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U.S. Nuclear Regulatory Commission
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Braidwood Station, Unit 2
Facility Operating License No. NPF-77
NRC Docket No. STN 50-457

Subject: Core Operating Limits Report, Braidwood Unit 2 Cycle 17

The purpose of this letter is to transmit the Core Operating Limits Report (COLR) for Braidwood Unit 2 Cycle 17, in accordance with Technical Specification 5.6.5, "Core Operating Limits Report (COLR)." Braidwood Unit 2 Cycle 17 COLR, Revision 6 was implemented during Braidwood Unit 2 Refueling Outage 16 in support of Cycle 17 operation. Note that Revision 6 is the original version of the Braidwood Unit 2 Cycle 17 COLR implemented in support of Cycle 17 operation. The revision number is based on a numbering convention that continues from the previous revision of the Unit 2 COLR, i.e., Braidwood Unit 2 Cycle 16 COLR, Revision 5.

If you have any questions regarding this matter, please contact Chris VanDenburgh, Regulatory Assurance Manager, at (815) 417-2800.

Sincerely,



Daniel J. Enright
Site Vice President
Braidwood Station

Attachment: Core Operating Limits Report (COLR) for Braidwood Unit 2 Cycle 17, Revision 6

cc: NRC Regional Administrator, Region III
NRC Senior Resident Inspector – Braidwood Station
NRC Project Manager, NRR
Illinois Emergency Management Agency – Division of Nuclear Safety

CORE OPERATING LIMITS REPORT (COLR)

FOR

BRAIDWOOD UNIT 2 CYCLE 17

EXELON TRACKING ID:

COLR BRAIDWOOD 2 REVISION 6

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 17

1.0 CORE OPERATING LIMITS REPORT

This Core Operating Limits Report (COLR) for Braidwood Station Unit 2 Cycle 17 has been prepared in accordance with the requirements of Technical Specification 5.6.5 (ITS).

The Technical Specifications affected by this report are listed below:

- SL 2.1.1 Reactor Core Safety Limits (SLs)
- LCO 3.1.1 SHUTDOWN MARGIN (SDM)
- LCO 3.1.3 Moderator Temperature Coefficient (MTC)
- LCO 3.1.4 Rod Group Alignment Limits
- LCO 3.1.5 Shutdown Bank Insertion Limits
- LCO 3.1.6 Control Bank Insertion Limits
- LCO 3.1.8 PHYSICS TESTS Exceptions – MODE 2
- LCO 3.2.1 Heat Flux Hot Channel Factor ($F_Q(Z)$)
- LCO 3.2.2 Nuclear Enthalpy Rise Hot Channel Factor ($F_{\Delta H}^N$)
- LCO 3.2.3 AXIAL FLUX DIFFERENCE (AFD)
- LCO 3.2.5 Departure from Nucleate Boiling Ratio (DNBR)
- LCO 3.3.1 Reactor Trip System (RTS) Instrumentation
- LCO 3.3.9 Boron Dilution Protection System (BDPS)
- LCO 3.4.1 Reactor Coolant System (RCS) Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits
- LCO 3.9.1 Boron Concentration

The portions of the Technical Requirements Manual affected by this report are listed below:

- TRM TLCO 3.1.b Boration Flow Paths – Operating
- TRM TLCO 3.1.d Charging Pumps – Operating
- TRM TLCO 3.1.f Borated Water Sources – Operating
- TRM TLCO 3.1.g Position Indication System – Shutdown
- TRM TLCO 3.1.h Shutdown Margin (SDM) – MODE 1 and MODE 2 with $k_{eff} \geq 1.0$
- TRM TLCO 3.1.i Shutdown Margin (SDM) – MODE 5
- TRM TLCO 3.1.j Shutdown and Control Rods
- TRM TLCO 3.1.k Position Indication System – Shutdown (Special Test Exception)

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 17

2.0 OPERATING LIMITS

The cycle-specific parameter limits for the specifications listed in Section 1.0 are presented in the following subsections. These limits are applicable for the entire cycle unless otherwise identified. These limits have been developed using the NRC-approved methodologies specified in Technical Specification 5.6.5.

