EXAMINE relevant failure modes identified in SFR –D1 and EVALUATE fragilities for critical failure modes. *CC I-II-III*	Addendum B omitted "all" relative to failure modes, and omitted explicit listing of the failure modes (since they are already presented in Supporting Requirement SFR-D1. This consideration is not addressed explicitly in the SPID. It should be noted, however, that this document does provide guidance regarding liquefaction and soil failure relative to performing the quality review of IPEEE quality review for those plants using the IHS in the screening process.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SFR-E	The seismic-fragility evaluation shall incorporate the findings of a detailed walkdown of the plant focusing on the anchorage, lateral seismic support, and potential systems interactions.		
SFR-E1	CONDUCT a detailed walkdown of the plant, focusing on equipment anchorage, lateral seismic support, spatial interactions and potential systems interactions (both structural and functional interactions). *CC I-II-III*	CONDUCT a detailed walkdown of the plant, focusing on equipment anchorage, lateral seismic support, spatial interactions and potential systems interactions (both structural and functional interactions). *CC I-II-III*	No change in Addendum B. This consideration is not addressed explicitly in the SPID.
SFR-E2	DOCUMENT the walkdown procedures, walkdown team composition and its members' qualifications, walkdown observations, and conclusions. *CC I-II-III*	DOCUMENT the walkdown procedures, walkdown team composition and its members' qualifications, walkdown observations, and conclusions. *CC I-II-III*	No change in Addendum B. This consideration is not addressed explicitly in the SPID.
SFR-E3	If components are screened out during or following the walkdown, DOCUMENT any anchorage calculations and PROVIDE the basis justifying such a screening. *CC I-II-III*	If components are screened out during or following the walkdown, DOCUMENT the basis including any anchorage calculations that justify such a screening. *CC I-II-III*	Wording changes in Addendum B account for the need to document the basis for any screening in addition to that based on anchorage calculations. Screening during the walkdown itself is not addressed explicitly in the SPID. Note, however, that Section 6.4.3 of this document provides explicit guidance for screening SSCs recognized to be very rugged and for performing more limited fragility calculations for certain other SSCs.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SFR-E4	During the walkdown, FOCUS on the potential for seismically induced fire and flooding. *CC I-II-III*	During the walkdown, FOCUS on the potential for seismically induced fire and flooding. *CC I-II-III*	No change in Addendum B. The assessment of seismically-induced fire and flooding is beyond the scope of the evaluations for which guidance is provided in the SPID.
SFR-E5	During the walkdown, EXAMINE potential sources of interaction (e.g., II/I issues,impact between cabinets, masonry walls, flammable and combustion sources, flooding, and spray) and consequences of such interactions on equipment contained in the systems model. *CC I-II-III*	During the walkdown, EXAMINE potential sources of interaction (e.g., II/I issues,impact between cabinets, masonry walls, flammable and combustion sources, flooding, and spray) and consequences of such interactions on equipment contained in the systems model. *CC I-II-III*	No change in Addendum B. This consideration is not addressed explicitly in the SPID.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SFR-F	The calculation of seismic-fragility parameters such as median capacity and variabilities shall be based on plant-specific data ~~supplemented as needed~~ or if necessary on earthquake experience data, fragility test data, and generic qualification test data. Use of such generic data shall be justified.		
SFR-F1	BASE component seismic-fragility parameters such as median capacity and variabilities (logarithmic standard deviations reflecting randomness and uncertainty) on plant-specific data supplemented as appropriate by earthquake experience data, fragility test data, and generic qualification test data. *CC I-II*	CALCULATE component seismic-fragility parameters such as median capacity and variabilities (logarithmic standard deviations reflecting randomness and uncertainty) based on plant-specific data or if necessary based on earthquake experience data, fragility test data, and generic qualification test data. *Exception*: JUSTIFY the use of generic fragility for any SSC as being appropriate for the plant. *CC I-II*	Addendum B changes the action verb from "BASE" to "CALCULATE" and changes the reference to sources beyond plant-specific data slightly. Section 6.4 of the SPID provides some guidance on the performance of the fragility analysis and suggests the use of the CDFM method to limit the amount of detailed analysis required. The SPID does not provide explicit guidance on how to calculate fragility parameters, but SPRA fragility analyses would be expected to satisfy this requirement.
SFR-F2	For all structures, or systems, or components, or a combination thereof (SSCs) that appear in the dominant accident cut sets, ENSURE that they have site-specific fragility parameters that are derived based on plant-specific information, such as anchoring and installation of the component or structure and plant-specific material test data. Exception: JUSTIFY the use of generic fragility for any SSC as being appropriate for the plant. *CC I-II*	For SSCs that appear in the significant accident sequences, ENSURE that they have site-specific fragility parameters that are derived based on plant-specific information, such as anchoring and installation of the component or structure and plant-specific material test data. *Exception*: JUSTIFY the use of generic fragility for any SSC as being appropriate for the plant. *CC I-II*	Addendum B changes the word "dominant" to the convention of "significant" used elsewhere in the PRA Standard. Section 6.4 of the SPID provides guidance on the performance of the fragility analysis that is intended to result in plant-specific estimates for risk-significant SSCs. This is intended to satisfy this supporting requirement.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SFR-F3	DEVELOP seismic fragilities for relays identified to be essential and that are included in the systems-analysis model. *CC II-III*	DEVELOP seismic fragilities for relays identified to be essential and that are included in the systems-analysis model. *CC II-III*	No change in Addendum B. Section 6.4 of the SPID provides guidance on the evaluation of relays. This guidance should be consistent with satisfying this supporting requirement.
SFR-F4	DEVELOP seismic fragilities for structures, or systems, or components, or a combination thereof that are identified in the systems models as playing a role in the large early release frequency part of the seismic PRA. (See Requirements SPR-A1 and SPR-A3). *CC I-II-III*	DEVELOP seismic fragilities for SSCs that are identified in the systems models as playing a role in the large early release frequency part of the seismic PRA. (See Requirements SPR-A1 and SPR-A3). *CC I-II-III*	No substantive change from Addenda A to Addenda B. This document provides guidance for identifying the SSCs that could be relevant for calculating LERF, and for which fragility analyses might be needed.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SFR-G	Documentation of the seismic fragility evaluation shall be consistent with the applicable supporting requirements.		
SFR-G1	DOCUMENT the seismic fragility analysis in a manner that facilitates PRA applications, upgrades, and peer review. *CC I-II-III*	DOCUMENT the seismic fragility analysis in a manner that facilitates PRA applications, upgrades, and peer review. *CC I-II-III*	No change in Addendum B. The SPID identifies specific elements that need to be documented, but it is expected that the fragility analysis will be thoroughly documented in a manner consistent with this supporting requirement.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SFR-G2	DOCUMENT the process used in the seismic fragility analysis. For example, this typically includes a description of: (a) The methodologies used to quantify the seismic fragilities of structures, or systems, or components, or a combination thereof, together with key assumptions, (b) The structure, or system, or component, or a combination thereof (SSC) fragility values that includes the method of seismic qualification, the dominant failure mode(s), the source of information, and the location of the component, (c) The fragility parameter values (i.e., median acceleration capacity, β_R and β_U) and the technical bases for them for each analyzed SSC, and (d) The different elements of seismic-fragility analysis, such as (1) the seismic response analysis, (2) the screening steps, (3) the walkdown, (4) the review of design documents, (5) the identification of critical failure modes for each SSC, and (6) the calculation of fragility parameter values for each SSC modeled. *CC I-II-III*	DOCUMENT the process used in the seismic fragility analysis. For example, this typically includes a description of: (a) The methodologies used to quantify the seismic fragilities of SSCs, together with key assumptions, (b) The structure, or system, or component, or a combination thereof (SSC) fragility values that includes the method of seismic qualification, the dominant failure mode(s), the source of information, and the location of the component, (c) The fragility parameter values (i.e., median acceleration capacity, β_R and β_U) and the technical bases for them for each analyzed SSC, and (d) The different elements of seismic-fragility analysis, such as (1) the seismic response analysis, (2) the screening steps, (3) the walkdown, (4) the review of design documents, (5) the identification of critical failure modes for each SSC, and (6) the calculation of fragility parameter values for each SSC modeled.. *CC I-II-III*	No substantive change in Addendum B. The SPID identifies specific elements that need to be documented, but it is expected that the fragility analysis will be thoroughly documented in a manner consistent with this supporting requirement.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SFR-G3	DOCUMENT the sources of model uncertainty and related assumptions associated with the seismic fragility analysis. *CC I-II-III*	Deleted. *CC I-II-III*	The SPID identifies specific elements that need to be documented, but it is expected that the fragility analysis will be thoroughly documented in a manner consistent with this supporting requirement.

Table 6-6
Comparison of SPID Guidance to ASME/ANS PRA Standard Supporting Requirements: Element SPR

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SPR-A	The seismic-PRA systems model SHALL include seismic-caused initiating events and other failures including seismic-induced SSC failures, non-seismic-induced unavailabilities, and human errors that give rise to significant accident sequences and/or significant accident progression sequences.		
SPR-A1	ENSURE that earthquake-caused initiating events that give rise to significant accident sequences and/or significant accident progression sequences are included in the seismic-PRA system model using a systematic process. *CC I–II-III*	ENSURE that earthquake-caused initiating events that give rise to significant accident sequences and/or significant accident progression sequences are included in the seismic-PRA system model using a systematic process. *CC I–II-III*	No change in Addendum B. The SPID does not provide explicit guidance regarding the general nature of the SPRA model, but it would be expected that the SPRA would satisfy this requirement.
SPR-A2	In the initiating-event selection process, DEVELOP a hierarchy to ensure that every earthquake greater than a certain defined size produces a plant shutdown within the systems model. *CC I-II-III*	In the initiating-event selection process, DEVELOP a hierarchy to ensure that every earthquake greater than a certain defined size produces a plant shutdown within the systems model. *CC I-II-III*	No change in Addendum B. The SPID does not provide explicit guidance relative to this aspect of the SPRA model, but it would be expected that the SPRA would satisfy this requirement.
SPR-A3	USE the event trees and fault trees from the internal-event full-power PRA model as the basis for the seismic event trees. *CC I-II-III*	USE the accident sequences and the systems logic model from the internal-event at-power PRA model as the basis for the seismic PRA model. *CC I-II-III*	The change from Addendum A to Addendum B makes the reference to the type of models used more general. The SPID does not provide explicit guidance, but it is expected that any SPRA would build on the existing models from the internal-events PRA.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SPR-A4 (Add. A) **SPR-A5 (Add. B)**	ENSURE that the PRA systems models reflect earthquake-caused failures and nonseismically induced unavailabilities and human errors that give rise to significant accident sequences or significant accident progression sequences. *CC I-II-III*	***Becomes SPR-A5 in Addendum B.*** ENSURE that the PRA systems models reflect earthquake-caused failures and nonseismically induced unavailabilities and human errors that give rise to significant accident sequences or significant accident progression sequences. *CC I-II-III*	There is no change in the supporting requirement from Addendum A to Addendum B, except that the insertion of a new SPR-A4 in Addendum B changes the number for this requirement. The SPID does not provide explicit guidance, but it is expected that any SPRA would build appropriately incorporate both seismic and non-seismic failures in the PRA model.
SPR-A4 (Add. B)	This supporting requirement is new to Addendum B and not included in Addendum A.	***New SPR-A4 in Addendum B.*** Under special circumstances based on the judgment of the analyst, DEVELOP an *ad hoc* systems model tailored especially to the seismic-PRA configurations or issues being modeled, instead of starting with the internal-events model and adapting it, as in SPR-A3. If this approach is used, ENSURE that the resulting model is consistent with the internal-events systems model regarding plant response and the cause-effect relationships of the failures. *CC I-II-III*	This supporting requirement was added for Addendum B. The SPID does not provide explicit guidance, but it is expected that any SPRA would employ an appropriate approach, consistent with this requirement, in applying any *ad hoc* models.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SPR-B	The seismic-PRA systems model shall be adapted to incorporate seismic-analysis aspects that are different from corresponding aspects found in the at-power full power, internal-events PRA systems model.		
SPR-B1	In each of the following aspects of the seismic-PRA systems-analysis work, SATISFY the corresponding requirements in Part 2, except where they are not applicable or where this Part includes additional requirements. DEVELOP a defined basis to support the claimed nonapplicability of any exceptions. The aspects governed by this requirement are (1) initiating-event analysis; (2) accident-sequence analysis; (3) success-criteria analysis; (4) systems analysis; (5) data analysis; (6) human-reliability analysis; (7) use of expert judgment. When the Part 2 requirements are used, USE the Capability Category designations in Part 2, and for consistency USE the same Capability Category in this analysis. *CC I–II-III*	In each of the following aspects of the seismic-PRA systems-analysis work, SATISFY the corresponding requirements in Part 2, except where they are not applicable or where this Part includes additional requirements. SPECIFY the basis to support the claimed nonapplicability of any exceptions. The aspects governed by this requirement are (1) initiating-event analysis; (2) accident-sequence analysis; (3) success-criteria analysis; (4) systems analysis; (5) data analysis; (6) human-reliability analysis; (7) use of expert judgment. *CC I–II-III*	There was a small wording change to remove "defined" basis and to indicate the need to "specify" the basis rather than to "develop" it. The SPID does not provide explicit guidance regarding following relevant elements of the PRA Standard for internal initiating events, but it would be expected that the SPRA would satisfy these requirements.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SPR-B2	In the human reliability analysis (HRA) aspect, EXAMINE additional postearthquake stresses that can increase the likelihood of human errors or inattention, compared to the likelihood assigned in the internal-events HRA when the same activities are undertaken in nonearthquake accident sequences. Whether or not increases in error probabilities are used, JUSTIFY the basis for this decision about what error rates to use. *CC I–II-III*	INCLUDE the following seismic impacts on Performance Shaping Factors (PSFs) for the control room and ex-control room post-initiator actions as appropriate to the Human Reliability Analysis (HRA) methodology used: (a) Additional post earthquake workload and stress that can increase the likelihood of human errors or inattention (b) Seismic failures that impact access (c) Cue availability *CC I–II*	The requirement related to HRA has been completely re-written for Addendum B (including defining a separate requirement for Capability Category III). The updated requirement clarifies what is expected of the seismic HRA. The SPID does not provide explicit guidance regarding the conduct of the HRA for the seismic PRA, but it would be expected that the SPRA would satisfy this requirement.
SPR-B3 (Add. A) **SPR-B4a (Add. B)**	If any screening is performed, PERFORM it using defined criteria that are documented in the PRA. *CC I–II-III*	***Deleted in Addendum B and addressed by SPR-B5 as follows.*** If screening out on the basis of seismic capacity is performed in the systems model, PERFORM the screening on a documented basis. *CC I-II-III*	This requirement has been deleted in Addendum B because it is covered by Supporting Requirement SPR-B5. Acceptable screening criteria are provided in Section 6.4.3 of the SPID.
SPR-B4 (Add. A) **SPR-B3 (Add. B)**	PERFORM an analysis of seismic-caused dependencies and correlations in a way so that any screening of SSCs appropriately accounts for those dependencies and correlations. USE bounding or generic correlation values and PROVIDE the basis for such use. *CC I-II*	PERFORM an analysis of seismic-caused correlations in a way so that any screening of SSCs appropriately accounts for those correlations. USE bounding or generic correlation values and PROVIDE the basis for such use. *CC I-II*	A relatively minor change in Addendum B was to delete the phrase “dependencies and” in two places. The intent of the requirement did not change. The SPID does not provide explicit guidance regarding the treatment of seismic correlations. The SPRA would be expected to satisfy this requirement.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SPR-B5 (Add. A) **SPR-B4a (Add. B)**	ENSURE that any screening of human-error basic events and non-seismic failure basic events does not significantly affect the PRA results. *CC I-II-III*	***Deleted in Addendum B and replaced by SPR-B4a as follows.*** If screening out on the basis of seismic capacity is performed in the systems model, SPECIFY the screening criterion. *CC I-II-III*	The requirement related to screening of non-seismic failures has been deleted, and a new requirement was added in Addendum B related to screening based on seismic capacity. The SPID provides acceptable guidance regarding screening based on seismic capacity in Section 6.4.3.
SPR-B4b (Add. B)	Not included in Addendum A.	If post-earthquake recovery actions are included in the systems model, INCLUDE them on a basis that is consistent and specified. *CC I-II-III*	This is a new requirement in Addendum B. The SPID does not provide explicit guidance regarding the treatment of recovery, but it is expected that any recovery actions would be appropriate documented in the SPRA.
SPR-B6 (Add. A) **SPR-B4 (Add. B)**	EXAMINE the effects of the chatter of relays and similar devices. *CC II-III*	INCLUDE the effects of the chatter of relays and similar devices in the systems model. *CC I-II-III*	Section 6.4.2 of the SPID provides acceptable guidance for the treatment of chattering due to high frequency ground motions.
SPR-B7 (Add. A) **SPR-B5 (Add. B)**	In the systems-analysis models, for each basic event that represents a seismically caused failure, INCLUDE the complementary "success" state where applicable to a particular SSC. *CC I-II-III*	In the systems-analysis models, for each basic event that represents a significant seismically caused failure, INCLUDE the complementary "success" state where applicable to a particular SSC and SPECIFY the criteria used for the term "significant" in this activity. *CC I-II*	The supporting requirement from Addendum A was retained in Addendum B for Capability Category III, with the word "significant" added for Capability Categories I and II in Addendum B. The SPID does not present explicit guidance relative to this aspect of the systems models; it is expected that a SPRA would satisfy this requirement.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SPR-B8 (Add. A) **SPR-B6 (Add. B)**	EXAMINE the possibility that a large earthquake can cause damage that blocks personnel access to safety equipment or controls, thereby inhibiting operator actions that might otherwise be credited. *CC I-II-III*	EVALUATE the possibility that a large earthquake can cause damage that blocks personnel access to safety equipment or controls, thereby inhibiting operator actions that might otherwise be credited. *CC I-II-III*	The only change from Addendum A to Addendum B was the change in action verb. The SPID does not present explicit guidance relative to this aspect of the treatment of operator response; it is expected that a SPRA would satisfy this requirement.
SPR-B9 (Add. A) **SPR-B7 (Add. B)**	EXAMINE the likelihood that system recoveries modeled in the internal-events PRA may be more complex or even not possible after a large earthquake, and ADJUST the recovery models accordingly. *CC II-III*	EVALUATE the likelihood that system recoveries modeled in the internal-events PRA may be more complex or even not possible after a large earthquake, and ADJUST the recovery models accordingly. It is acceptable to use generic or conservative recovery values. *CC II*	The action verb was changed from Addendum A to Addendum B. Also, the acceptability of using generic or conservative values was added for Capability Category II. The SPID does not present explicit guidance relative to this aspect of the treatment of recovery; it is expected that a SPRA would satisfy this requirement.
SPR-B10 (Add. A) **SPR-B8 (Add. B)**	EXAMINE the effect of including an earthquake-caused "small-small loss-of coolant accident" as an additional fault within each sequence in the seismic-PRA model. *CC I-II-III*	ASSUME the existence of an earthquake-caused "very small loss-of-coolant accident" in the SPRA accident sequences and system modeling, unless it is demonstrated that such a LOCA can be excluded, based on a walkdown or on another examination of the possible sources of such a LOCA. *CC I-II*	The requirement was made more definitive in Addendum B. The SPID does not provide explicit guidance on how to evaluate very small loss-of-coolant accidents, but the SPRA analyses would be expected to satisfy this requirement.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SPR-B11 (Add. A) **SPR-B9 (Add. B)**	In the seismic PRA walkdown, EXAMINE the potential for seismically induced fires and flooding following the guidance given in NUREG-1407. *CC I-II-III*	If the seismic PRA walkdown (see SFR-E4) identifies the potential for seismically induced fires and flooding, INCLUDE potential significant contributions to accident sequences in the systems model. *CC I-II-III*	The action verb was changed from Addendum A to Addendum B. The SPID does not present explicit guidance relative to this aspect of the treatment of recovery; it is expected that the walkdowns would include consideration of seismically induced fire and flooding. Any detailed analyses of scenarios associated with such consequential events would, however, be performed in a later assessment.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SPR-C	The seismic-PRA systems model shall reflect the as-built and as-operated plant being analyzed.		
SPR-C1	To ensure that the systems-analysis model reflects the as-built, as-operated plant, JUSTIFY any conservatisms or other distortions introduced by demonstrating that the seismic-PRA's validity for applications is maintained. *CC I-II-III*	To ensure that the systems-analysis model reflects the as-built, as-operated plant, JUSTIFY any conservatisms or other distortions that do not adequately reflect the as-built, as-operated plant. *CC I-II-III*	The language has been clarified in Addendum B. The SPID does not provide explicit guidance, but it would be expected that the SPRA would satisfy this requirement.
HLR-SPR-D	The list of SSCs selected for seismic-fragility analysis shall include <u>the</u> ~~all~~ SSCs that participate in accident sequences included in the seismic-PRA systems model.		
SPR-D1	USE the seismic PRA systems model as the basis for developing the seismic equipment list, which is the list of all SSCs to be considered by the subsequent seismic-fragility evaluation task. *CC I-II-III*	USE the PRA systems model as the basis for developing the seismic equipment list to support the fragility analysis of Section 5-2.2. INCLUDE structures and passive components that may not be present in the internal events model but which require consideration in the seismic PRA. SUPPLEMENT the list based on review of industry SPRA SELs (if available). *CC I-II-III*	"All" was deleted and the consideration of some structures was added in Addendum B. The SPID does not provide explicit guidance for developing the SEL, but it would be expected that the SPRA would satisfy this requirement.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SPR-E	The analysis to quantify core damage frequency and large early release frequency shall appropriately integrate the seismic hazard, the seismic fragilities, and the systems-analysis aspects.		
SPR-E1	In the quantification of core damage frequency and large early release frequency, PERFORM the integration using the seismic hazard, fragility, and systems analyses. *CC I-II-III*	In the quantification of core damage frequency and large early release frequency, PERFORM the integration using the seismic hazard, fragility, and systems analyses. *CC I-II-III*	No change in Addendum B. The SPID does not provide explicit guidance for performing the quantification, but this integration is a typical task in SPRA, and the SPRA would be expected to satisfy this requirement.
SPR-E2	In quantifying core damage frequency and large early release frequency, PERFORM the quantification on a cut-set-by-cut-set or accident-sequence-by-accident-sequence basis (or for defined groups of these), as well as on a comprehensive/integrated basis. It is acceptable to use broad groupings. *CC I-II*	PERFORM seismic sequence quantification in accordance with applicable requirements described in subsection 2.2.7. *CC I-II-III*	The language was revised in Addendum B to refer to guidance in the internal-events portion of the Standard. The SPID does not provide explicit guidance for performing the quantification, but this integration is a typical task in SPRA, and the SPRA would be expected to satisfy this requirement.
SPR-E3	In the analysis, USE the quantification process to ensure that any screening of SSCs does not affect the results, taking into account the various uncertainties. *CC I-II-III*	USE the quantification process to confirm and support the screening of SSCs (refer to SFR-B1). *CC I-II-III*	The language was clarified in Addendum B. The SPID provides acceptable guidance for the screening process, and the validity of the screening should be confirmed during the quantification.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
SPR-E4	In the integration/quantification analysis, ACCOUNT for all significant dependencies and correlations that affect the results. It is acceptable to use generic correlation values. If used, PROVIDE the basis for such use. *CC I-II*	In the integration/quantification analysis, INCLUDE the significant correlations that affect the results. It is acceptable to use generic correlation values. If used, PROVIDE the basis for such use. *CC I-II*	The language was clarified in Addendum B. The SPID does not provide explicit guidance for treating correlations. The SPRA would be expected to satisfy this requirement.
SPR-E5	USE the mean hazard, composite fragilities, and the systems analysis to generate point estimates for core damage frequency (CDF) and large early release frequency (LERF). ESTIMATE the uncertainties in overall CDF and LERF. *CC I-II*	In the integration/quantification analysis, analysis, INCLUDE in the uncertainties in core damage frequency and large early release frequency results that arise from each of the several inputs (the seismic hazard, the seismic fragilities, and the systems-analysis aspects). *CC II*	The requirement was re-written for Addendum B, and separate requirements are provided for each of the three capability categories. The SPID does not provide explicit guidance for assessing uncertainty in the results, but the SPRA would be expected to satisfy this requirement.
SPR-E6	PERFORM appropriate sensitivity studies to illuminate the sensitivity of the core damage frequency and large early release frequency results to the assumptions used about dependencies and correlations. *CC I-II-III*	Deleted in Addendum B	The SPID does not provide explicit guidance for sensitivity studies for the integrated model. The SPRA would be expected to satisfy this requirement.
SPR-E7 (Add. A) **SPR-E6 (Add. B)**	New requirement in Addendum B.	In the analysis of LERF, SATISFY the LERF requirements in the internal-initiators section (Part 2, Section 2.2.8), where applicable. *CC I-II-III*	This requirement is entirely new for Addendum B. The SPID provides some guidance for assessment of LERF in Section 6.5.1.

Standard Element	Requirements from Addendum A	Requirements from Addendum B	Relevant Intent of Guidance in SPID
HLR-SPR-F	The seismic-PRA analysis shall be documented in a manner that facilitates applying the PRA and updating it and that enables peer review.		
SPR-F1	DOCUMENT the seismic plant response analysis and quantification in a manner that facilitates PRA applications, upgrades, and peer review. *CC I-II-III*	DOCUMENT the seismic plant response analysis and quantification in a manner that facilitates PRA applications, upgrades, and peer review. *CC I-II-III*	No change in Addendum B. The SPID provides guidance for documentation of some aspects of the seismic PRA. The SPRA would be expected to satisfy this requirement.
SPR-F2	Error in Addendum A corrected in the supporting requirement in Addendum B.	DOCUMENT the process used in the seismic plant response analysis and quantification. *CC I-II-III*	The SPID provides guidance for documentation of some aspects of the seismic PRA. The SPRA would be expected to satisfy this requirement.
SPR-F3	DOCUMENT the sources of model uncertainty and related assumptions associated with the seismic plant response model development. *CC I-II-III*	DOCUMENT the sources of model uncertainty and related assumptions associated with the seismic plant response model development. *CC I-II-III*	No change in Addendum B. The SPID provides guidance for documentation of some aspects of the seismic PRA. The SPRA would be expected to satisfy this requirement.

6.7 Peer Review

This section describes the peer review requirements for the activities performed to meet the 50.54(f) letter [1] relative to the seismic 2.1 requests for information. The peer review need not assess all aspects of the SPRA or SMA against all technical requirements; however, enough aspects of the PRA shall be reviewed for the reviewers to achieve consensus on the adequacy of methodologies and their implementation for each PRA or SMA element. Alternative methods and approaches to meet the intent of SPRA/SMA technical requirements may be used if they provide results that are equivalent or superior to the methods usually used, and it is expected that the peer review team should concentrate on reviewing such alternate methods and approaches if they are used.

The peer review team shall have combined experience in the areas of systems engineering, seismic hazard, seismic capability engineering, and other elements of seismic PRAs or seismic margin methodologies. The reviewer(s) focusing on the seismic response and fragility work shall have successfully completed the SQUG Walkdown Screening and Seismic Evaluation Training Course [52] or equivalent.

One of the peer reviewers should be designated as the overall Team Leader. The peer review Team Leader is responsible for the entire peer review process, including completion of the final peer review documentation. The Team Leader is expected to provide oversight related to both the process and technical aspects of the peer review. The Team Leader should also pay attention to potential issues that could occur at the interface between various activities.

The peer review process includes a review of the following SPRA activities:

- Selection of the SSCs included on the SEL
- Seismic hazard assessment[3]
- Documentation from the Seismic Walkdowns
- Seismic response analyses
- Seismic fragility assessments
- Seismic risk quantification
- Final report

The results of the peer review should be documented in a separate report. Specific guidance on the key elements of the peer review process is found in Section 5.3 of the SPRA part of the ASME-ANS PRA Standard [12]

[3] Seismic hazard assessments submitted to and reviewed by NRC as described in Section 4 would not require additional reviews under the Peer Review. Additional hazard work performed for the SPRA (e.g. FIRS estimates) should be reviewed by the Peer Review.

entitled “Peer Review for Seismic Events At-Power.” This guidance is felt to be appropriate for this peer review, with the recommended exception that independent seismic fragility analyses are not required to be performed by the peer reviewers. Adequate peer review of the seismic fragilities can be accomplished (as in past SPRAs and SMAs) based on a review of a sample of the fragility calculations.

For the NTF 2.1 Seismic resolution, it may be preferable to conduct more focused peer review activities for individual SPRA elements during implementation of this program, to the extent practicalable, rather than waiting until all the work is complete. This “in-process” SPRA peer review should include the following elements:

- The SPRA findings should be based on a consensus process, and not based on a single peer review team member
- A final review by the entire peer review team must occur after the completion of the SPRA project
- An “in-process” peer review must assure that peer reviewers remain independent throughout the SPRA development activity

The NRC [22] has provided an additional detailed description of the important elements of the in-process peer review (in their comments to NEI 12-13 on external event PRA peer review guidelines) which should be incorporated into any such peer review for the 50.54(f) seismic 2.1 program.

6.8 SPRA Documentation

Documentation criteria for a SPRA areis identified throughout the ASME/ANS Standard [12]. Utilities are expected to retain that documentation consistent with the Standard.

A report should be submitted to the NRC summarizing the SPRA inputs, methods, and results. The level of detail needed in the submittal should be sufficient to enable NRC to understand and determine the validity of all input data and calculation models used, to assess the sensitivity of the results to all key aspects of the analysis, to make necessary regulatory decisions as a part of NTTF Phase 2 activities. It is not necessary to submit all of the SPRA documentation for such an NRC review. Relevant documentation should be cited in the submittal, and be available for NRC review in easily retrievable form.~~The guideline for judging the adequacy of retained documentation is that independent expert analysts should be able to convince themselves of the correctness and usefulness of the calculations in a straightforward, unambiguous manner. To the extent possible, the retained documentation should be organized along the lines identified in the areas of review. The retained documentation should be such that it can be used for any future applications, if necessary (e.g., if new understanding of the hazard emerges).~~

Appendix C to NUREG 1407 [11] documents the detailed documentation and reporting guidelines associated with the seismic PRA studies conducted as part of the IPEEE program. The pertinent documentation and reporting for the seismic IPEEE program are listed below and are intended to serve as guidance to licensees on elements of the SPRA documentation and/or reporting that may also be appropriate for this 50.54(f) response:

1. A description of the methodology and key assumptions used in performing the ~~Sseismic~~ PRA.
2. The hazard curve(s) (or table of hazard values) used and the associated spectral shape used in the analysis. Also, if an upper bound cutoff to ground motion is assumed, the results of sensitivity studies to determine whether the cutoff affected the overall results and the delineation and ranking of seismic sequences.
3. A summary of the walkdown findings and a concise description of the walkdown team and the procedures used. This summary should include any potential for seismically induced fire or internal flood identified during the walkdowns.
4. All functional/systemic seismic event trees ~~as well as data (including origin and method of analysis)~~. ~~Address to what extent the recommended enhancements have been incorporated in the IPEEE.~~ A description of how non-seismic failures, human actions, dependencies, relay chatter, soil liquefaction, and seismically induced floods/fires are accounted for. ~~Also, a list of important non-seismic failures with a rationale for the assumed failure rate given a seismic event.~~
5. A description of dominant functional/systemic sequences leading to core damage along with their frequencies and percentage contribution to overall seismic core damage frequencies. The description of the sequences should include a discussion of specific assumptions and human recovery actions.
6. The estimated CDF and LERF~~core damage frequency and plant damage state frequencies, the timing of the core damage,~~ including a qualitative discussion of uncertainties and how they might affect the final results, and contributions of different ground motions to ~~core damage frequencies~~CDF and LERF.

~~Any seismically induced containment failures and other containment performance insights. Particularly, vulnerabilities found in the systems/functions which will lead to early containment failure that might result in high consequences. This includes: isolation, bypass, integrity, and systems (e.g., igniters) required to prevent early failure. The computed fragilities of containment components, systems, and functions as applicable should be provided.~~

Comment [c1]: This was moved up.

Comment [c2]: Swap order of 5 and 6

Comment [JMR3]: These insights are covered by LERF above.

7. A table of fragilities used for screening as well as in the quantification. The estimated fragilities for the plant, dominant sequences, and dominant components should be reported where feasible.
8. A discussion of important non-seismic failures and human actions that are significant contributors, or have impacts on results~~.~~, including ~~Also, a list of important non-seismic failures with a~~the rationale for the assumed failure rate given a seismic event.

The 50.54(f) letter [1] requests that specific information be submitted by plants performing a SPRA including the following.

- A list of the significant contributors to SCDF and LERF, including importance measures (e.g., Risk Achievement Worth, FussellVesely and/or Birnbaum)
- A summary of the methodologies used to estimate the SCDF and LERF, and results obtained, including the following:
  - methodologies used to quantify the seismic fragilities of SSCs, together with key assumptions
  - SSC fragility values with reference to the method of seismic qualification, the dominant failure mode(s), and the source of information
  - seismic fragility parameters
  - important findings from plant walkdowns and any corrective actions taken
  - process used in the seismic plant response analysis and quantification, including the specific adaptations made in the internal events PRA model to produce the seismic PRA model and their motivation
  - assumptions about containment performance
- Description of the process used to ensure that the SPRA is technically adequate, including the dates and findings of any peer reviews
- Identified plant-specific vulnerabilities and actions that are planned or taken.

Section 7: Spent Fuel Pool Integrity Evaluation

The 50.54(f) letter requested that, in conjunction with the response to NTTF Recommendation 2.1, a seismic evaluation be made of the SFP. More specifically, plants were asked to consider "…all seismically induced failures that can lead to draining of the SFP." Such an evaluation would be needed for any plants that are not screened from further assessment prior to Step 3a~~5~~ in Figure 1-1.

This section provides guidance that may be employed in addressing this consideration for plant-specific evaluations.

7.1 Scope of the Seismic Evaluation for the SFP

The focus of the evaluation process described in this report is on elements of the SFP that might fail due to a seismic event such that draining of the SFP could result. This approach is intended to ensure that efforts to gain an understanding of potential seismic risks needed to respond to the 50.54(f) letter are appropriately focused on the most ris~~e~~k significant elements.~~make the best possible use of available resources~~.

In developing guidance for the walkdowns associated with NTTF Recommendation 2.3 [46], the emphasis was on SFP connections whose failure could result in "rapid drain-down." The definition of "rapid drain-down" encompasse~~sed~~ failures that could lead to uncovering of irradiated fuel stored in the SFP within 72 hours of the earthquake [46]. This criterion is used for the evaluations under NTTF Recommendation 2.1 as well; that is, the evaluations consider possible failures that could lead to uncovering fuel stored in the SFP within 72 hours.~~ Note that 72 hours is suggested as an upper bound of the time to be considered for this evaluation. The evaluation may be further limited to address only those failures that could drain the SFP in a shorter time if it can be shown that adequate measures are in place to provide SFP inventory makeup sooner. This could be justified if, for example, there is adequate instrumentation to provide indication of the status of the SFP, procedures exist to guide response by the operators, and makeup resources are available and are seismically rugged.~~

Comment [c4]: Human factors needs to be reintroduced. Need to specifically add a second sentence about accounting for hability and accessibility.

Failures that could conceivably lead to uncovering of irradiated fuel stored in the SFP would include the following:

- A significant failure of the steel-lined, reinforced concrete structure of the SFP, causing inventory in the pool to drain out.
- Failure of a connection penetrating the SFP structure (drain line, cooling-water line, etc.) below the top of the stored fuel.
- Failure of a connection penetrating the SFP structure above the fuel sufficient to drain significant inventory from the pool such that (in the absence of adequate makeup) evaporation and boil-off could cause fuel to be uncovered within 72 hours.
- Extensive sloshing such that sufficient water could be lost from the pool and, as in the previous item, lead to uncovering of the fuel within 72 hours.
- Failure of a cooling-water line or other connection that could siphon water out of the pool sufficient to lead to uncovering of the fuel within 72 hours.
- Tearing of the steel liner due to movement of fuel assemblies as a result of the earthquake.
- Failures that could lead to draining of SFP inventory when the pool and reactor are configured for refueling operations.

With regard to these possibilities, the evaluation may generally be focused on connected structures and systems that penetrate the SFP structure, rather than the basic structure of the SFP itself. ~~The rationale for focusing the scope of the evaluation in this manner accounts for the following:~~

Detailed assessments have been made of SFP structural integrity, including by the NRC on several structures, and these have found SFP structures to be reasonably rugged~~; and~~.

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- ~~Even if the SFP were to experience a structural failure that led to draining of its inventory, there should still be an ability to provide makeup to the SFP at a sufficient rate to prevent sustained uncovering of the fuel.~~

Previous evaluations in NUREG-1353 [57], NUREG-1738 [47] and NUREG/CR-5176 [48] characterized the generally robust nature of the design of SFPs currently in use. NUREG-1738 further identified a checklist that could be used to demonstrate that a SFP would achieve a high very HCLPF. Evaluations reported in NUREG/CR-5176 [48] for two older plants concluded that "…seismic risk contribution from spent fuel pool structural failures is negligibly small." In addition, previous screening criteria for civil structures in EPRI NP-6041 [39] (e.g. Table 2-3) provide principles that would be helpful in evaluating the ruggedness of SFP structures. Either the checklist in NUREG-1738 should be used to

demonstrate that the structure is sufficiently robust or another approach can be used if sufficiently justified.

Tearing of the stainless-steel liner due to overall structural failure of the fuel pool structure would be precluded by successful completion of the structural evaluations above. Tearing of the stainless-steel liner due to sliding or other movement of the fuel assemblies in the pool is considered to be very unlikely [57~~49~~].

Beyond the impact of possible failures on the cooling of the fuel stored in the SFP, for some plants the loss of inventory from the pool could cause flooding that could affect other systems. The assessment of flooding will be evaluated separately, as part of the response to a NTTF Tier 3 recommendation.

The remainder of this section outlines a process for identifying and evaluating features that could lead to draining of the SFP.

7.2 Evaluation Process for the SFP

The process for evaluating the SFP begins with the identification of any penetrations that should be considered. All penetrations should be identified and placed into one of the following three categories:

1. Those that are above the level of the fuel in the SFP;
2. Those that are at a level below the top of the fuel in the SFP; and
3. Those that may have the potential to siphon water from the SFP (most typically, the discharge line from the SFP cooling system).

The sections that follow provide guidance for addressing each of these categories. Figure 7-1 shows the general process for evaluating SFP penetrations.

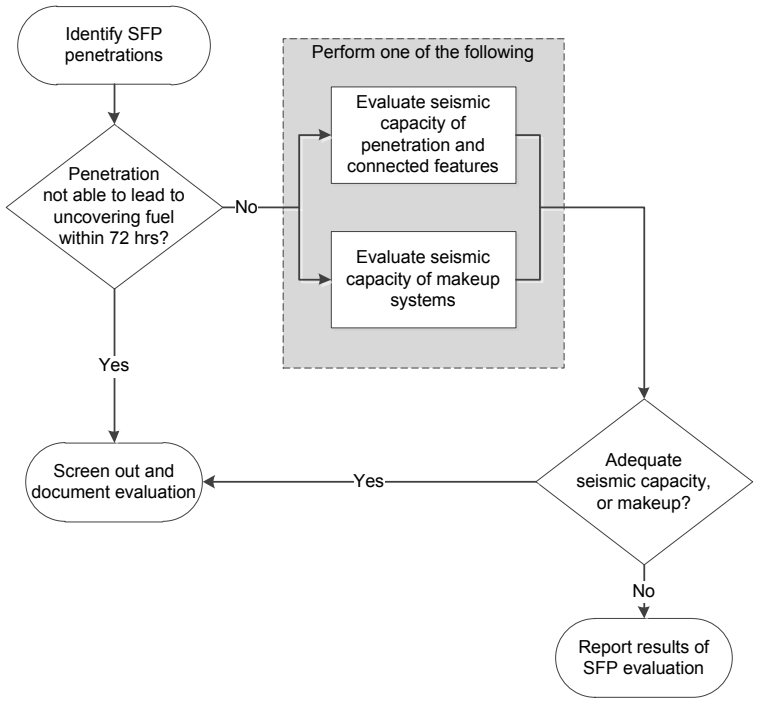

Figure 7-1
Basic Process for Evaluation of Potential Failures for SFP Penetrations

7.2.1 Evaluation of Penetrations above Top of Fuel

In most cases, penetrations in the SFP will be located above the top of the irradiated fuel. Assessment of these penetrations does not need to account for the potential that a failure would, in and of itself, result in draining the pool level below the fuel. Failures of these penetrations could, however, still affect SFP inventory. If the level in the pool could be lowered sufficiently due to a failure associated with a connection via such a penetration, the volume of water in the pool serving as a heat sink for the residual decay heat in the fuel assemblies could be reduced.

In this case, the evaluation should determine whether the potential failure could lead to uncovering the fuel within 72 hours. It is acceptable to evaluate either the seismic adequacy of the penetrations or the makeup capabilities to demonstrate overall SFP adequacy. Plants can choose which of these to address first (that is, the seismic capacity of the penetration or the availability of makeup adequacy). The evaluation should include the following.

- References identified in Table 6-1 provide acceptable guidance for computing seismic fragilities that ~~The criteria in NP-6041 [39]~~ can be used to evaluate the seismic adequacy of SFP features. Computed SSC HCLPFs would be compared with GMRS based demands for these evaluations.
- For a relatively large potential failure (such as that of the fuel transfer gate), the analysis should begin with an assumption that the level in the SFP drops to the bottom of the penetration at essentially the same time as when the failure occurred. For smaller failures, the time required to lower pool level to the bottom of the penetration may be significant (refer to Section 7.3.1 for guidance).
- The amount of water lost due to sloshing (refer to Section 7.3.2 for guidance) should be taken into account.