2.1 Reactor Core Safety Limits (SLs) (SL 2.1.1)

2.1.1 In MODES 1 and 2, the combination of Thermal Power, Reactor Coolant System (RCS) highest loop average temperature, and pressurizer pressure shall not exceed the limits specified in Figure 2.1.1.

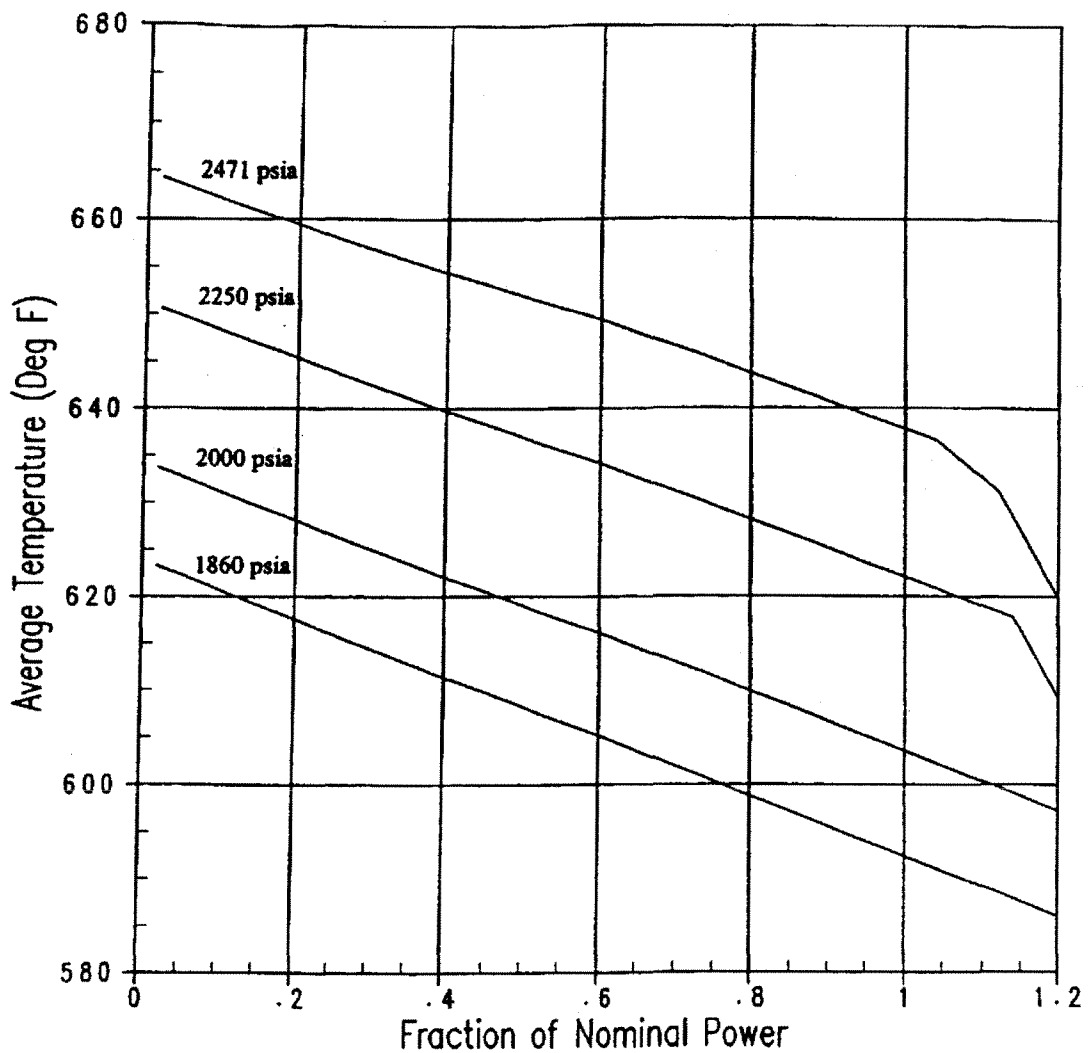


Figure 2.1.1: Reactor Core Limits

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 17

2.2 SHUTDOWN MARGIN (SDM)

The SDM limit for MODES 1, 2, 3, and 4 is:

2.2.1 The SDM shall be greater than or equal to 1.3% $\Delta k/k$ (LCOs 3.1.1, 3.1.4, 3.1.5, 3.1.6, 3.1.8, 3.3.9; TRM TLCOs 3.1.b, 3.1.d, 3.1.f, 3.1.h, and 3.1.j).

The SDM limit for MODE 5 is:

2.2.2 SDM shall be greater than or equal to 1.3% $\Delta k/k$ (LCO 3.1.1, LCO 3.3.9; TRM TLCOs 3.1.i and 3.1.j).

2.3 Moderator Temperature Coefficient (MTC) (LCO 3.1.3)

The Moderator Temperature Coefficient (MTC) limits are:

2.3.1 The BOL/ARO/HZP-MTC upper limit shall be $+1.772 \times 10^{-5} \Delta k/k/^{\circ}F$.

2.3.2 The EOL/ARO/HFP-MTC lower limit shall be $-4.6 \times 10^{-4} \Delta k/k/^{\circ}F$.

2.3.3 The EOL/ARO/HFP-MTC Surveillance limit at 300 ppm shall be $-3.7 \times 10^{-4} \Delta k/k/^{\circ}F$.

2.3.4 The EOL/ARO/HFP-MTC Surveillance limit at 60 ppm shall be $-4.3 \times 10^{-4} \Delta k/k/^{\circ}F$.

where: BOL stands for Beginning of Cycle Life
ARO stands for All Rods Out
HZP stands for Hot Zero Thermal Power
EOL stands for End of Cycle Life
HFP stands for Hot Full Thermal Power

2.4 Shutdown Bank Insertion Limits (LCO 3.1.5)

2.4.1 All shutdown banks shall be fully withdrawn to at least 224 steps.

2.5 Control Bank Insertion Limits (LCO 3.1.6)

2.5.1 The control banks, with Bank A greater than or equal to 224 steps, shall be limited in physical insertion as shown in Figure 2.5.1.