For a failure associated with a penetration above the top of the fuel, the loss of inventory through the break will be limited to the level of the penetration. Therefore, the makeup requirements are only those associated with matching decay heat. If it is necessary to consider makeup capabilities, the evaluation should confirm that the makeup systems have adequate seismic capacity to address the needs for restoring and maintaining SFP inventory.

Maintaining the SFP water level above about two-thirds of the height of the fuel assemblies in the pool should prevent overheating the fuel [49]. Therefore, the ability to maintain SFP inventory at a level of about two-thirds of the height of the fuel assemblies would be considered acceptable.

The makeup required to match decay heat if the SFP does not have fuel assemblies freshly removed from the reactor may be as low as 20 to 30 gpm. For an SFP that contains freshly offloaded fuel, the decay heat load may be significantly higher. Plants routinely maintain information needed to calculate the heat load in the SFP. Guidance for calculating the required makeup rates can be found in Appendix EE of the report documenting the technical bases for severe accident management guidance (SAMG) [49].

The evaluation should document the assessment of the penetrations, including the provisions for makeup to prevent uncovering the stored fuel. The cumulative effect of inventory losses from penetrations not screened out as seismically adequate should be considered in the evaluation, in addition to sloshing and boil off losses. If limitations are identified with respect to the capability of makeup systems, these results should be reported as part of the SFP seismic evaluation.

7.2.2 Evaluation of Penetrations below Top of Fuel

The SFPs for plants operating in the United States are generally configured so that they do not have penetrations below the top of the stored fuel. The absence of penetrations lower in the pool inherently

limits the potential to drain inventory sufficiently to begin uncovering fuel. It is possible; however, that some SFPs may have penetrations (e.g., drain lines) below the top of the stored fuel assemblies. There may also be some SFPs for which the bottom of the transfer gates is below the top of the fuel. A failure associated with such a penetration could drain the pool level below the top of the fuel.

The evaluation should include the following.

- References identified in Table 6-1 provide acceptable guidance for computing seismic fragilities that ~~The criteria in NP-6041 [39]~~ can be used to evaluate the seismic adequacy of SFP features. Computed SSC HCLPFs would be compared with GMRS based demands for these evaluations.
- For a relatively large potential failure (such as that of the fuel transfer gate), the analysis should begin with an assumption that the level in the SFP drops to the bottom of the penetration at essentially the same time as when the failure occurred. For smaller failures, the time required to lower pool level to the bottom of the penetration may be significant (refer to Section 7.3.1 for guidance).
- The amount of water lost due to sloshing (refer to Section 7.3.2 for guidance) should be taken into account.

The evaluation should confirm that the makeup systems have adequate seismic capacity to address the needs for restoring and maintaining SFP inventory. One consideration is that a significant failure low in the pool has the potential to drain water from the pool at a rate in excess of readily available makeup provisions. If the penetration is above about two-thirds of the height of the fuel assemblies in the pool, however, maintaining the water level at that point should prevent overheating the fuel [49]. So, for example, if the transfer gate extends down to 2 ft below the top of the fuel, its failure may be acceptable, even though it may not be possible to restore water level to above the top of the fuel.

The evaluation should document the assessment of the penetrations, including the provisions for makeup to prevent uncovering the stored fuel. The cumulative effect of inventory losses from penetrations not screened out as seismically adequate should be considered in the evaluation, in addition to sloshing and boil off losses. If limitations are identified with respect to the capability of makeup systems, these results should be reported as part of the SFP seismic evaluation.

7.2.3 Evaluation of Potential for Siphoning Inventory

Although designs differ from plant to plant, for some SFPs the discharge line from the SFP cooling system extends down into the pool. Cool water is introduced low in the pool, and the suction line takes warm water from closer to the top of the pool. If the SFP cooling system were to experience a failure, it is possible that water could be siphoned back through the

discharge line and out the break. To prevent such an occurrence, SFP cooling systems with this configuration are typically equipped with anti-siphon devices. If the anti-siphon device were to function improperly, the effect could ~~essentially~~ be similar to a break below the top of the fuel, as addressed in Section 7.2.2. The process for evaluating failures in the SFP cooling system that might lead to siphoning inventory from the pool is outlined in Figure 7-2.

The anti-siphon devices are expected to be relatively rugged; for purposes of this evaluation, an evaluation should be performed to confirm that, if such a feature is needed to prevent siphoning water from the pool. If there are questions about the ruggedness of the feature, the evaluation may follow one of three paths, depending on what information is most readily available:

- The seismic capacity of the anti-siphon feature can be assessed using the fragility references listed in Table 6-1 and the resulting HCLPF compared to the GMRS;
- The SFP cooling system can be examined to determine if there are effective isolation features that could be used to terminate the loss of inventory; or
- An evaluation of makeup capabilities could be made, as for other breaks below the level of the fuel.

In the very unlikely event that none of these options is viable, an evaluation can be made of the seismic capacity of the SFP system (analogous to the assessment called for in Section 7.2.2 for penetrations below the top of the stored fuel).

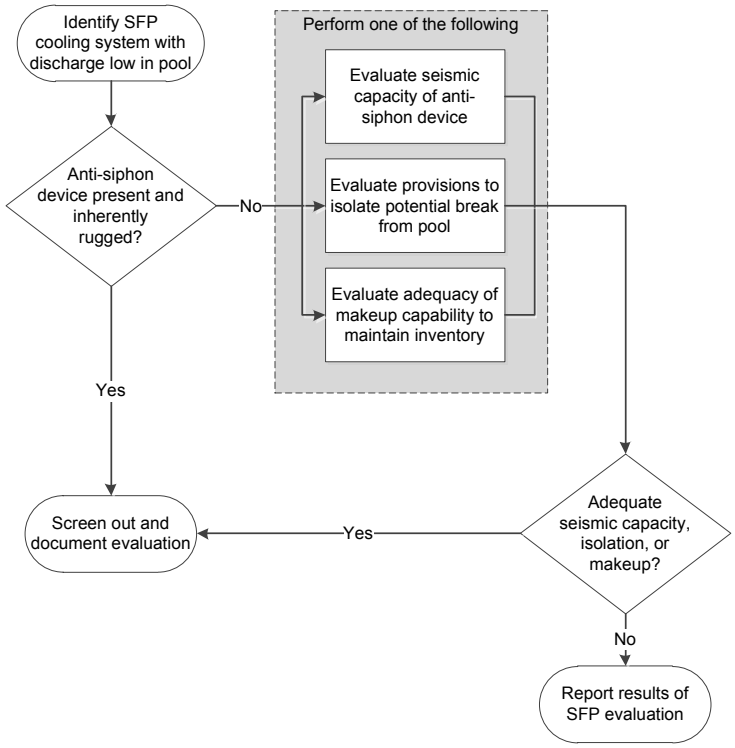

Figure 7-2
Basic Process for Evaluation of Potential Siphoning of SFP Inventory

7.3 Guidance for Additional Evaluations

To accomplish the tasks outlined in the preceding section, additional evaluations may be required. This section provides guidance for the assessment of the timing of uncovering fuel and for addressing the effects of sloshing.

7.3.1 Drain-down and Evaporative Losses

The evaluation of whether fuel could be uncovered in the event of a failure of an interconnection at a level above the fuel can be accomplished in a relatively straightforward manner.

For failures of piping systems connected above the top of the fuel, a flow rate can be approximated using standard correlations, and assuming a driving head equivalent to the initial height of water above the top of the connection. This flow rate can be used to bound the time it would take lower level to that of the connection.

For larger connections (such as the gate used for transferring fuel during refueling), the level can be assumed to drop to the bottom of the connection nearly instantaneously.

Once level drops to the connection, a calculation can be made to determine the additional loss of inventory that occurs up to 72 hours in the absence of makeup flow. This rate of boil off loss can be determined using the correlations provided in Appendix EE of the report documenting the technical bases for severe accident management guidance [49].

These evaluations can be used to determine whether the top of the fuel could begin to be uncovered within 72 hours.

7.3.2 Assessment of the Potential for Sloshing

To support the assessments described in Section 7.2, an estimate is needed of the amount of water lost from the SFP due to sloshing. An initial, bounding assessment can be made using the approach described in this section.

The natural frequency (f_{c1}) for the fundamental convective (sloshing) mode of vertical oscillation of the water surface in a rectangular pool due to shaking input in either horizontal direction can be expressed as follows:

$$f_{c1} = (1/(2\pi))[3.16g / L) \tanh(3.16h / L)]^{0.5} \qquad \text{Equation 7-1}$$

where: L = pool length in the direction of shaking
h = water depth
g = gravity

Next, the slosh height (h_{s1}) for the fundamental convective mode can be estimated from:

$$h_{s1} = \tfrac{1}{2}L(SA_{c1} / g) \qquad \text{Equation 7-2}$$

where: SA_{c1} = ½% damped horizontal spectral acceleration at the top of the pool wall at the frequency f_{c1} in the direction of motion

In order to account for higher convective modes of sloshing and nonlinear sloshing effects (more upward splash than downward movement) observed during stronger shaking, the theoretical slosh height predicted by Equation 7-2 may be increased by 20%. Thus, the total estimated slosh height becomes:

$$h_s = 0.6L(SA_{c1} / g) \qquad \text{Equation 7-3}$$

For a rectangular pool of length a in the x-direction, and width b in the y-direction, the slosh height due to x-direction shaking, and y-direction shaking can be computed independently by substituting a and b,

respectively, into Equations 7-1 and 7-3. Next, the total slosh height (h_{st}) can be estimated from:

$$h_{st} = [h_{sx}^2 + h_{sy}^2]^{0.5} \qquad \text{Equation 7-4}$$

where: h_{sx} = slosh height due to x shaking
h_{sy} = slosh height due to y shaking

An upper bound estimate of the total volume V of water that might splash out of the pool can be estimated from:

$$V = (h_{st} - h_f)ab \qquad \text{Equation 7-5}$$

where: h_f = freeboard height of the wall above the top of the water

Note that this approach reflects that sloshing in a pool is a very low frequency phenomenon governed by either the peak ground displacement or the peak ground velocity of the ground motion. It is independent of the PGA of the ground motion.

While this approach is expected to produce a reasonable estimate of the slosh height, it is expected to produce a very conservative estimate of the volume of water displaced from the pool. It effectively assumes that a solid mass of water equivalent to the product of the splash height above the side of the pool and the pool area is lost from the pool.

This relatively simple calculation is adequate for purposes of estimating the loss of SFP inventory due to sloshing. For most scenarios, it is judged that this conservative estimate of the inventory lost due to sloshing will not have a significant effect on the estimate of SFP drain-down. If the inventory lost due to sloshing has a significant impact on SFP drain-down, a more careful calculation may be required. Such a calculation~~s~~ would need to account for the time histories of a range of earthquakes, and is likely to require significant resources, including an independent review.

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Appendix A: Control Point Discussion from Standard Review Plan

NUREG-0800 USNRC Standard Review Plan Rev. 2 1989:

> The FSAR 2.5.2 Vibratory Ground Motion described the development of the site SSE. Typically a peak ground acceleration (PGA) is determined and a generic spectral shape was defined; e.g., Housner spectra, Modified Newmark spectra, RG 1.60 spectra.
>
> In FSAR 3.7.1 the implementation of the SSE ground motion for seismic analysis and design is described. As discussed above the methodologies for seismic analysis and design varied depending on the vintage of the Plant.

NUREG-0800 Rev. 2 August 1989 provides the acceptance criteria for the later set of existing NNP designs:

> 3.7.1 Seismic Design Parameters states under 1. Design Ground Motion the following:
>
> > "The control motion should be defined to be a free ground surface...Two cases are identified depending on the soil characteristics at the site...uniform sites of soil or rock with smooth variation of properties with depth, the control point (location at which the control motion is applied) should be specified on the soil surface at the top of finished grade...for sites composed of one or more thin soil layers overlaying a competent material...the control point is specified on an outcrop or a hypothetical outcrop at a location on the top of the competent material..."
>
> 3.7.2 Seismic System Analysis states under II Acceptance Criteria the following:

"Specific criteria necessary to meet the relevant requirements of GDC 2 and Appendix A to Part 100 are as follows:

4. Soil-Structure Interaction...

C. Generation of Excitation System...

The control point...for profile consisting of component soil or rock, with relatively uniform variations of properties with depth the control motion should be located...at top of the finish grade...For profiles consisting of one or more thin soil layers overlaying component material, the control motion should be located at an outcrop (real or hypothetical) at top of the competent material...

...The spectral amplitude of the acceleration response spectra (horizontal component of motion) in the free field at the foundation depth shall be not less than 60 percent of the corresponding design response spectra at the finish grade in the free field..."

Appendix B: Development of Site-Specific Amplification Factors

B1.0 Introduction

It has long been recognized that the amplitude and frequency content of ground motions at a site are strongly influenced by the characteristics of the near-surface materials. For most sites, however, the properties of the near-surface materials and the parameters that control the dynamic response are not known with certainty. The uncertainty in these parameters needs to be accounted for when developing site specific hazard curves. Ultimately, the goal or objective of the site response analysis is to produce site-specific hazard curves and response spectra which reflect the desired exceedance frequencies, that is, preserve the reference site annual exceedance frequency (AEF) thereby maintaining hazard consistency for results produced at any elevation in the profile. However, the uncertainty in characterizing the soil profile and dynamic properties of the near-surface materials presents a challenge to preserving hazard levels for sites that differ from some specified reference condition.

Previously, the state of practice in calculating a site-specific ground motion has been to calculate probabilistic reference rock ground motions and then multiply them by deterministic site-amplification factors [17, 18]. However, as stated above, there is uncertainty in the layering, spatial distribution and dynamic properties of near-surface materials. This leads to uncertainty in the estimation of site amplification functions. To alleviate this problem it is necessary to calculate the effects of uncertainty on the estimate of the site-amplification functions and use the resulting site-amplification distribution within a probabilistic methodology [24, 16, 8].

The first step in developing site-specific seismic hazard curves and response spectra consists of performing a Probabilistic Seismic Hazard Analysis (PSHA) that reflects an outcropping reference site condition. The reference site condition is usually hard or firm rock and is consistent with assumptions made in the development of the most recent ground motion prediction equations. For central and eastern North America (CENA) this represents a site with a theoretical shear wave velocity over the top 1 km

of the crust of 2.83 km/sec with a specified shallow crustal damping parameter [18]. The shear-wave velocity is based on the empirical Mid-continent compressional-wave velocity model of Pakiser and Mooney (1989) [28], taken by EPRI (1993) to represent the CENA, and an assumed Poisson ratio. For western U.S. (WUS) sites an appropriate reference condition should be selected that is well-constrained by observational data in the ground motion prediction equations (GMPEs). Site-specific amplification functions are then developed relative to the reference site condition.

After completing PSHA calculations for reference rock site conditions, the development of hazard consistent, site specific horizontal seismic hazard results may be considered as involving two independent analyses. The first is the development of frequency and amplitude dependent relative amplification factors (for 5% damped response spectra, Sa) between the site of interest and the reference site $(Sa_{SITE}(f)/Sa_{REFERENCE}(f))$ that accommodates potential linear or nonlinear site response. Currently the state-of-practice approach involves vertically propagating shear-waves and approximations using equivalent-linear analysis with either a time domain method (e.g. SHAKE) or a more computationally efficient frequency domain random vibration theory (RVT) method [32].

Subsequent to the development of the amplification factors, site-specific motions are computed by scaling the reference site motions with the transfer functions. As suggested above, probabilistic methods have been developed [24, 8] that accurately preserve the reference site hazard level and result in full site-specific hazard curves. These fully probabilistic approaches represent a viable and preferred mechanism to properly incorporate the site-specific aleatory (randomness) and epistemic (uncertainty) variabilities of the dynamic properties and achieve desired hazard levels and performance goals. The following sections describe the specific steps in the development of the site-specific amplification functions.

B2.0 Description of Sites Requiring Response Analysis and Basis for Alternative Models

It has long been recognized that the amplitude and frequency content of ground motions at a site are strongly influenced by the characteristics of the near-surface materials. For most sites, however, the properties of the near-surface materials and the parameters that control the dynamic response are not known with certainty. The uncertainty in these parameters needs to be accounted for when developing site specific hazard curves. Ultimately, the goal or objective of the site response analysis is to produce site-specific hazard curves and response spectra which reflect the desired exceedance frequencies, that is, preserve the reference site annual exceedance frequency (AEF) thereby maintaining hazard consistency for results produced at any elevation in the profile. However, the uncertainty in characterizing the soil profile and dynamic

properties of the near-surface materials presents a challenge to preserving hazard levels for sites that differ from some specified reference condition.

Previously, the state of practice in calculating a site-specific ground motion has been to calculate probabilistic reference rock ground motions and then multiply them by deterministic site-amplification factors [17, 18]. However, as stated above, there is uncertainty in the layering, spatial distribution and dynamic properties of near-surface materials. This leads to uncertainty in the estimation of site amplification functions. To alleviate this problem it is necessary to calculate the effects of uncertainty on the estimate of the site-amplification functions and use the resulting site-amplification distribution within a probabilistic methodology [24, 16, 8].

The first step in developing site-specific seismic hazard curves and response spectra consists of performing a Probabilistic Seismic Hazard Analysis (PSHA) that reflects an outcropping reference site condition. The reference site condition is usually hard or firm rock and is consistent with assumptions made in the development of the most recent ground motion prediction equations. For central and eastern North America (CENA) this represents a site with a theoretical shear wave velocity over the top 1 km of the crust of 2.83 km/sec with a specified shallow crustal damping parameter [18]. The shear-wave velocity is based on the empirical Mid-continent compressional-wave velocity model of Pakiser and Mooney (1989) [28], taken by EPRI (1993) to represent the CENA, and an assumed Poisson ratio. For western U.S. (WUS) sites an appropriate reference condition should be selected that is well-constrained by observational data in the ground motion prediction equations (GMPEs). Site-specific amplification functions are then developed relative to the reference site condition.

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approaches represent a viable and preferred mechanism to properly incorporate the site-specific aleatory (randomness) and epistemic (uncertainty) variabilities of the dynamic properties and achieve desired hazard levels and performance goals. The following sections describe the specific steps in the development of the site-specific amplification functions.

B2.1 Background on the Treatment of Uncertainties

There are two different types of uncertainty in the development of site-specific amplification functions (AF(*f*)). First, at any given site, at the spatial dimensions of typical nuclear facilities (100-200m (~325-650ft) scale dimensions) there is expected to be some variability in important site response parameters such as shear-wave velocities, non-linear dynamic material properties at any depth across the footprint of the facility, and the overall thickness of soil/soft rock above firm rock site conditions. It is important to attempt to capture this uncertainty in the final AF(*f*) estimates. This is treated as an aleatory (randomness) type of variability. Current practice represents this variability by developing a candidate shear-wave velocity profile, depth and overall thickness of soil/soft rock and associated non-linear dynamic material properties (shear modulus reduction and damping curves). This is referred to as a "base case" model. Subsequently, potential variations in shear-wave velocity and layer thickness are represented by correlated random perturbations to the base-case values. This is frequently referred to as a randomization process. A sufficient number of realizations (30 or more) are used to develop statistical estimates (log median and log standard deviation) of the amplification functions.

The second type of uncertainty is epistemic or lack of knowledge uncertainty. This represents the uncertainty in the development of the base-case models for site profile, dynamic properties, and seismological parameters. For well-characterized sites with abundant high-quality data this uncertainty would be reduced, possibly eliminating the need to vary some of the site parameters such as the site profile. This epistemic uncertainty would increase with decreasing confidence in the available data and information. This uncertainty is evaluated through the development of alternative base-case models. The approach applied for the development of alternative base-case models (epistemic uncertainty) is discussed in more detail in the following sections.

The following information is required to perform the site-specific response analyses: site shear-wave velocity profiles, non-linear dynamic material properties, estimates of low-strain site damping (parameterized through the parameter, kappa), and input or control motions (including relevant seismological parameters). These various factors are discussed individually in the following sections.

B3.0 Development of Base-Case Profiles and Assessment of Epistemic Uncertainty in Profiles and Dynamic Material Properties

Epistemic uncertainty in depth to hard rock site conditions and dynamic material properties, which includes shear-wave velocity profiles, site material damping at low strain (parameterized through kappa), and modulus reduction and hysteretic damping curves, should be accommodated through the development of alternative mean cases. The specific methodology utilized to develop the alternative cases will depend on the amount of information available at a given site. Conceptually in this context, for poorly characterized sites with few if any measured dynamic material properties, multiple cases should be developed with broad ranges of epistemic uncertainty applied in the development of the parameters of the alternative cases. For sites that have more complete site characterization information available, smaller epistemic uncertainty factors can be employed in the development of the alternative cases.

For those cases where limited or no at-site information is available, a minimum of three profile estimates combined with three kappa estimates and two sets of modulus reduction and hysteretic damping curves should be developed. If significant uncertainty exists in the thickness of soil above firm or hard rock conditions, this thickness should be treated as an epistemic uncertainty. The three cases for shear-wave velocity profiles and kappa are referred to hereafter as base-case, and upper-range and lower-range models. A general set of guidelines should be employed to develop these cases for dynamic material properties and associated weights and is described more fully below. The general computational framework for developing the mean site amplification functions and associated standard deviations is illustrated in Figure B-1.

B-3.1 Development Process for Base-Case Shear-Wave Velocity Profiles

In order to predict site response as accurately as possible, and ultimately prevent error from propagating into other engineering calculations, it is important to define a detailed shear wave velocity (V_S) profile that represents the known or inferred in-situ velocity structure as realistically as possible. The following discussion describes the development of the mean or base-case V_S profile. The alternative (upper-range and lower-range) models are derived from the base-case model utilizing an information-informed epistemic factor. The development of the upper-range and lower-range models is discussed in Section B-3.2 after the base-case development.

For sites with sparse or very limited information regarding dynamic material properties (e.g., a measured shear-wave velocity profile was unavailable), typically an estimate based on limited surveys (e.g.,

compressional-wave refraction) is available over some shallow, limited depth range. For such cases, as well as to provide a basis for extrapolating profiles specified over shallow depths to hard rock basement material, a suite of profile velocity templates has been developed, parameterized with V_{S30} (time averaged shear-wave velocity over upper 30m of the profile) ranging from 190m/s to 2,032m/s (620ft/s to 6670ft/s). The suite of profile templates is shown in Figure B-2 to a depth of 305m (1,000ft). The templates are from [40] supplemented for the current application with profiles for V_{S30} values of 190m/s, 1,364m/s, and 2,032m/s. The latter two profiles were added to accommodate cases where residual soils (saprolite) are present and overly hard rock. For both soil and soft rock sites, the profile with the closest velocities over the appropriate depth range should be adopted from the suite of profile templates and adjusted by increasing or decreasing the template velocities or, in some cases, stripping off material to match the velocity estimates provided.

For sites with no direct velocity measurements of any type and limited geological information it may be necessary to utilize a proxy to estimate near-surface shear-wave velocity. There are four currently employed V_{S30} proxies, surficial geology [42, 41], Geomatrix site category [14], topographic slope [39], and terrain [43]. Analysis of these proxies suggests a relatively constant variability of measured V_{S30} about the predicted value amongst the various methods [34]. If proxy methods are used to infer near-surface shear-wave velocities, additional levels of epistemic uncertainty may need to be considered (see Section B-3.2).

For intermediate cases, such as when only the upper portion of a deep soil profile is constrained with measured velocities, the V_S template profile with velocities closest to the observed velocity at the appropriate depth should be identified. This template can then be used to provide a rational basis to extrapolate the profile to the required depth.

For soft or firm rock sites, which are often composed of Cenozoic or Paleozoic sedimentary rocks such as shales, sandstones, siltstones, or similar rock types, a constant shear-wave velocity gradient of 0.5m/s/m (0.5ft/s/ft) should be used as a template and used to estimate the velocities over the appropriate depth range. This gradient is based on deep measurements in similar rock types in Japan [20]. The 0.5m/s/m velocity gradient is also consistent with measurements in sedimentary rocks of similar type at the Varian well in Parkfield, California [22]. It is recognized that the soil or firm rock gradients in the original profiles are primarily driven by confining pressure and may not be strictly correct for each adjusted profile template at each site. However, any shortcoming in the assumed gradient is not expected to be significant as the range in multiple base-case profiles accommodates the effects of epistemic uncertainty in the profile gradient on the resulting amplification functions.

For all sites where limited data exists, or only exists for very shallow depths, it is necessary to fully evaluate and integrate all existing geological information into the development of the base-case profile. For sites with soil or soft rock at the surface and much stiffer materials at relatively shallow depths (less than approximately 60m (200ft)) the potential for strong resonance in the frequency range of engineering interest exists. All relevant geological information should be assessed to ensure this condition is identified.

An example is provided in Figure B-3 to schematically illustrate how a combination of geological information and geophysical measurements may be used to develop a base-case profile. The data available at this hypothetical site consists of shallow shear-wave velocity measurements (a single S-wave refraction profile) over only about the upper ~30m (100ft) of the profile with a V_{S30} of approximately 450m/s. There are also geologic profiles and regional data available in the FSAR that indicate firm rock is present at a depth about 45m (150ft) beneath the site. A shear-wave velocity of approximately1525m/s (5000ft/s) is inferred for the firm rock based on velocity measurements on comparable material elsewhere. Regional data indicates the firm, sedimentary rocks extend to a depth of at least 1 km before crystalline basement rock is encountered. The information is combined in the following manner to construct a base case profile. The closest template profile to the 450m/s V_{S30} estimate is the 400m/s profile. The velocities in the 400m/s template are scaled by a factor of 1.125 (450/400) to adjust to the desired V_{S30} value. At the 45m (150ft) depth, a velocity discontinuity is inserted with a velocity of 1525m/s (5000ft/s). Below this depth the firm rock gradient model of 0.5m/s/m is used to estimate velocities. This gradient is extended to a sufficient depth such that 2830m/sec is reached or the depth is greater than the criteria for no influence on response for frequencies greater than 0.5Hz. The uncertainty in the depth to the soil-firm rock interface is incorporated in the treatment of epistemic uncertainty as discussed below.

B-3.2 Capturing Epistemic Uncertainty in Velocity Profiles

There are basically two approaches for constructing shear wave velocity profiles, either through inference from geotechnical/geologic information or through the use of geophysical measurements. Each approach will inherently have some level of uncertainty associated with its ability to accurately represent the in-situ velocity structure. The level of uncertainty will depend on the amount of information available along with how well the information is correlated with shear-wave velocity. By adopting the general mean based approach outlined in Section B-3.1, a level of uncertainty can be assigned to a template velocity profile, commensurate with the available information, in order to account for the epistemic uncertainty associated with the in-situ velocity structure.

For sites where geophysical information such as very limited shear-wave velocity data exists or compressional-wave velocities are used to infer shear-wave velocities, the estimate for uncertainty in shear-wave velocity is to be taken as:

$$\sigma_{\mu}\ln = 0.35$$

This value is similar to a Coefficient of Variation (COV) of 0.25 which is consistent with Toro (1997) [37] for observed spatial variability over a structural footprint of several hundred meters. The profile epistemic uncertainty factor of 0.35 (σ_{μ} ln) is to be applied throughout the profile and is based on the estimates of epistemic uncertainty in V_{S30} developed for stiff profiles [14]. The logarithmic factor assumes shear-wave velocities are lognormally distributed and was originally developed to characterize the epistemic uncertainty in measured V_{S30} at ground motion recording sites where measurements were taken within 300m (900ft) from the actual site. The uncertainty accommodates spatial variability over maximum distances of 300m, and is adopted here as a reasonable and realistic uncertainty assessment reflecting a combination of: (1) few velocity measurements over varying depth ranges, (2) shear-wave velocities inferred from compressional-wave measurements, and (3) the spatial variability associated with observed velocities. While velocities are undoubtedly correlated with depth beyond 30m, which forms the basis for the use of V_{S30} as an indicator of relative site amplification over a wide frequency range, clearly the correlation is neither perfect nor remains high over unlimited depths [11]. An example of the resulting mean ± σ_{μ} ln shear-wave profiles for the 760m/s template is shown in Figure B-4.

For sites where site-specific velocity measurements are particularly sparse (e.g., based on inference from geotechnical/geologic information rather than geophysical measurements) a conservative estimate of the uncertainty associated with the template velocity is to be taken as:

$$\sigma_{\mu}\ln = 0.5$$

For sites where multiple, detailed shear-wave velocity profiles are available, the level of uncertainty may be taken as zero if justified. For sites with an intermediate level of information available, such as a single shear-wave velocity profile of high-quality, a reduced σ_{μ} (ln) value may be applied if justified.

B-3.2.1 Epistemic Uncertainty in Final Hazard Calculations

It is necessary to represent the epistemic uncertainty in the distribution of potential shear-wave velocity profiles (mean base-case and a σ_{μ} (ln) of 0.35, for example) in the final site-specific hazard results. Practicality requires this be accomplished with the minimum number of cases. The recommended approach for this application is to utilize three cases, the

mean base-case and upper range and lower range base-cases with relative weights applied. An accurate three point approximation of a normal distribution which preserves the mean utilizes the 50th-percentile (median) and 10th and 90th percentiles, with relative weights of 0.30 for the 10th and 90th percentiles and 0.40 for the median applied [23]. These values are summarized in Table B-1. The 10th and 90th fractiles correspond to a profile scale factor of 1.28 σ_{μ}. When σ_{μ} ln = 0.35 the 10th or 90th percentiles are obtained by subtracting or adding 0.45□ in natural log units to the shear wave velocity. For σ_{μ} ln = 0.5, a value of 0.64 is subtracted or added to the natural log of the shear wave velocity for the 10th and 90th percentile values. This is equivalent to an absolute factors of 1.57 or 1.90 applied to the mean base-case profile for σ_{μ} ln = 0.35 or σ_{μ} ln = 0.5, respectively. Figure B-5 illustrates application of these two factors applied to the 760 m/s (1525 ft/s) V_{S30} template. Figure B-6 illustrates the same type of curves for the firm rock template derived using the empirical gradient of [20]. For some individual sites it may be necessary to deviate from these standard weights if application of the standard factors results in velocities that are not deemed credible.

Figure B-7 illustrates the development of Upper Range and Lower Range profiles to accommodate epistemic uncertainties for the hypothetical example shown in Figure B-3. A σ_{μ} ln = 0.35 has been used to develop the 10th and 90th-percentile curves in the upper portion of the profile where sparse Vs measurements were available. A σ_{μ} ln = 0.50 was applied to the lower portion of the profile where the Vs of the Base Case was inferred from geological information. The 90th-percentile curve was capped at a value equal to the 2830m/s Vs value assumed for the hard rock basement. This example illustrates the broad range of velocities encompassed by the Upper Range, Mean, and Lower Range profiles for sites lacking in good data.

For sites where the depth to firm rock conditions is poorly constrained, that depth should be treated as a separate epistemic uncertainty as illustrated in Figure B-1.

B-3.3 Nonlinear Dynamic Material Properties

The potential nonlinear response of near-surface materials to input ground motions is an important element of the site that needs to be characterized in a proper site response analysis. To characterize the epistemic uncertainty in nonlinear dynamic material properties for both soil, and soft/firm rock sites, the use of two sets of modulus reduction and hysteretic damping curves is suggested.

For soils, the two sets of proposed curves are the EPRI (1993) and Peninsular Range [32, 40] results. The two sets of generic curves are appropriate for cohesionless soils comprised of sands, gravels, silts, and low plasticity clays. The EPRI (1993) curves, illustrated in Figure B-8,

were developed for application to CENA sites and display a moderate degree of nonlinearity. The EPRI (1993) curves are depth (confining pressure) dependent as shown in Figure B-8.

The Peninsular Range curves reflect more linear cyclic shear strain dependencies than the EPRI (1993) curves [40] and were developed by modeling recorded motions as well as empirical soil amplification in the Abrahamson and Silva WNA (Western North America) GMPE [32, 2]. The Peninsular Range curves reflect a subset of the EPRI (1993) soil curves with the 51 to 120 ft (15 to 37 m) EPRI (1993) curves applied to the 0 to 50 ft (0 to 15 m) depth range and the EPRI (1993) 501 to 1,000 ft (153 to 305 m) curve applied to the 51 to 500 ft (15 to 152 m) depth range, below which linear behavior is assumed.

The two sets of soil curves are considered to reflect a realistic range in nonlinear dynamic material properties for cohesionless soils. The use of these two sets of cohesionless soil curves implicitly assumes the soils considered do not have response dominated by soft and highly plastic clays or coarse gravels or cobbles. The presence of relatively thin layers of hard plastic clays are considered to be accommodated with the more linear Peninsular Range curves while the presence of gravely layers are accommodated with the more nonlinear EPRI (1993) soil curves, all on a generic basis. The potential impact on the amplification functions of the use of these two sets of nonlinear dynamic property curves was evaluated and is shown in Figure B-9. The results indicate that above 1 Hz the difference can be significant and the resulting epistemic uncertainty needs to be included in the development of the final amplification functions.

The two sets of soil curves are given equal weights (Table B-1 and Figure B-1) and are considered to represent a reasonable accommodation of epistemic uncertainty in nonlinear dynamic material properties for the generic types of soils found at most in CEUS sites which include:

1. Glaciated regions which consist of both very shallow Holocene soils overlying tills as well as deep soils such as the Illinois and Michigan basins, all with underlying either firm rock (e.g., Devonian Shales) and then hard basement rock or simply hard basement rock outside the region of Devonian Shales,

2. Mississippi embayment soils including loess,

3. Atlantic and Gulf coastal plain soils which may include stiff hard clays such as the Cooper Marl,

4. Residual soils (saprolite) overlying hard metamorphic rock along the Piedmont and Blue Ridge physiographic regions.

For soft or firm rock site conditions, taken generally as Paleocene sedimentary rocks, such as shale, sandstones, or siltstones, two

alternative expressions of nonlinear dynamic material behavior are proposed: the EPRI "rock curves" (Figure B-10) and linear response. The EPRI rock curves were developed during the EPRI (1993) project by assuming firm rock, with nominal shear-wave velocities in the range of about 914m/s to 2134m/s (3,000ft/s to about 7,000ft/s, about 5,000ft/s on average), behaves in a manner similar to gravels [18] being significantly more nonlinear with higher damping than more fine grained sandy soils. The rock curves were not included in the EPRI report as the final suite of amplification factors was based on soil profiles intended to capture the behavior of soils ranging from gravels to low plasticity sandy clays at CEUS nuclear power plants. With the stiffness typically associated with consolidated sedimentary rocks, cyclic shear strains remain relatively low compared to soils. Significant nonlinearity in the soft-to-firm rock materials is largely confined to the very high loading levels (e.g. ≥ 0.75g).

As an alternative to the EPRI rock curves, linear response should be assumed. Implicit in this model is purely elastic response accompanied with damping that remains constant with cyclic shear strain at input loading levels up to and beyond 1.5g (reference site). Similar to the two sets of curves for soils, equal weights were given to the two sets of nonlinear properties for soft/firm rock sites as summarized in Table B-1.

B-3.4 Densities

Because relative (soil surface/reference site) densities play a minor role in site-specific amplification, a simple model based on the shear-wave velocity of the mean base-case profile is proposed for those sites where a profile density is not available. This model relating estimated shear-wave velocity and density is summarized in Table B-2.

Due to the square root dependence of amplification on the relative density, a 20% change in soil density results in only a 10% change in amplification and only for frequencies at and above the column resonant frequency. As a result only an approximate estimate of profile density is considered necessary with the densities of the mean base-case profile held constant for the upper and lower range base-case profiles. This approach provides a means of accommodating epistemic uncertainty in both density as well as shear-wave velocity (Section B-3.1) in the suite of analyses over velocity uncertainty.

B-4.0 Representation of Aleatory Variability in Site Response

To accommodate the aleatory variability in dynamic material properties that is expected to occur across each site (at the scale of the footprint of a typical nuclear facility), shear-wave velocity profiles as well as G/G_{MAX} and hysteretic damping curves should be randomized. The aleatory variability about each base-case set of dynamic material properties should

be developed by randomizing (a minimum of thirty realizations) shear-wave velocities, layer thickness, depth to reference rock, and modulus reduction and hysteretic damping curves. For all the sites considered, where soil and firm rock extended to depths exceeding 150m (500ft), linear response can be assumed in the deep portions of the profile [32, 33, 35, 36].

B-4.1 Randomization of Shear-Wave Velocities

The velocity randomization procedure makes use of random field models [37] to generate V_S profiles. These models assume that the shear-wave velocity at any depth is lognormally distributed and correlated between adjacent layers. The layer thickness model also replicates the overall observed decrease in velocity fluctuations as depth increases. This realistic trend is accommodated through increasing layer thicknesses with increasing depth. The statistical parameters required for generation of the velocity profiles are the standard deviation of the natural log of the shear-wave velocity (σ_{lnVs}) and the interlayer correlation (ρ_{IL}). For the footprint correlation model, the empirical σ_{lnVs} is about 0.25 and decreases with depth to about 0.15 below about 15m (50ft) [32]. To prevent unrealistic velocity realizations, a bound of $\pm 2\sigma_{lnVs}$ should be imposed throughout the profile. In addition, randomly generated velocity should be limited to 2.83km/s (9200ft/s). All generated velocity profiles should be compared to available site-specific data as a check to ensure that unrealistic velocity profiles are removed (and replaced) from the set of velocity profiles used to develop site response amplification functions. This process should be documented as part of the site response analysis.

B-4.2 Aleatory Variability of Dynamic Material Properties

The aleatory variability about each base-case set of dynamic material properties (EPRI depth dependent vs. Peninsular for example) will be developed by randomizing modulus reduction and hysteretic damping curves for each of the thirty realizations. A log normal distribution may be assumed with a σ_{ln} of 0.15 and 0.30 at a cyclic shear strain of $3 \times 10^{-2}\%$ for modulus reduction and hysteretic damping respectively [32]. Upper and lower bounds of $+/-2\sigma$ should be applied. The truncation is necessary to prevent modulus reduction or damping models that are not physically realizable. The distribution is based on an analysis of variance of measured G/G_{MAX} and hysteretic damping curves and is considered appropriate for applications to generic (material type specific) nonlinear properties [32]. The random curves are generated by sampling the transformed normal distribution with a σ_{ln} of 0.15 and 0.30 as appropriate, computing the change in normalized modulus reduction or percent damping at $3 \times 10^{-2}\%$ cyclic shear strain, and applying this factor at all strains. The random perturbation factor is then reduced or tapered near the ends of the strain range to preserve the general shape of the

base-case curves [18, 32]. Damping should be limited to a maximum value of 15% in this application.

B-5.0 Development of Input Motions

The ground motion used as input to site response analyses is commonly referred to as the "control motion." This can be reflected in time histories matched or scaled to a response spectrum or, in the case of Random Vibration Theory, a power spectral density (PSD). Because of the very large number of cases that will need to evaluated to capture the range of epistemic uncertainty and aleatory variability in this application (See Figure B-1 and Table B-3) the following discussion will assume that the much more efficient random vibration theory (RVT) approach to performing site response analyses will be utilized as opposed to a time series (TS) based technique. Recent studies [29] have confirmed that the two approaches yield similar results. The following sections describe the model used in the development of the control motions and the parameters of that model that require an assessment of uncertainty.

B-5.1 Simple Seismological Model to Develop Control Motions

The methodology suggested for developing the input or control motions relies on a widely used, simple seismological model to represent earthquake source, propagation path and site characteristics ([10] and references therein). The ground motions recorded at a given site from an earthquake can be represented in the frequency domain as:

$$Y(M_o, R, f) = E(M_o, f) \cdot P(R, f) \cdot G(f).$$

Where $Y(M_o,R,f)$ is the recorded ground motion Fourier amplitude spectrum, $E(M_o,f)$ is the Brune point-source seismic spectrum, $P(R,f)$ represents the propagation path effects, and $G(f)$ represents the modification due to site effects. In this equation M_o is the seismic moment of the earthquake, R is the distance from the source to the site, and f is frequency. The seismic moment and the earthquake magnitude are related through the definition of the moment magnitude, M [21]:

$$M = 2/3 Log_{10} M_o - 10.7.$$

The $P(R, f)$ term accounts for path effects, geometrical spreading and frequency dependent deep crustal damping and can be expressed as:

$$P(R,f) = S(R)\, exp((-\pi f R)/(Q(f)V_S)).$$

Where $S(R)$ is the geometrical spreading function, $Q(f)$ is the seismic quality factor, and V_S is the shear-wave velocity in the upper crust.

The *G(f)* term accounts for upper crustal amplification and frequency-independent shallow crustal damping:

$$G(f) = A(f) \cdot D(f).$$

Where *A(f)* is the amplification function relative to source depth velocity conditions and *D(f)* represents the frequency-dependent damping term ($D(f) = exp(-\pi\kappa_0 f)$). The *A(f)* term may be calculated using simplified square-root impedence methods ($A(f) = (Zsource/Zavg)^{0.5}$), where *Z* is the product of density and velocity (ρV_S), for example) or using more detailed full resonant techniques.

Kappa (κ_0) is an upper crustal site ground motion attenuation parameter that accounts for the overall damping in the basement rock immediately beneath a site. The properties and behavior of the upper few hundreds of meters of the crust has been shown to produce as much as 50% or more of the total attenuation (*Q(f)*) of the high-frequency portion of the ground motion spectrum [1, 4]. The value of kappa influences the shape of the ground motion spectrum observed at a given site. High values of kappa result in enhanced attenuation of the high-frequency portion of the spectrum.

The factors in the simple seismological model that affect the spectral shape of the input motions are kappa, magnitude, attenuation model and source model. These factors are discussed below. An example of the potential effect of these parameters on the spectral shape of the input ground motions (Fourier amplitude spectra and 5%-damped response spectra) is shown in Figure B-11 (input parameters are summarized in Table B-7).