2.5.2 Each control bank shall be considered fully withdrawn from the core at greater than or equal to 224 steps.

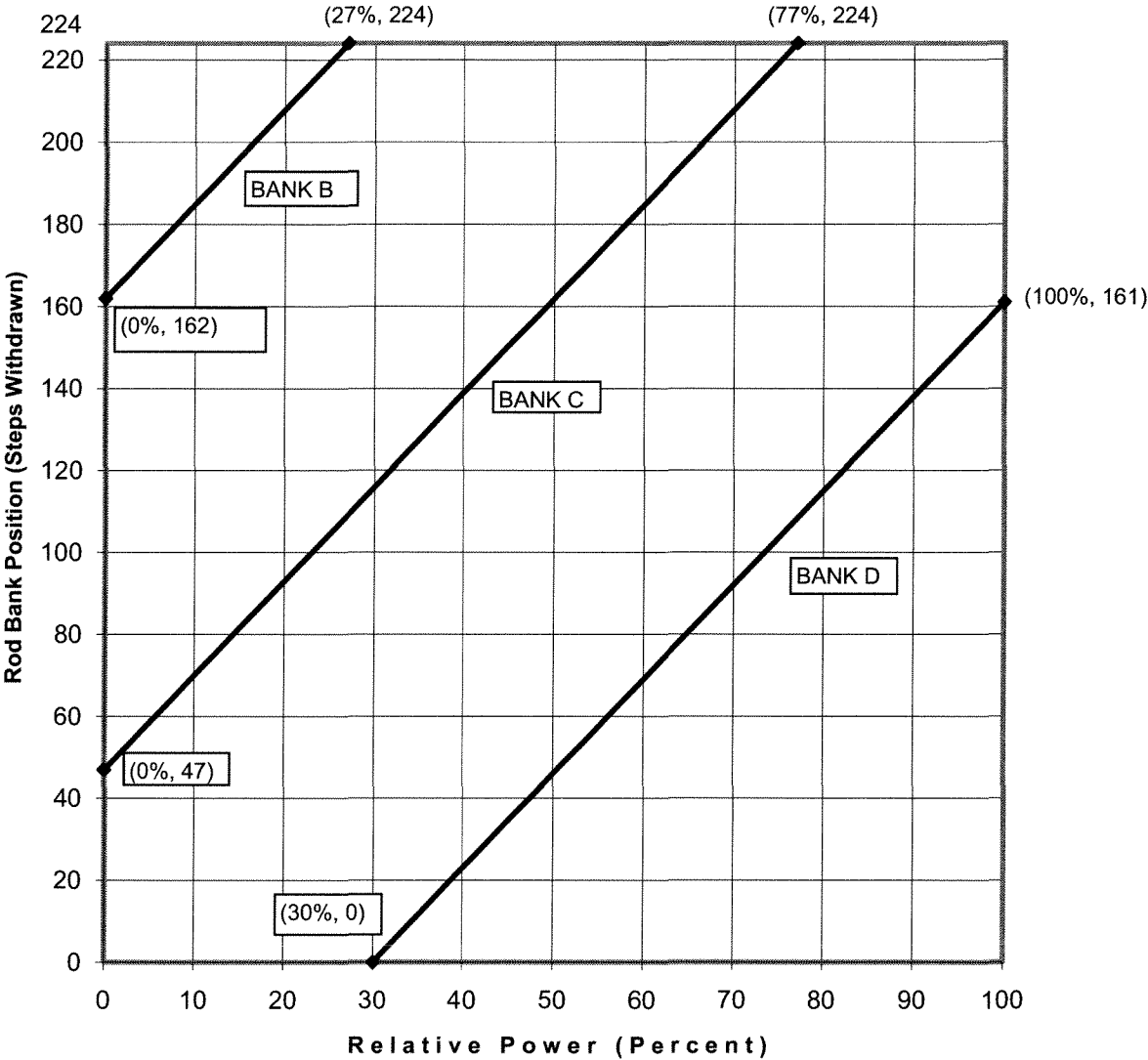
2.5.3 The control banks shall be operated in sequence by withdrawal of Bank A, Bank B, Bank C and Bank D. The control banks shall be sequenced in reverse order upon insertion.

2.5.4 Each control bank not fully withdrawn from the core shall be operated with the following overlap limits as a function of park position:

Park Position (step)	Overlap Limit (step)
226	111
227	112
228	113
229	114

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 17

**Figure 2.5.1:
Control Bank Insertion Limits Versus Percent Rated Thermal Power**



CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 17

2.6 Heat Flux Hot Channel Factor ($F_Q(Z)$) (LCO 3.2.1)

2.6.1 Total Peaking Factor:

$$F_Q(Z) \leq \frac{F_Q^{RTP}}{0.5} \times K(Z) \quad \text{for } P \leq 0.5$$

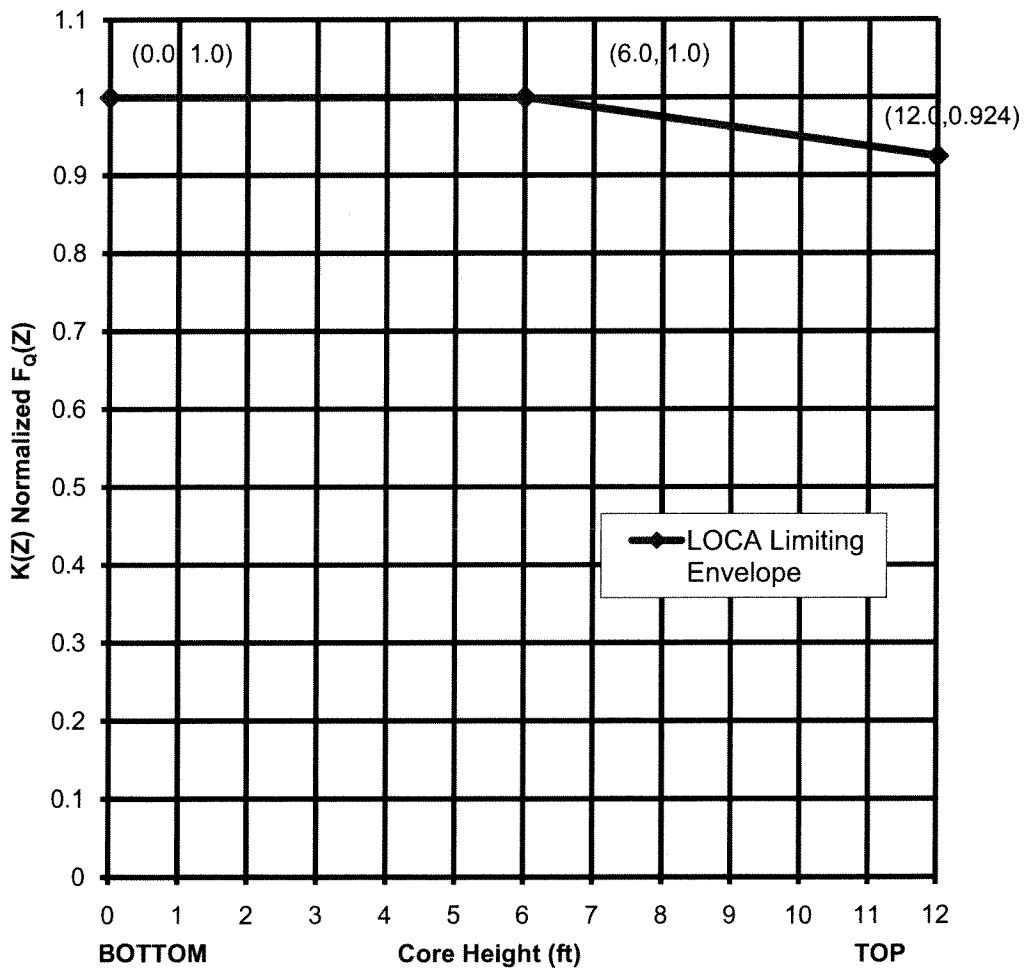
$$F_Q(Z) \leq \frac{F_Q^{RTP}}{P} \times K(Z) \quad \text{for } P > 0.5$$

where: P = the ratio of THERMAL POWER to RATED THERMAL POWER

$$F_Q^{RTP} = 2.60$$

$K(Z)$ is provided in Figure 2.6.1.

Figure 2.6.1
 $K(Z)$ - Normalized $F_Q(Z)$ as a Function of Core Height



CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 17

2.6.2 W(Z) Values:

a) When PDMS is OPERABLE, $W(Z) = 1.00000$ for all axial points.

b) When PDMS is inoperable, $W(Z)$ is provided as:

- 1) Table 2.6.2.a are the normal operation $W(Z)$ values for the full cycle and correspond to the AXIAL FLUX DIFFERENCE (AFD) Acceptable Operation Limits provided in Figure 2.8.1.a. The normal operation $W(Z)$ values have been determined at burnups of 150, 6000, 14000, and 20000 MWD/MTU.
- 2) The EOL-only normal operation $W(Z)$ values provided in Table 2.6.2.b may be used for cycle burnups ≥ 18000 MWD/MTU. The EOL-only $W(Z)$ values correspond to the REDUCED AXIAL FLUX DIFFERENCE (AFD) Acceptable Operation Limits provided in Figure 2.8.1.b. The EOL-only normal operation $W(Z)$ values have been determined at burnups of 18000 and 20000 MWD/MTU and the last column of $W(Z)$ values is a duplicate of the 20000 MWD/MTU values. If invoked, the EOL-only $W(Z)$ values are to be used for the remainder of the cycle unless superseded by a subsequent analysis.