B-5.1.1 Magnitude

Conditional on reference site peak acceleration, amplification factors depend, to some extent, upon control motion spectral shape due to the potential nonlinear response of the near-surface materials. For the same reference site peak acceleration, amplification factors developed with control motions reflecting **M** 5.5 will differ somewhat with those developed using a larger or smaller magnitude, for example.

Figure B-12 shows amplification factors developed for the 400m/s V_{S30} template profile (Figure B-2) using the single-corner source model for magnitudes **M** 5.5, 6.5, and 7.5. For this sensitivity analysis the more nonlinear EPRI G/G_{max} and hysteretic damping curves (Figure B-8) were used. The dependence on control motion spectral shape is observed to decrease with degree of nonlinearity becoming independent for linear analyses. As Figure B-12 illustrates, the largest amplification reflects the lowest magnitude (**M** 5.5). Over the frequency range of about 5 to 10 Hz, and the ground motion amplitude range of most engineering interest (between 0.1 g and 0.75 g), the difference in the derived amplification

functions between the magnitudes is minor. The largest difference in amplification is about 20% and at the highest loading levels (≥ 0.75g). The largest difference in amplification is between **M** 5.5 and **M** 6.5 with little difference (< 10%) between **M** 6.5 and **M** 7.5. Given the most current source characterization model in CENA [26] and the distribution of existing NPP sites, the dominant contribution for the annual exceedance frequencies (AEF) of 10^{-4} and below are from magnitudes in the range of about **M** 6 to **M** 7+. Given these factors, and the large number of analyses required (Table B-3) a single magnitude (**M** 6.5) is proposed for development of the control motions. This is felt to adequately characterize the amplification, with tacit acceptance of slight conservatism for magnitude contributions above about **M** 7.

B-5.1.2 Attenuation (*Q(f)*) Model

As illustrated in Figure B-11, major differences in the assumed crustal attenuation model will influence the spectral shape of the control motions. However, within a given tectonic region, the CENA or the WUS for example, changes in the crustal attenuation model do not contribute significantly to changes in the derived amplification functions. Appropriate, widely referenced crustal attenuation models are proposed for the CEUS and WUS sites (Table B-4).

B-5.1.3 Kappa

In the context of this discussion, the kappa referred to here is the profile damping contributed by both intrinsic hysteretic damping as well as scattering due to wave propagation in heterogeneous material. Both the hysteretic intrinsic damping and the scattering damping within the near-surface profile and apart from the crust are assumed to be frequency independent, at least over the frequency range of interest for Fourier amplitude spectra (0.33 to about 25.0 Hz). As a result, the kappa estimates reflect values that would be expected to be measured based on empirical analyses of wavefields propagating throughout the profiles at low loading levels and reflect the effective damping or "effective" Q_s within the profile [12]. Changes in kappa can exert a strong impact on derived control motion spectra (Figure B-11) and as a result are an important part of the input model for development of control motions. Hence, similar to the treatment of uncertainties in shear-wave velocity profiles, multiple base-cases (mean and upper and lower ranges) may be developed for kappa.

B-5.1.3.1 Development of Base Case Kappa Models

Mean base-case kappa values were developed differently for soil and firm rock sites.

Rock Sites: For rock sites with at least 3,000ft (1000m) of firm sedimentary rock (V_{S30} > 500 m/s) overlying hard rock, the kappa-V_{S100} (average shear-wave velocity over the upper 100ft of the profile) relationship of

$$\log(\kappa) = 2.2189 - 1.0930*\log(V_{S100}),$$

Where V_{S100} is in ft/s, is proposed to assign a mean base-case estimate for kappa [36, 38]. The requirement of a 3000ft (1,000m) thickness of firm materials reflects the assumption that the majority of damping contributing to kappa occurs over the upper km of the crust with a minor contribution from deeper materials (e.g., 0.006s for hard rock basement material). As an example, for a firm sedimentary rock with a shear-wave velocity of 5,000ft/s (1525m/s), this relationship produces a kappa estimate of about 0.02s. The assumption that is typically used implies a kappa of 0.014s is contributed by the sedimentary rock column and 0.006s from the underlying reference rock (Table B-4), and reflects an average Q_s of about 40 over the 3,000ft depth interval. The Q_s value of 40 for sedimentary rocks is consistent with the average value of 37 observed (measured) over the depth range of 0m to 298m in Tertiary claystones, siltstones, sandstones, and conglomerates at a deep borehole in Parkfield, California [22].

For soft/firm rock cases with low estimated velocity values, an upper bound kappa value of 0.04s should be imposed. The maximum kappa value of about 0.04s reflects a conservative average for soft rock conditions [31, 32].

For cases where the thickness of firm rock is less than about 3,000ft (1000m) and the relationship cited above is not applicable, the kappa contributed by the firm rock profile can be computed assuming a Q_s of 40 plus the contribution of the reference rock profile of 0.006s (Table B-4). For the three base-case firm rock template profiles shown in Figure B-6, the total kappa values assuming a Q_s of 40 are 0.019s, 0.025s, and 0.015s for the mean, lower range, and upper range base-cases respectively.

Soil Sites:

For soil sites (either in the WUS or CEUS) with depths exceeding 1000m (3,000ft) to hard rock, a mean base-case kappa of 0.04s should be assumed based upon observed average values for deep soil sites and low loading levels. The mean base-case kappa of 0.04s adopted for deep firm soils is lower than the value of approximately 0.06s based on recordings at alluvium sites located in Southern California [4, 32]. For soil sites, due to nonlinear effects, low strain kappa may be overestimated depending upon loading level and the nonlinear dynamic material properties. To avoid potential bias in the deep, firm soil, low strain kappa, the value of 0.04s is based on inversions of the Abrahamson and Silva [3] soil site GMPE [32]. In that inversion, a range of rock site loading levels was used

with the soil value of 0.04s based upon a rock site peak acceleration of 5% g or less, clearly a low strain estimate. The deep soil mean base-case kappa of 0.04s is adopted for both the upper and lower range profiles with the assumption that the suite of profiles reflect deep firm soils. The assumed kappa of 0.04s for deep (≥ 1000m (3,000ft)) firm soils in the CEUS is somewhat less than the 0.054s inferred by Campbell [12] based on Cramer et al. [15] analyses for effective Q_s within the 960m deep sedimentary column in the Mississippi embayment near Memphis, Tennessee. The deep firm soil kappa of 0.04s is in fair agreement with 0.052s found by Chapman, et al. [13] for the 775m thick sedimentary column near Summerville, South Carolina.

In summary, for deep firm soil sites (≥ 1000m (3,000ft) to basement rock) in the CEUS, a nominal kappa value of 0.04s based on an average of many empirical estimates predominately in the WNA tectonic regime is proposed. Sparse analyses for deep soil sites in the CEUS suggest 0.04s reflects some conservatism. However it should be noted the small strain total kappa is rapidly exceeded (i.e., becomes less important) as loading level increases due to nonlinear response. The initial low strain kappa serves primarily as a means of adjusting (lowering) kappa to accommodate the scattering component due to the profile randomization. Hence, no significant bias in the final amplification functions at loading levels of engineering interest is anticipated.

For cases of shallower soils, less than 1000m (3,000ft) to hard rock basement material, the empirical relation of Campbell [12] should be used for the contribution to kappa from the thickness of the sediment column (H):

(ms) = 0.0605*H (thickness in meters).

The assumed basement kappa value of 0.006s (Table B-4) is used in lieu of Campbell's [7] estimate of 0.005s to estimate the soil contribution to total kappa. For 1000m (3,000ft) of soil, Campbell's [12] relation predicts a total kappa of 0.0665s (0.0605 contribution from soil and 0.006 contribution from basement rock) , considerably larger than the mean base-case value of 0.04s, suggesting a degree of conservatism at low loading levels for CENA firm soils. For continuity, in the implementation of Campbell's equation, a maximum kappa of 0.04s should be implemented for sites with less than 1000m (3,000ft) of firm soils.

B-5.1.3.2 Representation of Epistemic Uncertainty in Kappa

The parameter kappa is difficult to measure directly. Since no measurements of the type required exist at the sites of interest, a large uncertainty is applied in the site response analyses. Epistemic uncertainty in kappa is taken as 50% ($\sigma_{\mu Ln}$ = 0.40, Table B-1; [18]) about the mean

base-case estimate for this assessment. The uncertainty is based on the variability in kappa determined for rock sites which recorded the 1989 **M** 6.9 Loma Prieta Earthquake [18], and adopted here as a reasonable expression of epistemic uncertainty at a given site. As with the shear-wave velocity profiles (Section 3.2.1), the +/-0.51 natural log units (1.28 σ_μ) variation is considered to reflect 10% and 90% fractiles with weights of 0.30 and a weight of 0.40 for the mean base-case estimate. The models for epistemic uncertainty are summarized in Table B-1.

B-5.1.4 Source Model

Alternative conceptual models to represent the earthquake source spectral shape exist in the literature. A single corner frequency model of the earthquake source spectrum has been widely used in the simple seismological model described above [10 and 18]. However, based on the limited ground motion data in CENA as well as inferences from intensity observations, an alternative empirical two-corner source model for CENA earthquakes has been developed [5]. The two-corner source model addresses the potential for CENA source processes to reflect a significant spectral sag at large magnitude (**M** ≥ 6) and intermediate frequency [6], compared to source processes of tectonically active regions. Such a trend was suggested by the 1988 **M** 5.9 Saguenay, Canada and 1985 **M** 6.8 Nahanni, Canada earthquakes. The two-corner source model for CENA [6] incorporates the spectral sag between two empirical corner frequencies which are dependent on magnitude. The two-corner model merges to the single-corner model for **M** less than about **M** 5. Interestingly, the two-corner model has been implemented for tectonically active regions and shown to be more representative of WNA source processes than the single-corner model [7], albeit with a much less pronounced spectral sag than the CENA model.

The debate regarding the applicability of these two source models continues. The lack of relevant observations for **M** >6 in CENA precludes identifying either model as a unique, preferred model. As a result, in the interest of representing the epistemic uncertainty in this element of the control motions, both single- and two-corner [6] source models were used with **M** 6.5 to develop control motions. The two models were considered to reflect a reasonable range in spectral composition for large magnitude CENA sources. As a result, equal weights were selected as shown in Table B-1 to develop amplification factors using each source model.

Additionally, for moderately stiff soils, typical for NPP siting, the difference in amplification between single- and double-corner source models becomes significant only at the higher loading levels as Figure B-13 illustrates. Figure B-13 compares the amplification computed for both the single- and double-corner source models using the EPRI modulus reduction and hysteretic damping curves (Figure B-8), the most nonlinear set of curves for soils. These results suggest the difference in amplification between single- vs. double-corner source models are significant enough to

consider the implied epistemic uncertainty in central and eastern North America (CENA) source processes at large magnitude (**M** >6).

B-5.1.4.1 Development of Input Motions

It is necessary to define the site response over a broad range of input amplitudes to develop amplification functions. For sites in the CEUS, the Mid-continent crustal model [18] (Table B-5) with a shear-wave velocity of 2830 m/s, a defined shallow crustal damping parameter (kappa; [4]) of 0.006 s, and a frequency dependent deep crustal damping Q model of 670 $f^{0.33}$ [18] is used to compute reference motions (5% damped pseudo absolute acceleration spectra). The selected Q(f), kappa, and reference site shear-wave velocities are consistent with the EPRI GMPEs (Ground Motion Prediction Equations) [19]. The site-specific profiles are simply placed on top of this defined crustal model which has a reference shear-wave velocity of 2830 m/s (≈ 9,300 ft/s) and a reference kappa value of 0.006 s. Distances are then determined to generate a suite of reference site motions with expected peak acceleration values which cover the range of spectral accelerations (at frequencies of 0.5, 1.0, 2.5, 5.0, 10.0, 25.0, 100.0 Hz) anticipated at the sites analyzed. To cover the range in loading levels, eleven expected (median) peak acceleration values at reference rock are needed to span from 0.01g to 1.50g. Table B-4 lists the suite of distances for the single-corner source model and Table B-6 lists the corresponding distances for the double-corner model.

Amplification factors (5% damping response spectra) are then developed by placing the site profile on the Mid-continent crustal model at each distance with the input motion being equal to the reference rock motion convolved with a diminution function which implements the site specific kappa (e.g. kappa from the equations in Section 5.1.3 and a 0.006s contribution from basement rock) , generating soil motions, and taking the ratios of site-specific response spectra (5% damped) to hard rock reference site response spectra. For the higher levels of rock motions, above about 1 to 1.5g for the softer profiles, the high frequency amplification factors may be significantly less than 1, which may be exaggerated. To adjust the factors for these cases an empirical lower bound of 0.5 is to be implemented [18, 3].

The general framework for the site response calculations are summarized in Figure B-1 and Tables B-1 and B-3.

B-6.0 Development of Probabilistic Hazard Curves

The procedure to develop probabilistic site-specific soil hazard curves was described by McGuire et al. (2001) and by Bazzurro & Cornell (2004) [24, 8]. That procedure (referred to as Approach 3) computes a site-specific soil hazard curve for the spectral acceleration at a selected spectral frequency (or period) given the site-specific hazard curve for the bedrock

spectral acceleration at the same oscillator period and site-specific estimates of soil response. The soil response is quantified through the period/frequency-dependent amplification factor, AF(f). The function AF(f) is given by:

$$AF(f) = Sa_{SOIL}(f)/Sa_{ROCK}(f),$$

where f is frequency, and $Sa_{SOIL}(f)$ and $Sa_{ROCK}(f)$, are the 5% damped spectral accelerations at the soil surface and bedrock, respectively. Since the near-surface materials frequently exhibit nonlinear behavior, the variation of AF(f) with input intensity needs to be captured. Most commonly the input intensity is quantified by Sa_{ROCK} at the frequency of interest.

In the fully probabilistic approach, the annual probability of exceedance of soil ground motion level z ($G_Z(z)$) at spectral frequency f is computed as:

$$G_z(z) = \int_0^\infty P\left\langle AF \geq \frac{z}{x}\middle| x\right\rangle f_x(x)dx$$

Where $P\left\langle AF \geq \frac{z}{x}\middle| x\right\rangle$ is the probability that AF is greater than the quantity z/x, given a bedrock amplitude of x, and $f_X(x)$ is the probability density function of Sa_{ROCK}.

In discretized form, the above equation can be expressed as:

$$G_Z(z) = \sum_{all\ x_j} P\left[\left(AF \geq \frac{z}{x}\middle| x_j\right)\right] p_x(x_j)$$

Where $p_x(x_j)$ is the annual probability of occurrence for Sa_{ROCK} equal to x_j. This probability is obtained by differentiating the appropriate rock hazard curve. Then, $P\left[\left(AF \geq \frac{z}{x}\middle| x_j\right)\right]$ can be computed by assuming AF is lognormally distributed and a function of x, since

$$P\left[\left(AF \geq \frac{z}{x}\middle| x\right)\right] = \hat{\varphi}\left(\frac{\ln\frac{z}{x} - \mu_{lnAF|x}}{\sigma_{lnAF|x}}\right)$$

Where $\mu_{lnAF|x}$ is the mean value of $ln\ AF$ given $Sa_{ROCK} = x$, and $\sigma_{lnAF|x}$ is the standard deviation of $ln\ AF$ given $Sa_{ROCK} = x$. The term for $\hat{\varphi}(*)$ is simply the standard Gaussian cumulative distribution function. The parameters $\mu_{lnAF|x}$ and $\sigma_{lnAF|x}$ are obtained from the distribution for AF derived from the site response analyses described above, and are a function of bedrock amplitude x.

The site amplification functions are to be developed as described in Sections B-1 through B-5. As discussed in those sections, multiple models

of site amplification functions are derived. To compute site-specific hazard results using the equations above, these multiple models are to be combined, with associated weights (See Figure B-1 and Table B-1), to derive overall log-mean and log-standard deviation values for each spectral frequency. For each spectral frequency and input rock amplitude, the total log-mean, μ_T ($\mu_{\ln AF|x}$ in the equation above), and log-standard deviation, σ_T ($\sigma_{\ln AF|x}$ in the equation above), are calculated as:

$$\mu_T = \sum_i w_i \mu_i$$

$$\sigma_T = \sqrt{\sum_i w_i ((\mu_i - \mu_T)^2 + \sigma_i^2)}$$

Where i indicates individual site amplification models, w_i is the weight on each model, and μ_i and σ_i are the log-mean and log-standard deviation of each site amplification model, i.

B-7.0 Hazard-Consistent, Strain-Compatible Material Properties

In the current approach to develop site-specific design motions (UHRS, DRS), a probabilistic method was used which correctly preserves the annual exceedance probabilities of the generic hard rock PSHA, while properly incorporating variabilities (aleatory and epistemic) in site-specific dynamic material properties. For structural analyses, strain-compatible material properties are desired which are consistent with the probabilistically-based design motions. To achieve hazard consistency, strain-compatible properties, must reflect both the hazard level (ground motion and exceedence frequency) as well as the aleatory and epistemic components in site-specific dynamic material properties incorporated in developing the design motions.

Simply using control motions based on a generic rock site hazard to drive the site-specific soil column (Approaches 1 or 2; McGuire et al., 2001; RG 1.165, NRC, 1997) will, in general, not result in strain-compatible properties consistent with the site-specific soil hazard developed using a fully probabilistic approach. Additionally, this approach is not viable for cases where the generic reference site hazard was developed for soil or soft rock site conditions. For such cases, the rock control motions which are appropriate for the base of the soil conditions are not generally available.

As a result an approach has been developed that is intended to result in strain compatible properties that are consistent with the fully probabilistic site-specific hazard. The approach assumes strain-compatible properties are approximately lognormally-distributed, consistent with observed strong ground motion parameters (Abrahamson and Shedlock, 1997), and makes use of the distributions of strain-compatible properties catalogued during development of the suites of

amplification factors. Specifically, the approach examines the site-specific horizontal or vertical hazard curves at the AEF of interest, determines the ground motion (interpolating logarithmically as necessary), and locates the corresponding amplification factors and associated strain-compatible properties at the ground motion levels determined from the hazard curve. For each case of epistemic uncertainty, median and sigma estimates (over aleatory variability) are interpolated (logarithmically) to the appropriate ground motion level as specified by the site-specific hazard curve at the desired annual exceedance probability. To accommodate epistemic variability in site-specific properties, the same weights used in developing the site-specific hazard curves are applied to the corresponding strain compatible properties. The weighted median (mean log) set of strain compatible properties (for each layer) and log standard deviations, reflecting upper- and lower-range properties, are given by the last two equations in Section B-6.0. The associated variance includes both the aleatory component for each epistemic case as well as the variability of mean properties for each base case.

The approach accommodates both the median estimates as well as aleatory and epistemic variabilities in strain-compatible properties that are consistent with the site-specific horizontal and vertical hazard used for design. The approach results in a suite of hazard-consistent strain-compatible properties. To examine consistency in strain-compatible properties across structural frequency, as the magnitude contributions can vary, the entire process is performed at PGA (typically 100 Hz), and again at low frequency, typically 1 Hz. Since amplification factors are typically developed for a range in magnitude reflecting contributions at low (< 2 Hz) and high (> 2 Hz) frequencies, the consistency check at PGA and 1 Hz covers the typical range in magnitude contribution to the control motions. If the differences in properties at high- and low- frequency is less than 10%, the high-frequency properties are used since this frequency range typically has the greatest impact on soil nonlinearity. If the difference exceeds 10% the HCSCP developed at PGA and those developed at 1 Hz are combined with equal weights.