Table 2.6.2.c shows the $F_{\alpha}^C(z)$ penalty factors that are greater than 2% per 31 Effective Full Power Days (EFPD). These values shall be used to increase the $F_{\alpha}^W(z)$ as per Surveillance Requirement 3.2.1.2. A 2% penalty factor shall be used at all cycle burnups that are outside the range of Table 2.6.2.c.

2.6.3 Uncertainty:

The uncertainty, U_{FQ} , to be applied to the Heat Flux Hot Channel Factor $F_{\alpha}(Z)$ shall be calculated by the following formula

$$U_{FQ} = U_{qu} \bullet U_e$$

where:

U_{qu} = Base F_{α} measurement uncertainty = 1.05 when PDMS is inoperable
(U_{qu} is defined by PDMS when OPERABLE.)

U_e = Engineering uncertainty factor = 1.03

2.6.4 PDMS Alarms:

$F_{\alpha}(Z)$ Warning Setpoint = 2% $F_{\alpha}(Z)$ Margin

$F_{\alpha}(Z)$ Alarm Setpoint = 0% $F_{\alpha}(Z)$ Margin

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 17

Table 2.6.2.a

Full Cycle W(Z) versus Core Height for AFD Acceptable Operation Limits in Figure 2.8.1.a
(Top and Bottom 8% Excluded per WCAP-10216)

Height (feet)	150 MWD/MTU	6000 MWD/MTU	14000 MWD/MTU	20000 MWD/MTU
0.00 (core bottom)	1.2875	1.4463	1.4370	1.3976
0.20	1.2772	1.4175	1.4240	1.3732
0.40	1.2740	1.4005	1.4070	1.3580
0.60	1.2698	1.3866	1.3910	1.3429
0.80	1.2558	1.3654	1.3460	1.3163
1.00	1.2465	1.3486	1.3280	1.2974
1.20	1.2488	1.3320	1.3130	1.2864
1.40	1.2598	1.3231	1.3010	1.2723
1.60	1.2461	1.3017	1.2720	1.2601
1.80	1.2337	1.2821	1.2616	1.2524
2.00	1.2207	1.2615	1.2539	1.2430
2.20	1.2059	1.2383	1.2430	1.2305
2.40	1.1926	1.2175	1.2332	1.2201
2.60	1.1768	1.1958	1.2201	1.2057
2.80	1.1694	1.1796	1.2092	1.1924
3.00	1.1618	1.1758	1.1965	1.1780
3.20	1.1531	1.1719	1.1812	1.1640
3.40	1.1475	1.1675	1.1690	1.1637
3.60	1.1426	1.1616	1.1654	1.1620
3.80	1.1375	1.1557	1.1624	1.1614
4.00	1.1332	1.1493	1.1588	1.1620
4.20	1.1339	1.1409	1.1538	1.1615
4.40	1.1345	1.1325	1.1492	1.1591
4.60	1.1341	1.1231	1.1422	1.1608
4.80	1.1328	1.1137	1.1347	1.1683
5.00	1.1304	1.1037	1.1268	1.1738
5.20	1.1270	1.0962	1.1292	1.1786
5.40	1.1227	1.0913	1.1314	1.1819
5.60	1.1231	1.0876	1.1318	1.1925
5.80	1.1300	1.0907	1.1311	1.2108
6.00	1.1366	1.0963	1.1425	1.2264
6.20	1.1414	1.1043	1.1551	1.2389
6.40	1.1451	1.1110	1.1667	1.2486
6.60	1.1479	1.1177	1.1754	1.2542
6.80	1.1487	1.1234	1.1811	1.2580
7.00	1.1495	1.1282	1.1859	1.2588
7.20	1.1474	1.1345	1.1868	1.2546
7.40	1.1565	1.1449	1.1867	1.2496
7.60	1.1655	1.1536	1.1827	1.2396
7.80	1.1738	1.1617	1.1778	1.2277
8.00	1.1812	1.1694	1.1700	1.2139
8.20	1.1870	1.1748	1.1592	1.1962
8.40	1.1924	1.1804	1.1475	1.1784
8.60	1.1957	1.1885	1.1329	1.1597
8.80	1.1995	1.2000	1.1324	1.1452
9.00	1.2019	1.2182	1.1364	1.1396
9.20	1.2110	1.2354	1.1422	1.1340
9.40	1.2222	1.2522	1.1508	1.1510
9.60	1.2325	1.2674	1.1583	1.1950
9.80	1.2427	1.2818	1.1730	1.2330
10.00	1.2523	1.2955	1.2010	1.2670
10.20	1.2597	1.3075	1.2330	1.2970
10.40	1.2577	1.3100	1.2560	1.3220
10.60	1.2327	1.3055	1.2780	1.3360
10.80	1.2356	1.3037	1.2880	1.3540
11.00	1.2318	1.3136	1.2870	1.3680
11.20	1.2407	1.3177	1.2760	1.3770
11.40	1.2411	1.3300	1.2820	1.3700
11.60	1.2445	1.3359	1.2664	1.3562
11.80	1.2498	1.3426	1.2603	1.3501
12.00 (core top)	1.2574	1.3584	1.2560	1.3518

Note: W(Z) values at 20000 MWD/MTU may be applied to cycle burnups greater than 20000 MWD/MTU to prevent W(Z) function extrapolation

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 17

Table 2.6.2.b EOL-only W(Z) versus Core Height for AFD Acceptable Operation Limits in Figure 2.8.1.b (Top and Bottom 8% Excluded per WCAP-10216)			
Height (feet)	18000 MWD/MTU	20000 MWD/MTU	25330 MWD/MTU
0.00 (core bottom)	1.2717	1.2572	1.2572
0.20	1.2569	1.2365	1.2365
0.40	1.2428	1.2226	1.2226
0.60	1.2290	1.2083	1.2083
0.80	1.1982	1.1862	1.1862
1.00	1.1827	1.1707	1.1707
1.20	1.1733	1.1627	1.1627
1.40	1.1636	1.1513	1.1513
1.60	1.1581	1.1443	1.1443
1.80	1.1533	1.1388	1.1388
2.00	1.1475	1.1320	1.1320
2.20	1.1394	1.1227	1.1227
2.40	1.1324	1.1150	1.1150
2.60	1.1261	1.1108	1.1108
2.80	1.1223	1.1110	1.1110
3.00	1.1223	1.1132	1.1132
3.20	1.1236	1.1180	1.1180
3.40	1.1253	1.1227	1.1227
3.60	1.1261	1.1264	1.1264
3.80	1.1266	1.1298	1.1298
4.00	1.1291	1.1369	1.1369
4.20	1.1320	1.1438	1.1438
4.40	1.1364	1.1521	1.1521
4.60	1.1445	1.1608	1.1608
4.80	1.1512	1.1683	1.1683
5.00	1.1559	1.1738	1.1738
5.20	1.1601	1.1786	1.1786
5.40	1.1634	1.1819	1.1819
5.60	1.1700	1.1925	1.1925
5.80	1.1797	1.2108	1.2108
6.00	1.1940	1.2264	1.2264
6.20	1.2069	1.2389	1.2389
6.40	1.2178	1.2486	1.2486
6.60	1.2249	1.2542	1.2542
6.80	1.2294	1.2580	1.2580
7.00	1.2320	1.2588	1.2588
7.20	1.2295	1.2546	1.2546
7.40	1.2257	1.2496	1.2496
7.60	1.2173	1.2396	1.2396
7.80	1.2074	1.2277	1.2277
8.00	1.1950	1.2139	1.2139
8.20	1.1789	1.1962	1.1962
8.40	1.1624	1.1784	1.1784
8.60	1.1436	1.1597	1.1597
8.80	1.1341	1.1452	1.1452
9.00	1.1315	1.1396	1.1396
9.20	1.1298	1.1340	1.1340
9.40	1.1425	1.1510	1.1510
9.60	1.1699	1.1950	1.1950
9.80	1.1978	1.2330	1.2330
10.00	1.2303	1.2670	1.2670
10.20	1.2628	1.2970	1.2970
10.40	1.2886	1.3220	1.3220
10.60	1.3080	1.3360	1.3360
10.80	1.3236	1.3540	1.3540
11.00	1.3301	1.3680	1.3680
11.20	1.3292	1.3770	1.3770
11.40	1.3272	1.3700	1.3700
11.60	1.3109	1.3562	1.3562
11.80	1.3037	1.3501	1.3501
12.00 (core top)	1.3011	1.3518	1.3518