B-8.0 References

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Tables

Table B-1.
Site Independent Relative Weights and Epistemic Uncertainty

Parameter	Relative Weight	σ_μ
Mean Base-Case Profile	0.40	0.35
Lower-Range	0.30	
Upper-Range	0.30	
Mean Base-Case Kappa	0.40	0.40
Lower-Range	0.30	
Upper-Range	0.30	
G/Gmax and Hysteretic Damping Curves		0.15*, 0.30**
Soil		
EPRI Cohesionless Soil	0.5	
Peninsular Range	0.5	
Firm Rock		
EPRI Rock	0.5	
Linear	0.5	

* Modulus variability at cyclic shear strain 3 x 10^{-2}%

** Shear-wave damping variability at cyclic shear strain 3 x 10^{-2}%

Table B-2.
Model to Estimate Density from Shear-Wave Velocity

Shear-Wave Velocity (m/s)	Density (g/cm^3)
<500	1.84
500 to 700	1.92
700 to 1,500	2.10
1,500 to 2,500	2.20
>2,500	2.52

Table B-3.
Maximum Number of Models to Characterize Epistemic Uncertainty

Parameter	Maximum	Soil	Firm Rock	Soil/Firm Rock
	N	N	N	N
Profile	3	3	3	3
Curves	2	2	2	2
Kappa	3	1	3	3
Magnitude	2	1	1	1
1,2-Corner	2	2	1	2
Total Models	**72**	**12**	**18**	**36**

Table B-4
Suite of Hard Rock Peak Accelerations, Source Epicentral Distances, and Depths (M 6.5; 1-corner source model)

Expected Peak Acceleration (%g)	Distance (km)	Depth (km)
1	230.00	8.0
5	74.00	8.0
10	45.00	8.0
20	26.65	8.0
30	18.61	8.0
40	13.83	8.0
50	10.45	8.0
75	4.59	8.0
100	0.0	7.0
125	0.0	5.6
150	0.0	4.7

Additional parameters used in the point-source model are:

$\Delta\sigma$ (1-corner) = 110 bars
ρ = 2.71 cgs
β = 3.52 km/s
R_C = 60 km, crossover hypocentral distance to $R^{-0.5}$ geometrical attenuation
T = $1/f_c + 0.05$ R, RVT duration, R = hypocentral distance (km)
Q_o = 670
η = 0.33
kappa(s) = 0.006

Table B-5
Generic CEUS Hard Rock Crustal Model

Thickness (km)	V_s (km/sec)	ρ (cgs)
1	2.83	2.52
11	3.52	2.71
28	3.75	2.78
--	4.62	3.35

Table B-6
Suite of Hard Rock Peak Accelerations, Source Epicentral Distances, and Depths (M 6.5; 2-corner source model)

Expected Peak Acceleration (%g)	Distance (km)	Depth (km)
1	230.00	8.0
5	81.00	8.0
10	48.00	8.0
20	28.67	8.0
30	20.50	8.0
40	15.60	8.0
50	12.10	8.0
75	6.30	8.0
100	0.0	7.9
125	0.0	6.4
150	0.0	5.4

Table B-7
Geometrical spreading and attenuation models for the CEUS and WUS

Region	Geometric Spreading			Anelastic Attenuation
CEUS	1/R	for	R ≤ 60km	$Q(f) = 670f^{0.33}$
	$(1/60)(60/R)^{0.5}$	for	R > 60km	
WUS	1/R	for	R ≤ 35km	$Q(f) = 270f^{0.60}$
	$(1/35)(35/R)^{0.5}$	for	R > 35km	

Figures

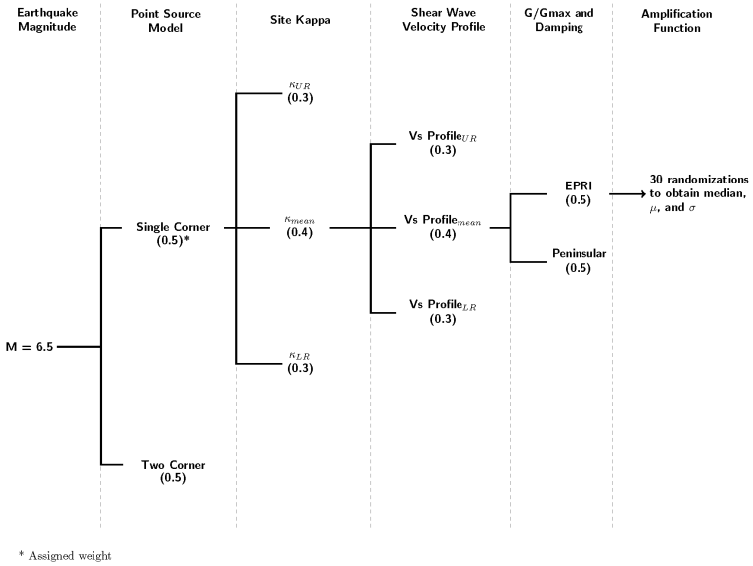


Figure B-1
Logic tree illustrating the process for capturing uncertainty in the development of site-specific amplification functions.

This illustration is for a site with limited at-site geophysical and geotechnical data available. UR and LR indicate Upper-Range and Lower-Range about the mean Base-Case model.

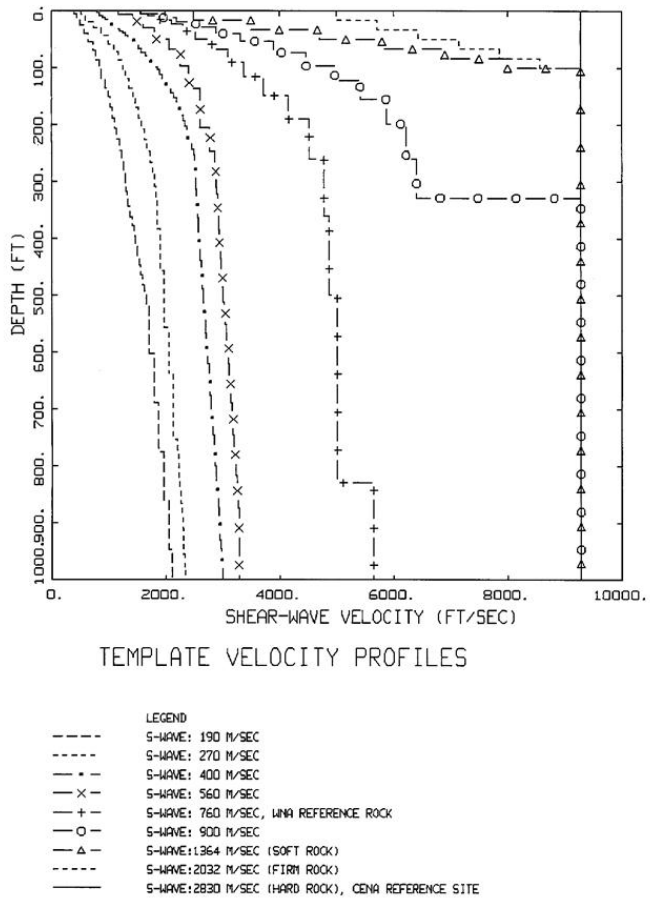


Figure B-2
Template Shear Wave Velocity Profiles for Soils, Soft Rock, and Firm Rock. Rock Profiles Include Shallow Weathered Zone. Indicated velocities are for V_{S30}.

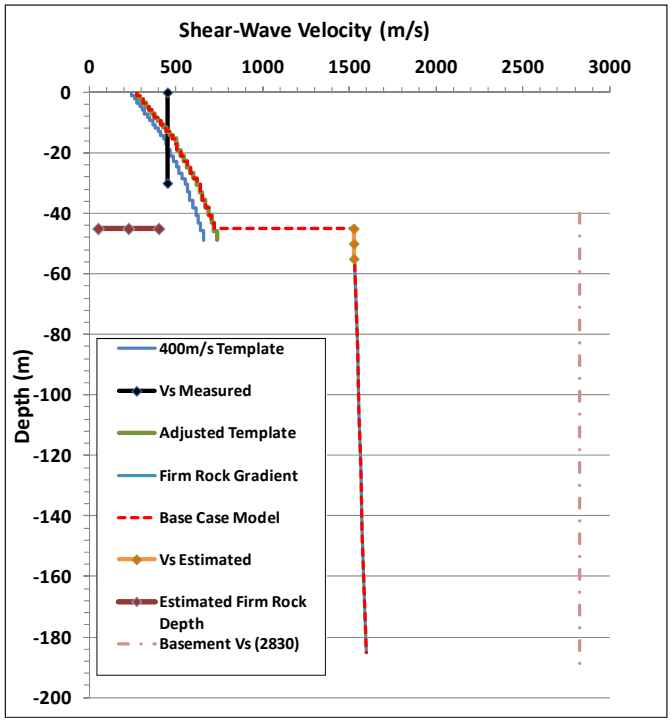


Figure B-3
Illustration of how available information is used to develop a mean base-case profile.

The information available is represented by the measured near-surface soil VS30 (solid black line), estimated depth to firm rock (solid brown line) and estimated firm rock VS (solid orange line). Proposed mean base-case VS profile is indicated by dashed red line.

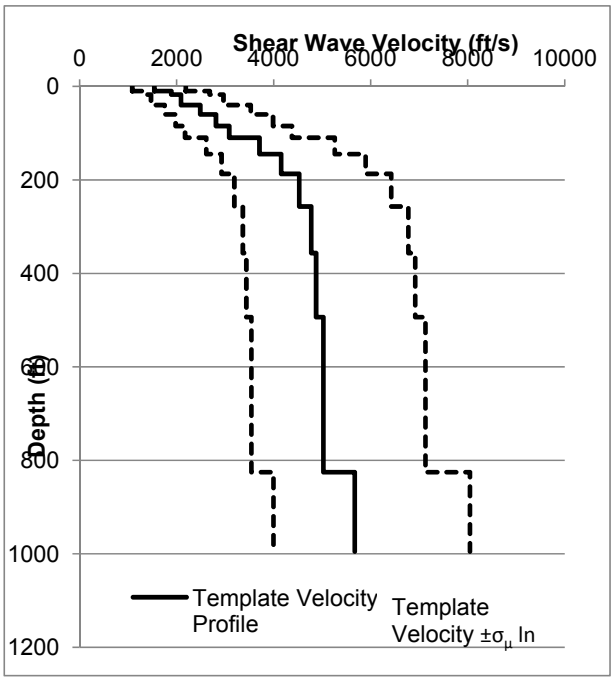


Figure B-4
Illustration of the range of velocity

This shows a case implied by the method used to account for epistemic uncertainty in site specific shear wave velocity profiling where sparse or limited information is available. Displayed is the 760 m/s WNA reference rock template velocity (solid curve) with dashed curves representing ± $\sigma_{\mu\ \ln}$ = 0.35.

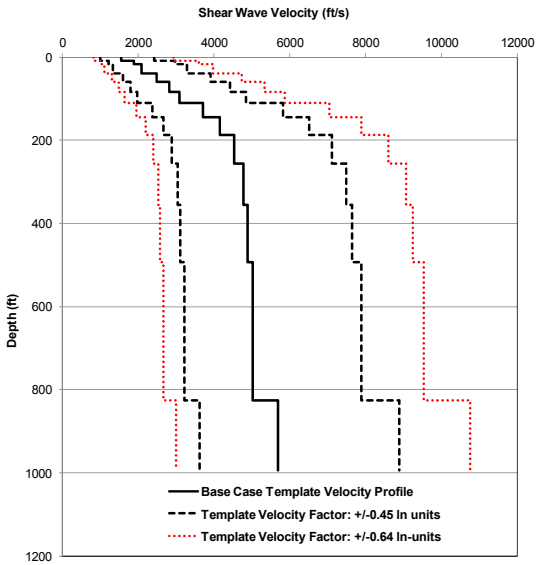


Figure B-5
Method used to account for epistemic uncertainty in site specific shear wave velocity profiling where very limited or no information is available.

Displayed is the 760m/s reference template velocity (solid black curve) with dashed curves representing 10th and 90th-percentile values (±0.45 natural log units which corresponds to a σ_{μ} ln = 0.35). Dotted red curves are for ±0.64 natural log units which corresponds to a σ_{μ} ln = 0.5.

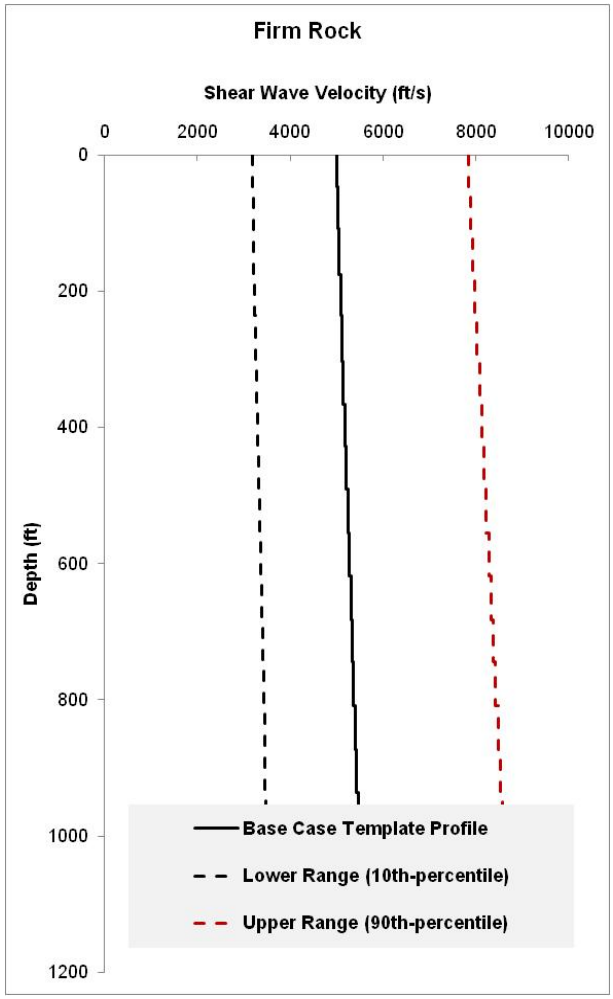


Figure B-6
Illustration of Upper Range and Lower Range Base-Case profiles (10th and 90th percentiles) developed to represent the epistemic uncertainty in the Mean Base-Case for firm rock conditions.

A mean surface velocity of 5000ft/s (1525m/s) was assumed for the Base Case and the empirical gradient of Fukishima et al. (1995) [19] was applied. A $\sigma_{\ln}$ = 0.35 was used.

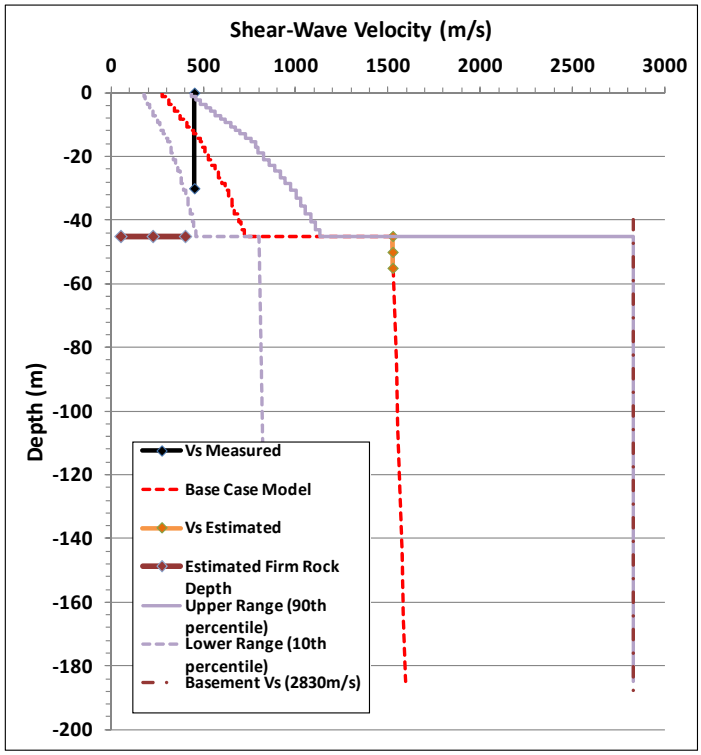


Figure B-7
Illustration of the development of Upper Range and Lower Range profiles to accommodate epistemic uncertainties for the hypothetical example shown in Figure B-3.

A $\sigma_{\mu\ \ln}$ = 0.35 has been used to develop the 10th and 90th-percentile curves in the upper portion of the profile where sparse Vs measurements were available. A $\sigma_{\mu\ \ln}$ = 0.50 was applied to the lower portion of the profile where the Vs of the Base Case was inferred from geological information. The 90th-percentile curve was capped at a value equal to the 2830m/s Vs value assumed for the hard rock basement.

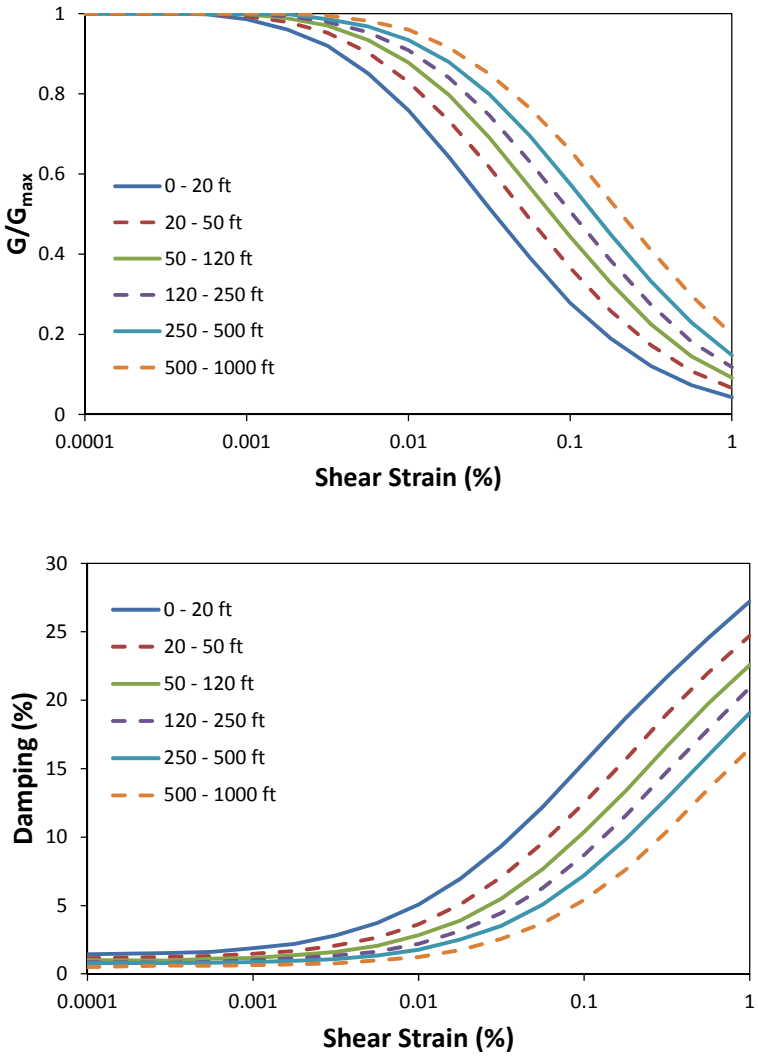


Figure B-8
Generic G/GMAX and hysteretic damping curves for cohesionless soil [12]. Note that damping will be limited to 15% for this application.

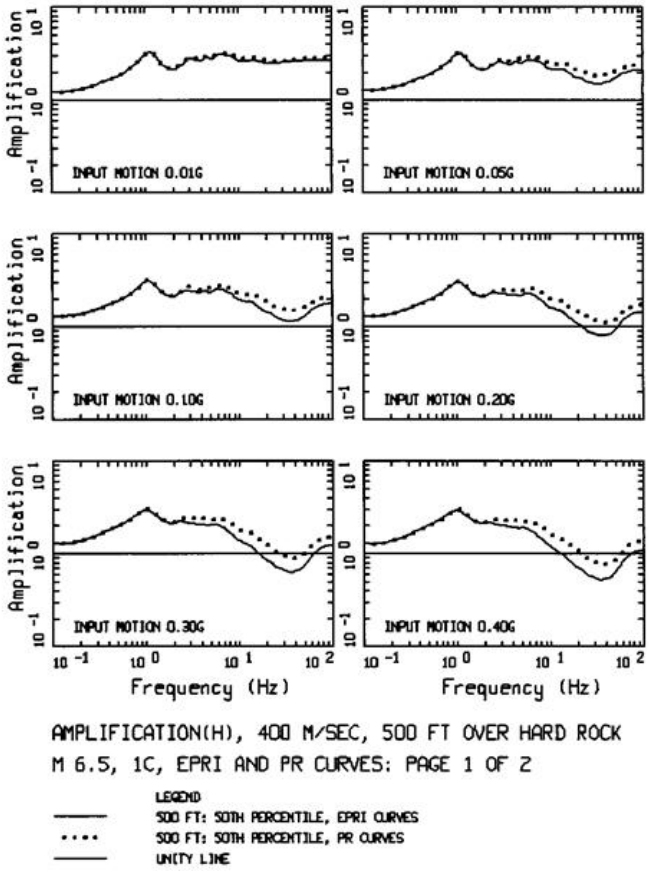


Figure B-8a.
Comparison of median amplification functions (5%-damped PSa) derived using the EPRI (1993) [12] (see Figure B-7) and Peninsular Range [28]) G/GMAX and hysteretic damping curves.

The results are for the 400 m/sec VS30 template profile and a single-corner source model for reference rock loading levels of 0.01 to 1.50g.

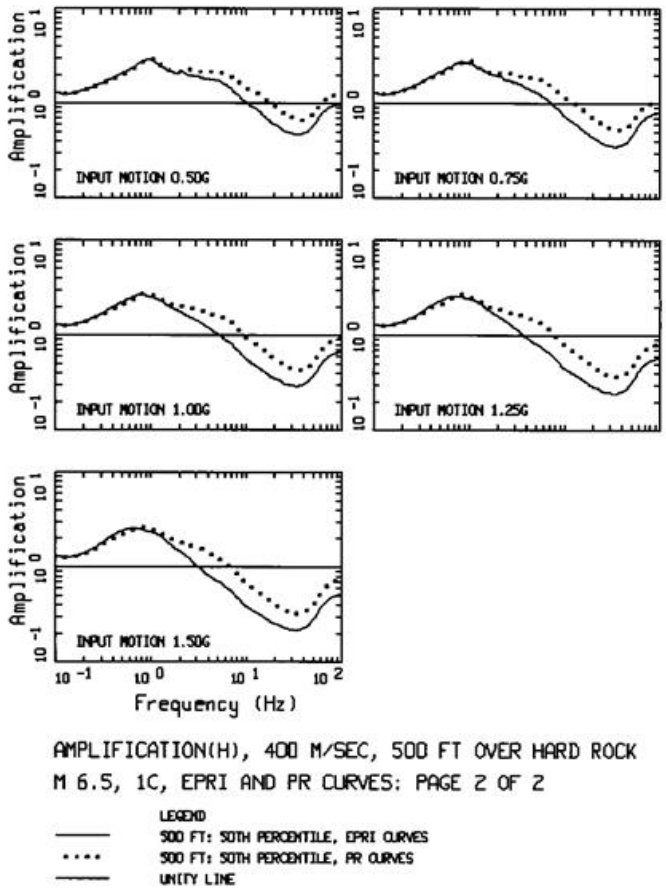


Figure B-8b.
Comparison of median amplification functions (5%-damped PSa) derived using the EPRI (1993) [12] (see Figure B-7) and Peninsular Range [28] G/GMAX and hysteretic damping curves.

The results are for the 400 m/sec VS30 template profile and a single-corner source model for reference rock loading levels of 0.01 to 1.50g.

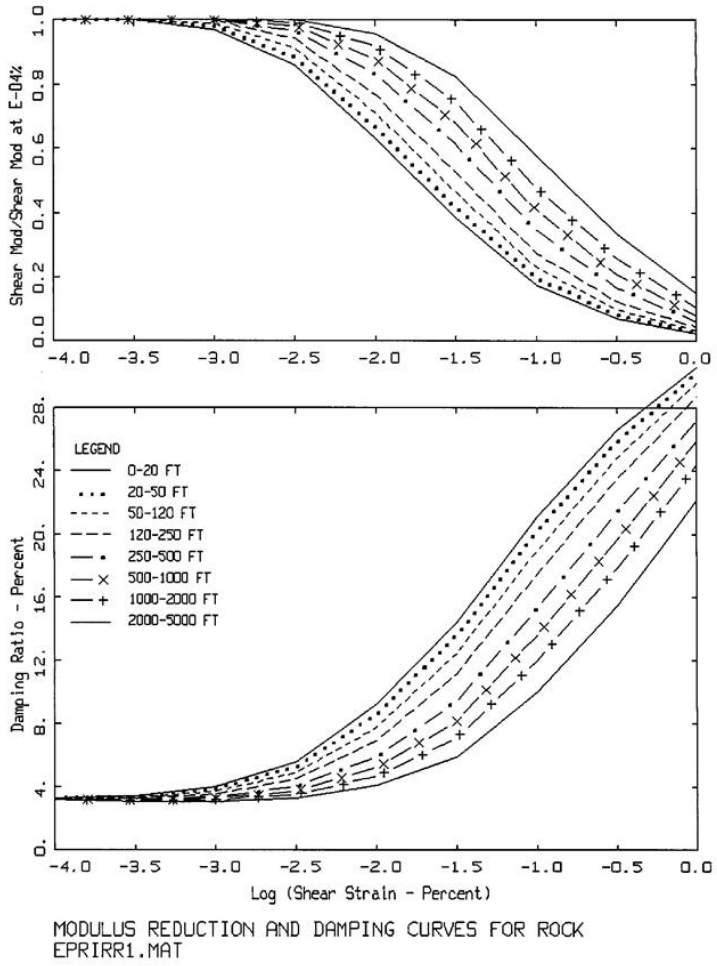


Figure B-9.
Generic G/GMAX and hysteretic damping curves developed for firm rock in the EPRI (1993) study [12] (from Dr. Robert Pyke).

Note that damping is limited to 15% in this application.

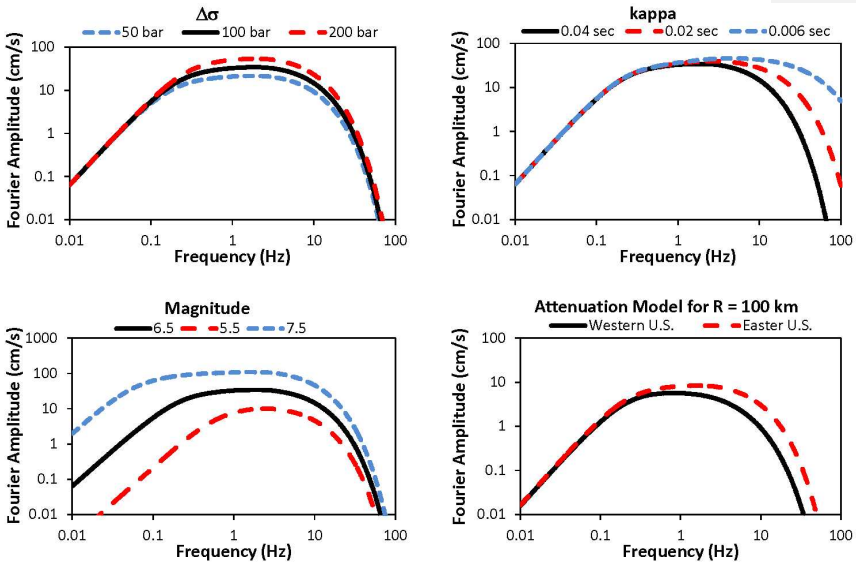


Figure B-10.
Illustration of effect of various factors in the simple seismological model on Fourier spectral shape.

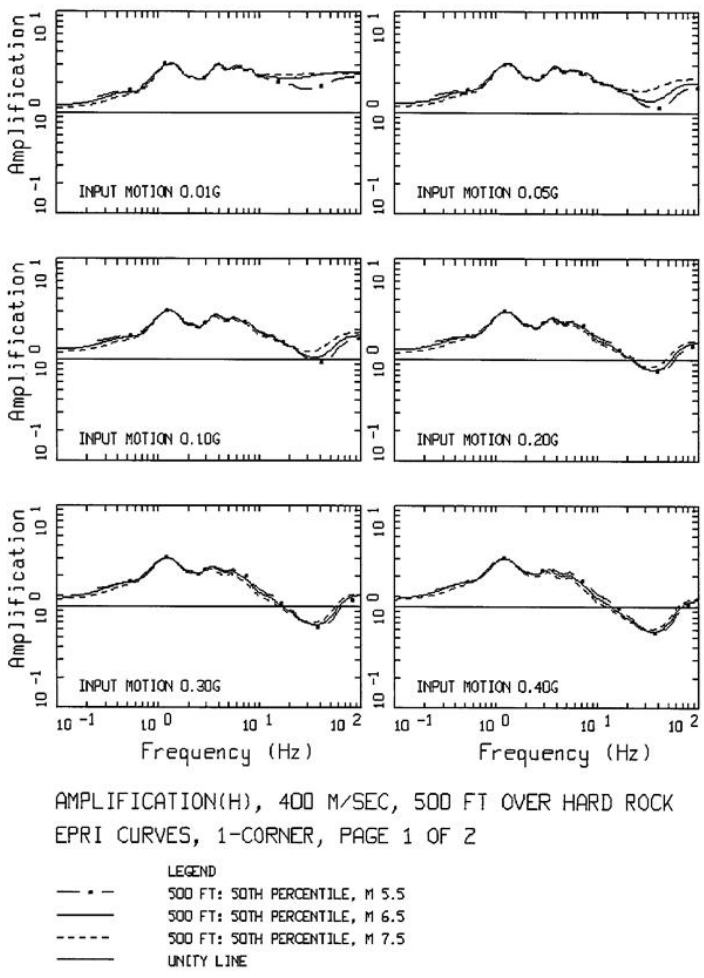


Figure B-11a.
Comparison of amplification functions (5%-damped PSa) computed for magnitudes of M 5.5, 6.5, and 7.5, using the single-corner source model and the 400 m/sec VS30 stiff-soil template profile (Figure B-2) with the EPRI (1993) [12] G/GMAX and hysteretic damping curves (Figure B-7).

The input reference rock loading levels varied from 0.01 to 1.50 g (Table B-4).

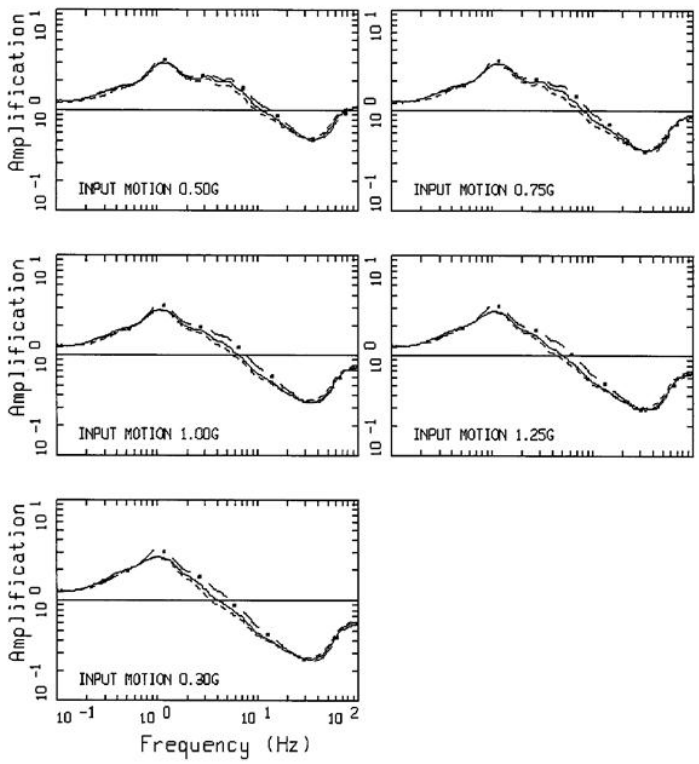


Figure B-11b.
Comparison of amplification functions (5%-damped PSa) computed for magnitudes of M 5.5, 6.5, and 7.5.

This figure uses the single-corner source model and the 400 m/sec VS30 stiff-soil template profile (Figure B-2) with the EPRI (1993) [12] G/GMAX and hysteretic damping curves (Figure B-7). The input reference rock loading levels varied from 0.01 to 1.50 g (Table B-4).

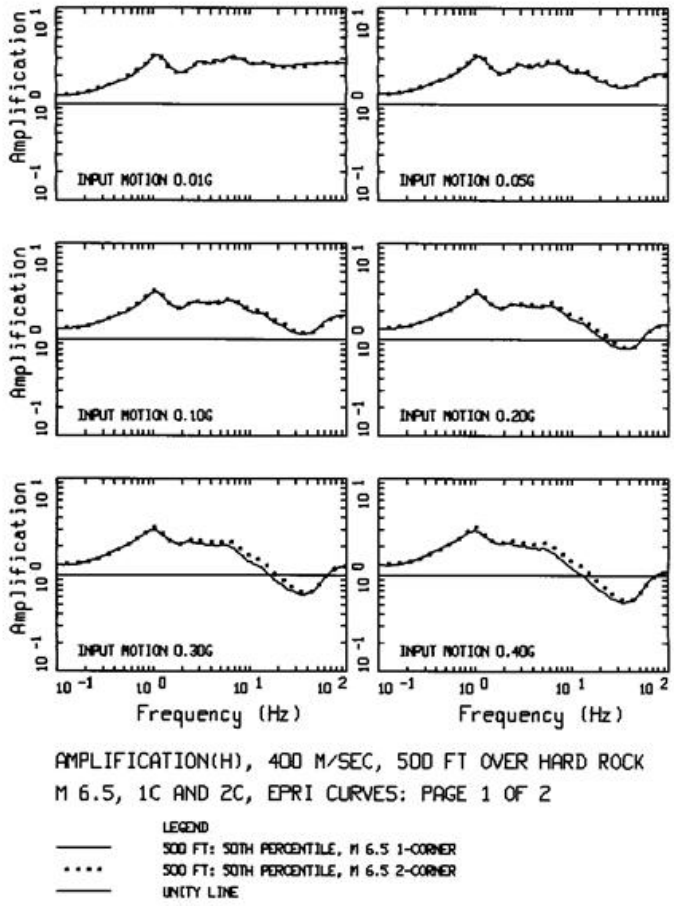


Figure B-12a.
Comparison of amplification functions (5% damped PSa) computed using the Single- and Double-Corner source models (Tables B-4 and B-6) for the 400 m/sec VS30 stiff-soil template profile (Figure B-2) with the EPRI (1993) [12] G/GMAX and hysteretic damping curves (Figure B-7).

The input reference rock loading levels varied from 0.01 to 1.50 g (Tables B-4 and B-6).

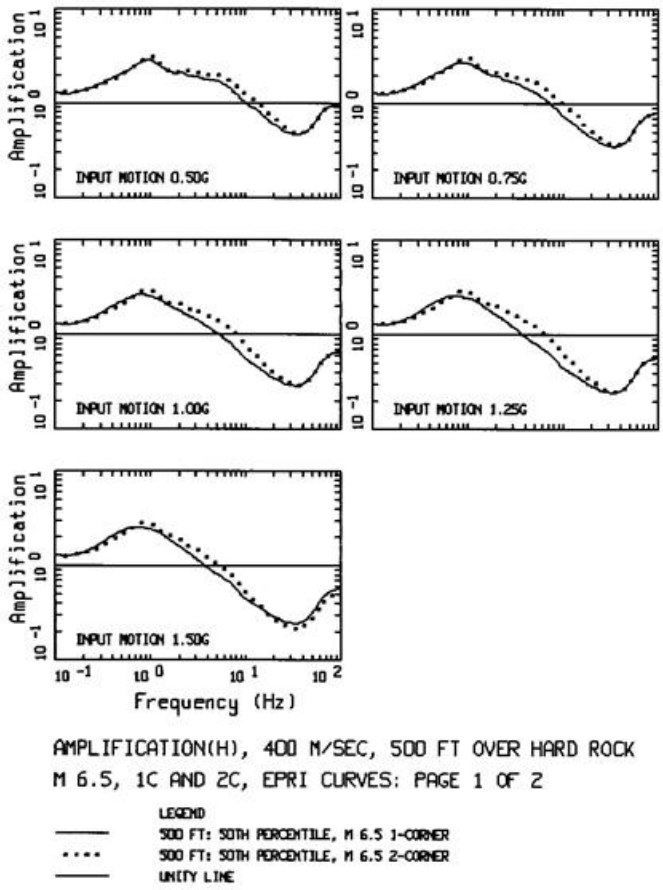


Figure B-12b.
Comparison of amplification functions (5% damped PSa) computed using the Single- and Double-Corner source models (Tables B-4 and B-6) for the 400 m/sec VS30 stiff-soil template profile (Figure B-2) with the EPRI (1993) [12] G/GMAX and hysteretic damping curves (Figure B-7).

The input reference rock loading levels varied from 0.01 to 1.50 g (Tables B-4 and B-6).

Appendix C: Sensitivity Studies to Develop Criteria for Analyzing Rock-Founded Structures as Fixed-Base Models

Two examples of models of existing structures at a nuclear power plant were analyzed in a study to compare the ISRS developed from a fixed-base dynamic analysis with the ISRS obtained from SSI analyses with various shear wave velocities. The first example analyzed a containment structure with a fundamental frequency of about 5 Hz, and the second example was for a Main Steam Valve House (MSVH) structure which has a fundamental frequency of about 10 Hz in one horizontal direction.

C1.0 Containment Structure

The containment structure, which has a horizontal fundamental mode of about 5 Hz, was analyzed [45] using a fixed-base model and, subsequently, with SSI analyses using three sets of shear wave velocities (Vs) for this site: lower bound (about 3,400 ft/sec), best estimate (about 5,200 ft/sec) and an upper bound (about 7,900 ft/sec). Figures C-1, C-2 and C-3 show the results of the analyses at the operating deck of the structure (about 75 ft above the basemat) in the east-west, north-south, and vertical directions, respectively. All three figures compare the results from SSI analyses using the lower and best estimate shear wave velocity values with the results of the fixed-base analysis. The results from the SSI analysis using the upper bound Vs were very close to the fixed-base results and therefore are not shown in this Appendix but are included in [45].

From these figures, it can be seen that for the east-west and north-south directions, the lower bound Vs case resulted in a slight shift of frequency of the spectral peak as compared to the fixed-base model (about 1 Hz lower with SSI) because of the rotational effects. For the best estimate

case (Vs of 5200 ft/sec), the frequency shift was smaller in the north-south direction, and there was no shift in the east-west direction. In the vertical direction, the two SSI cases gave identical ISRS, and there was also no frequency shift compared to the fixed-base case. In all three directions, the spectral peaks determined from the fixed-base analysis were higher than those from the SSI analyses, and the shapes of the ISRS in the entire frequency range remained about the same. Therefore, based on the analysis of this structure, a fixed-base analysis can be used for sites with rock Vs > 3,500 ft/sec only if one can accept a slight potential frequency shift of the spectral peak. If not, a fixed-base analysis is only recommended for rock Vs > 5,000 ft/sec. It is noted that peak-broadening or peak-shifting of the ISRS in fragility analyses can alleviate the effect of a slight frequency shift between the SSI and fixed-base analyses.

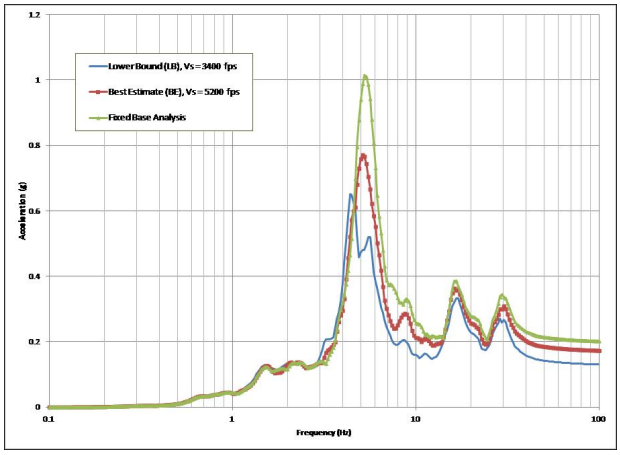


Figure C-1
Reactor Containment Internal Structure, Operating Floor
East-West Direction, 5% Damping

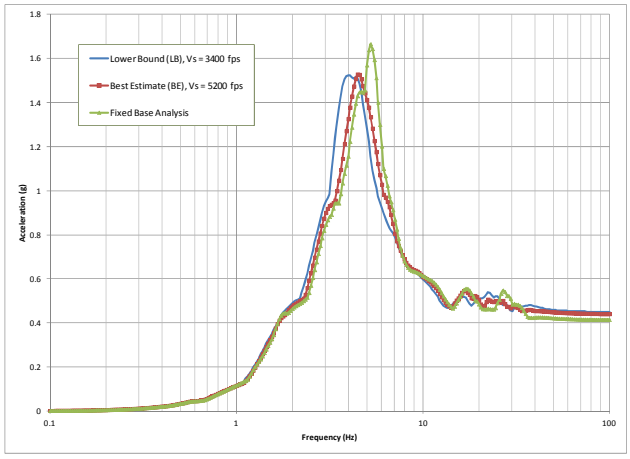


Figure C-2
Reactor Containment Internal Structure, Operating Floor
North-South Direction, 5% Damping

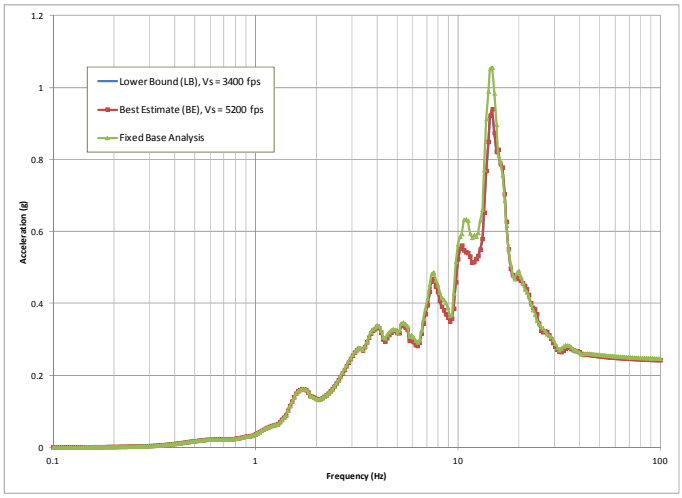


Figure C-3
Reactor Containment Internal Structure, Operating Floor
Vertical Direction, 5% Damping

C2.0 Main Steam Valve House Structure

The MSVH structure, which has a fundamental mode of about 10 Hz in one horizontal direction, was analyzed [55] using a fixed-base model and, subsequently, with SSI analyses using two sets of shear wave velocities

(Vs): 3,500 ft/sec and 5,200 ft/sec (best estimate Vs for this site). Three sets of input motions were used in the analysis; however, ISRS for only two sets of representative motions are presented in this Appendix (results of the third set of input motions are about the same as the first set and are documented in [55]). The two set of results presented here are: (a) ISRS using time histories derived from the design basis earthquake (DBE) ground spectrum shape of this plant, which is similar to that of RG 1.60, and (b) ISRS using time histories derived from GMRS from a recent PSHA for a hard rock site that contains high frequency content.

Figures C-4, C-5 and C-6 show the ISRS for case (a) and Figures C-7, C-8 and C-9 show the ISRS for case (b), in the east-west, north-south, and vertical directions, respectively. These ISRS are compared at a node (Elevation 306 ft) that is about 60 ft above the basemat of the structure. These figures compare the results from the fixed-base case and the SSI cases with (Vs) = 3,500 ft/sec case and (Vs) = 5,200 ft/sec.

From these figures, it can be seen that the comparison of fixed-base ISRS with the (Vs) = 5,200 ft/sec case is reasonably good for all directions, whereas the lower bound Vs =3500 ft/sec case results in a slight shift of frequency and the spectral peaks are also slightly different in comparison to the fixed-base model. The shapes of the ISRS in the entire frequency range remain about the same from all these analyses. Therefore, based on the analysis of this structure, a fixed-base analysis is appropriate for rock sites with Vs > about 5,000 ft/sec. A fixed-base analysis can be used for sites with rock Vs > 3,500 ft/sec only if one can accept potential small frequency and amplitude shifts of the spectral peaks. Again, peak-broadening or peak-shifting of the ISRS in fragility analyses can alleviate the effect of a slight frequency shift between the SSI and fixed-base analyses.

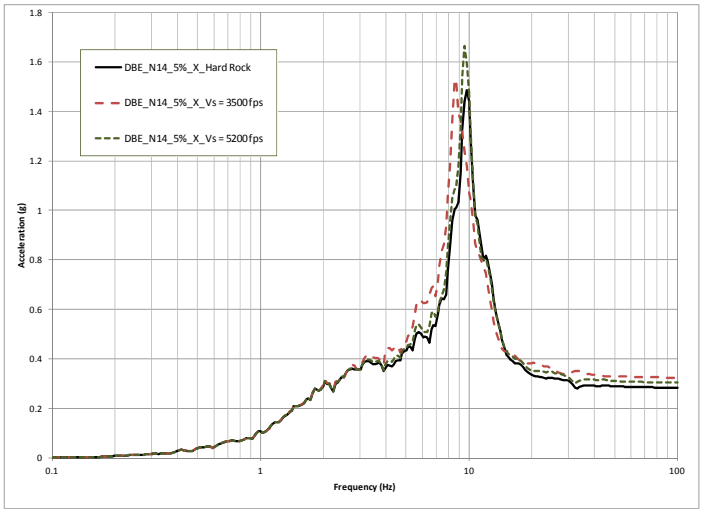


Figure C-4
Main Steam Valve House, Elevation 306 ft
East-West Direction, 5% Damping

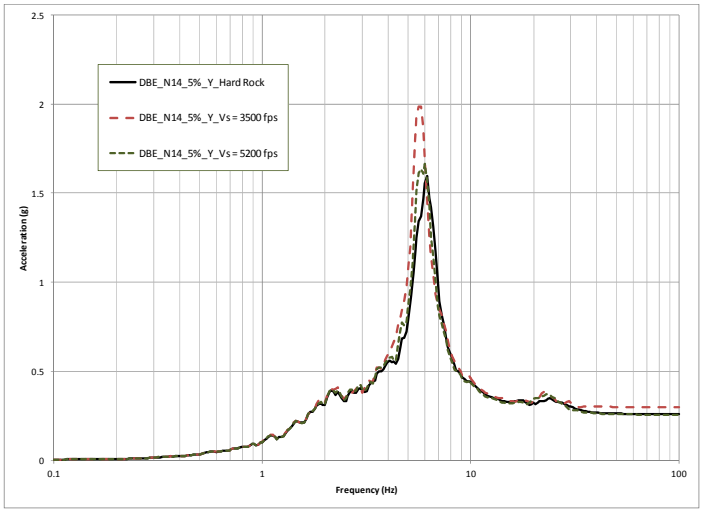


Figure C-5
Main Steam Valve House, Elevation 306 ft
North-South Direction, 5% damping

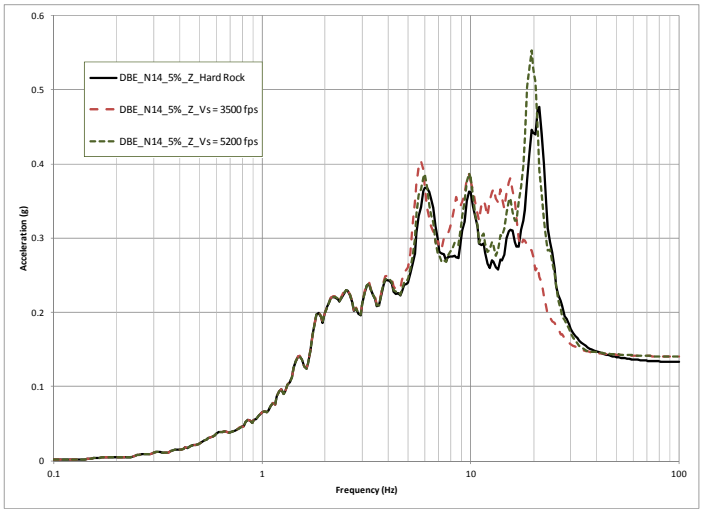


Figure C-6
Main Steam Valve House, Elevation 306 ft
Vertical Direction, 5% damping

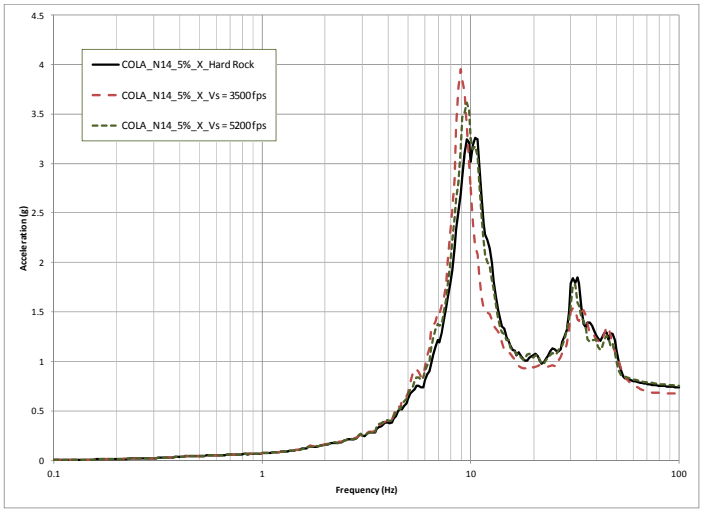


Figure C-7
Main Steam Valve House, Elevation 306 ft
East-West Direction, 5% damping

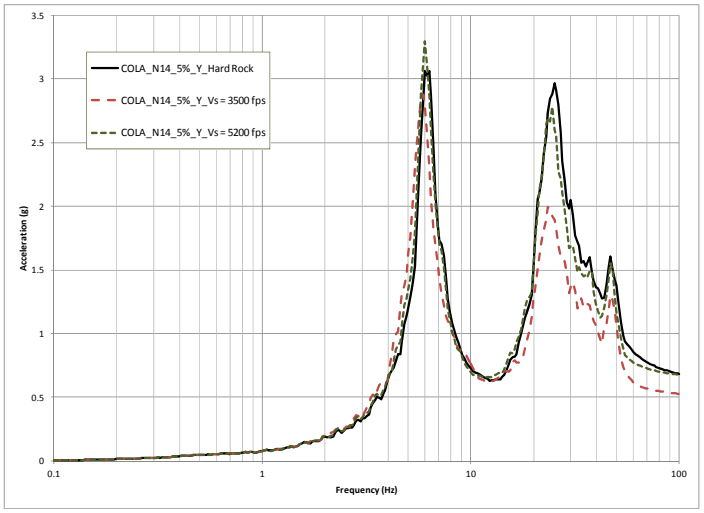


Figure C-8
Main Steam Valve House, Elevation 306 ft
North-South Direction, 5% damping

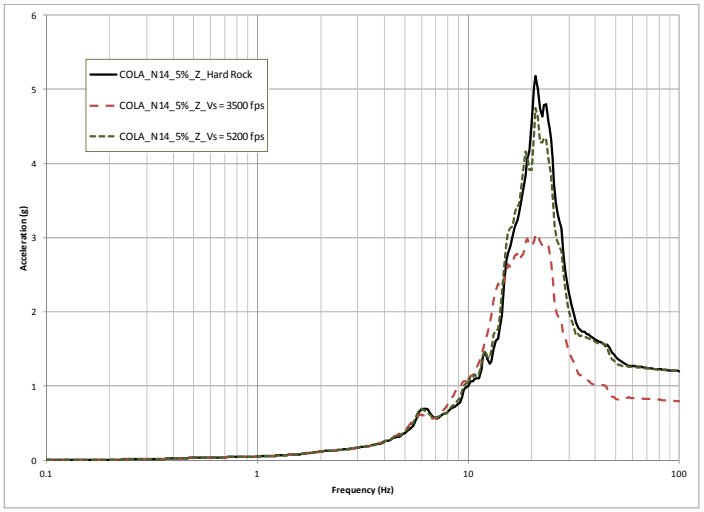


Figure C-9
Main Steam Valve House, Elevation 306 ft
Vertical Direction, 5% damping

Appendix D: Sensitivity of Computed Annual Probability of Failure P_F to Assumed Logarithmic Standard Deviation β Used in Hybrid Method with Capacities Defined by 1% Failure Probability Capacity $C_{1\%}$

D1.0 Introduction

In the Hybrid Method, the 1% failure probability capacity $C_{1\%}$ is computed by the CDFM Method. Then an estimate of the composite logarithmic standard deviation β_C and its subdivision into random variability β_R and uncertainty β_U are used to estimate the corresponding fragility curve. As noted in Ref. 1, typically β_C lies within the range of 0.3 to 0.6. In fact, if all of the sources of variability discussed in Ref. 2 are appropriately considered, it is not possible to obtain an estimated β_C less than approximately 0.3.

The Hybrid Method is based on the observation that the annual probability of unacceptable performance P_F for any Seismic Source Characterization is relatively insensitive to β_C. Thus, annual probability (seismic risk) can be computed with adequate precision from the CDFM Capacity, C_{CDFM}, and an estimate of β_C. It is concluded in Ref. 1 that the computed seismic risk at $\beta = 0.3$ is approximately 1.5 times that at $\beta = 0.4$, while at $\beta = 0.6$ the computed seismic risk is approximately 60% of that at $\beta = 0.4$.

In Section 3, it is demonstrated that the Ref. 1 conclusion concerning the lack of sensitivity of the computed seismic risk remains valid over the full

practical range of hazard curve slopes. This demonstration uses the Simplified Seismic Risk Equation defined in Section 2.

D2.0 Simplified Seismic Risk Equation

Typical seismic hazard curves are close to linear when plotted on a log-log scale (for example see Figure 1). Thus over any (at least) ten-fold difference in exceedance frequencies such hazard curves may be approximated by a power law:

$$H(a) = K_I a^{-K_H}$$ Equation 1

where H(a) is the annual frequency of exceedance of ground motion level a, K_I is an appropriate constant, and K_H is a slope parameter defined by:

$$K_H = \frac{1}{\log(A_R)}$$ Equation 2

in which A_R is the ratio of ground motions corresponding to a ten-fold reduction in exceedance frequency defined by:

$$A_R = \frac{SA_{0.1H}}{SA_H}$$ Equation 3

where SAH is the spectral acceleration at the mean exceedance frequency H and $SA_{0.1H}$ is the spectral acceleration at 0.1H.

So long as the fragility curve $P_{F(a)}$ is lognormally distributed and the hazard curve is defined by Equation 1, a rigorous closed-form solution exists for the seismic risk. This closed-form solution has been derived in a number of references including Appendix C of Ref. 1 and Appendix A of Ref. 3:

$$P_F = H\ F_{50\%}^{-K_H}\ e^{\alpha}$$ Equation 4

in which

$$F_{50\%} = \frac{C_{50\%}}{C_H}$$ Equation 5

and

$$\alpha = \tfrac{1}{2}(K_H \beta)^2$$ Equation 6

where H is any reference exceedance frequency, C_H is the UHRS ground motion level that corresponds to this reference exceedance frequency H

from the seismic hazard curve, $C_{50\%}$ is the median fragility capacity, and β is the logarithmic standard deviation of the fragility.

Equation 4 is referred to herein as the Simplified Seismic Risk Equation. The only approximations in its derivation are that the hazard curve is approximated by Equation 1 over the exceedance frequency range of interest and the fragility curve is lognormally distributed.

By defining C_H at the 1% failure probability capacity C1%, then H is replaced in Equation 4 by $H_{1\%}$ corresponding to the annual frequency of exceeding $C_{1\%}$. With these definitions for C_H and H, Equations 4 through 6 can be rewritten as:

$$F_{50\%} = (C_{50\%}/C_{1\%}) = e^{2.326\beta} \qquad \text{Equation 7}$$

$$(P_F/H_{1\%}) = e^{\alpha_1} \qquad \text{Equation 8}$$

$$\alpha 1 = 0.5\,(KH\beta)^2 - 2.326\,(K_H\beta) \qquad \text{Equation 9}$$

From Equations 8 and 9, it can be observed that the ratio $(P_F/H_{1\%})$ is only a function of the product $(K_H\beta)$.

D3.0 Sensitivity of Failure Probability P_F to β

Table D-1 presents the ratio $(P_H/H_{1\%})$ computed over the full practical range of ground motion ratios A_R from 1.5 to 4.5 and β values ranging from 0.3 to 0.6 using the Simplified Seismic Risk Equation.

As shown in Table 4 of Reference 1, the failure probability P_F values computed by the Simplified Seismic Risk Equation tend to be about 10% to 20% conservatively biased when compared to P_F computed by numerical convolution of the hazard curve and fragility curve. This bias is due to the slight downward curvature of the hazard curve when plotted on a log-log plot as is shown in Figure D-1. Even with this slight conservative bias, the Simplified Seismic Risk Equation can be used to compare P_F values computed for different β values at any specified A_R ratio

AR values ranging from 2.0 to 4.5 are typical for CEUS sites over the 10^{-4} to 10^{-6} annual frequency of exceedance range. Within this A_R range, the $(P_F/H_{1\%})$ ratios shown in Table D-1 for β = 0.3 range from 1.32 to 1.44 of the $(P_F/H_{1\%})$ ratios shown for β = 0.4 with an average ratio of about 1.4. Many numerical convolutions of hazard and fragility curves have confirmed the conclusion that P_F computed using β = 0.3 will be less than about 1.5 times the P_F computed using β = 0.4 for the same $C_{1\%}$ capacity.

Over this same A_R range from 2.0 to 4.5, the $(P_F/H_{1\%})$ ratios shown in Table D-1 for β = 0.6 range from 58% to 64% of the $(P_F/H_{1\%})$ ratios computed for β = 0.4 with an average of about 60%. Again, many

numerical convolutions of hazard and fragility curves have confirmed this value of about 60%.

For high seismic western sites, the A_R values will typically range between 1.5 and 2.25 over the 10^{-4} to 10^{-6} annual frequency range. For A_R values less than about 1.8, the lower tail of the lognormal fragility curve below the 1% failure probability capacity $C_{1\%}$ begins to significantly affect computed P_F. Many experienced seismic capacity engineers question the validity of extending the lower tail of the lognormal fragility curve substantially below the $C_{1\%}$ capacity. However, if one conservatively includes this lower tail on the lognormal fragility curve, the resulting $(P_F/H_{1\%})$ ratios computed by numerical convolution of hazard and fragility are similar to the results shown in Table D-1. As A_R is reduced below 1.75, the ratio of $(P_F/H_{1\%})$ for $\beta = 0.3$ to $(P_H/H_{1\%})$ for $\beta = 0.4$ will begin to rapidly reduce below 1.4. Conversely, as A_R is reduced below 2.0, the ratio of $(P_H/H_{1\%})$ for $\beta = 0.6$ to $(P_H/H_{1\%})$ for $\beta = 0.4$ will begin to rapidly increase. In fact at $A_R = 1.5$, the $(P_H/H_{1\%})$ ratio for $\beta = 0.6$ will be 1.8 times that for $\beta = 0.4$. This unexpected result is directly attributable to the tail of the lognormal fragility curve below the $C_{1\%}$ capacity. For sites with A_R less than 1.75:

1) one should assess whether it is appropriate to extend the lognormal fragility curve below the $C_{1\%}$ capacity, and

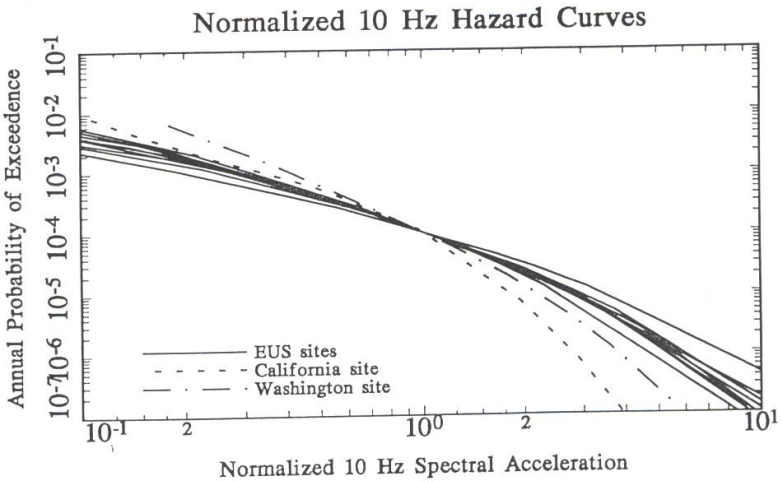
2) if the fragility curve is extended below $C_{1\%}$, one needs to carefully estimate β for those situation where β might exceed 0.5 (i.e., active components mounted at high elevation in structures).

D4.0 References

1. Kennedy, R.P. Overview of Methods for Seismic PRA and Margin Analysis Including Recent Innovations, Proceedings of the OECD-NEA Workshop on Seismic Risk, Tokyo, August 1999.
2. Methodology for Developing Seismic Fragilities, EPRI TR-103959, Electric Power Research Institute, June 1994.
3. Kennedy, R.P., Performance-Goal Based (Risk Informed) Approach for Establishing the SSE Site Specific Response Spectrum for Nuclear Power Plants. Nuclear Engineering and Design 241, pp. 648-656, 2011 (also Transactions SMiRT 19 Toronto, August, 2007).
4. Risk Engineering, Inc., REI (2001), Technical Basis for Revision of Regulatory Guidance on Design Ground Motions: Hazard- and Risk-consistent Ground Motion Spectra Guidelines, US Nuclear Regulatory Commission Report NUREG/CR-6728, October, 2001.

Table D-1
Sensitivity of (PF/H1%) to Estimated β over Range of AR Values

A_R	K_H	$(P_F/H_{1\%})$			
		β=0.3	β=0.4	β=0.5	β=0.6
1.5	5.68	0.081	0.067	0.076	0.120
1.75	4.11	0.121	0.084	0.069	0.068
2.0	3.32	0.162	0.110	0.083	0.070
2.5	2.51	0.230	0.160	0.118	0.093
3.0	2.10	0.282	0.202	0.151	0.118
3.5	1.84	0.323	0.237	0.180	0.141
4.0	1.66	0.355	0.266	0.205	0.162
4.5	1.53	0.382	0.290	0.226	0.180



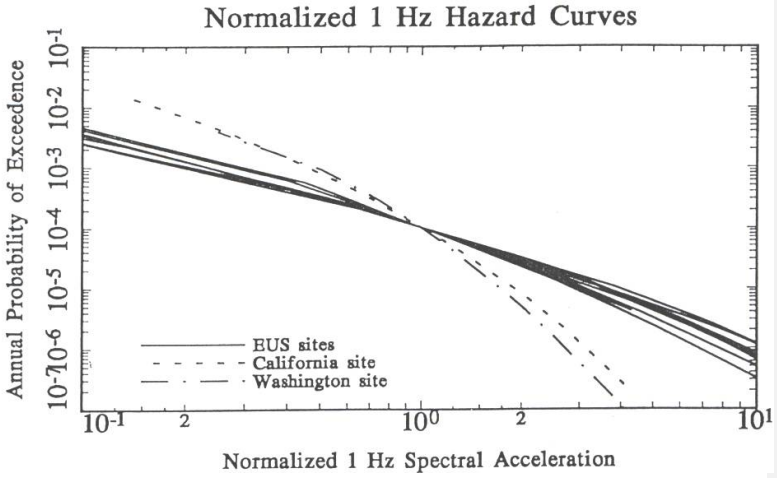


Figure D-2
SA (10 Hz) and SA (1 Hz) hazard curves for the eleven sites normalized by the acceleration value corresponding to mean 10-4 annual probability.

(From Figures. 7.7 and 7.8 of REI, 2001)