Note: W(Z) values at 20000 MWD/MTU may be applied to cycle burnups greater than 20000 MWD/MTU to prevent W(Z) function extrapolation

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 17

Table 2.6.2.c	
Penalty Factors in Excess of 2% per 31 EFPD	
Cycle Burnup	Penalty Factor
(MWD/MTU)	$F_{\alpha}^C(z)$
11218	1.0200
11391	1.0217
11564	1.0241
11736	1.0264
11909	1.0287
12082	1.0311
12255	1.0329
12428	1.0345
12601	1.0360
12774	1.0332
12947	1.0297
13120	1.0260
13293	1.0219
13466	1.0200

Notes:

Linear interpolation is adequate for intermediate cycle burnups.

All cycle burnups outside the range of the table shall use a 2% penalty factor for compliance with the 3.2.1.2 Surveillance Requirements.

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 17

2.7 Nuclear Enthalpy Rise Hot Channel Factor ($F_{\Delta H}^N$) (LCO 3.2.2)

2.7.1 $F_{\Delta H}^N \leq F_{\Delta H}^{RTP} [1.0 + PF_{\Delta H}(1.0 - P)]$

where: P = the ratio of THERMAL POWER to RATED THERMAL POWER

$F_{\Delta H}^{RTP} = 1.70$

$PF_{\Delta H} = 0.3$

2.7.2 Uncertainty:

The uncertainty, $U_{F_{\Delta H}}$, to be applied to the Nuclear Enthalpy Rise Hot Channel Factor $F_{\Delta H}^N$ shall be calculated by the following formula:

$U_{F_{\Delta H}} = U_{F_{\Delta Hm}}$

where:

$U_{F_{\Delta Hm}} = \text{Base } F_{\Delta H}^N \text{ measurement uncertainty} = 1.04 \text{ when PDMS is inoperable}$
($U_{F_{\Delta Hm}}$ is defined by PDMS when OPERABLE.)

2.7.3 PDMS Alarms:

$F_{\Delta H}^N \text{ Warning Setpoint} = 2\% F_{\Delta H}^N \text{ Margin}$

$F_{\Delta H}^N \text{ Alarm Setpoint} = 0\% F_{\Delta H}^N \text{ Margin}$

2.8 AXIAL FLUX DIFFERENCE (AFD) (LCO 3.2.3)

2.8.1 When PDMS is inoperable, the AXIAL FLUX DIFFERENCE (AFD) Acceptable Operation Limits are provided in the Figures described below or the latest valid PDMS Surveillance Report, whichever is more conservative.

a) Figure 2.8.1.a is the full cycle AFD Acceptable Operation Limits associated with the full cycle W(Z) values in Table 2.6.2.a.

b) Figure 2.8.1.b is the Reduced AFD Acceptable Operation Limits which may be applied after 18000 MWD/MTU. The Reduced AFD Acceptable Operation Limits are associated with the EOL-only W(Z) values in Table 2.6.2.b. Prior to changing to Figure 2.8.1.b, confirm that the plant is within the specified AFD envelope.

2.8.2 When PDMS is OPERABLE, no AFD Acceptable Operation Limits are applicable.

2.9 Departure from Nucleate Boiling Ratio (DNBR) (LCO 3.2.5)

2.9.1 $DNBR_{APSL} \geq 1.539$

The Axial Power Shape Limiting DNBR ($DNBR_{APSL}$) is applicable with THERMAL POWER $\geq 50\%$ RTP when PDMS is OPERABLE.

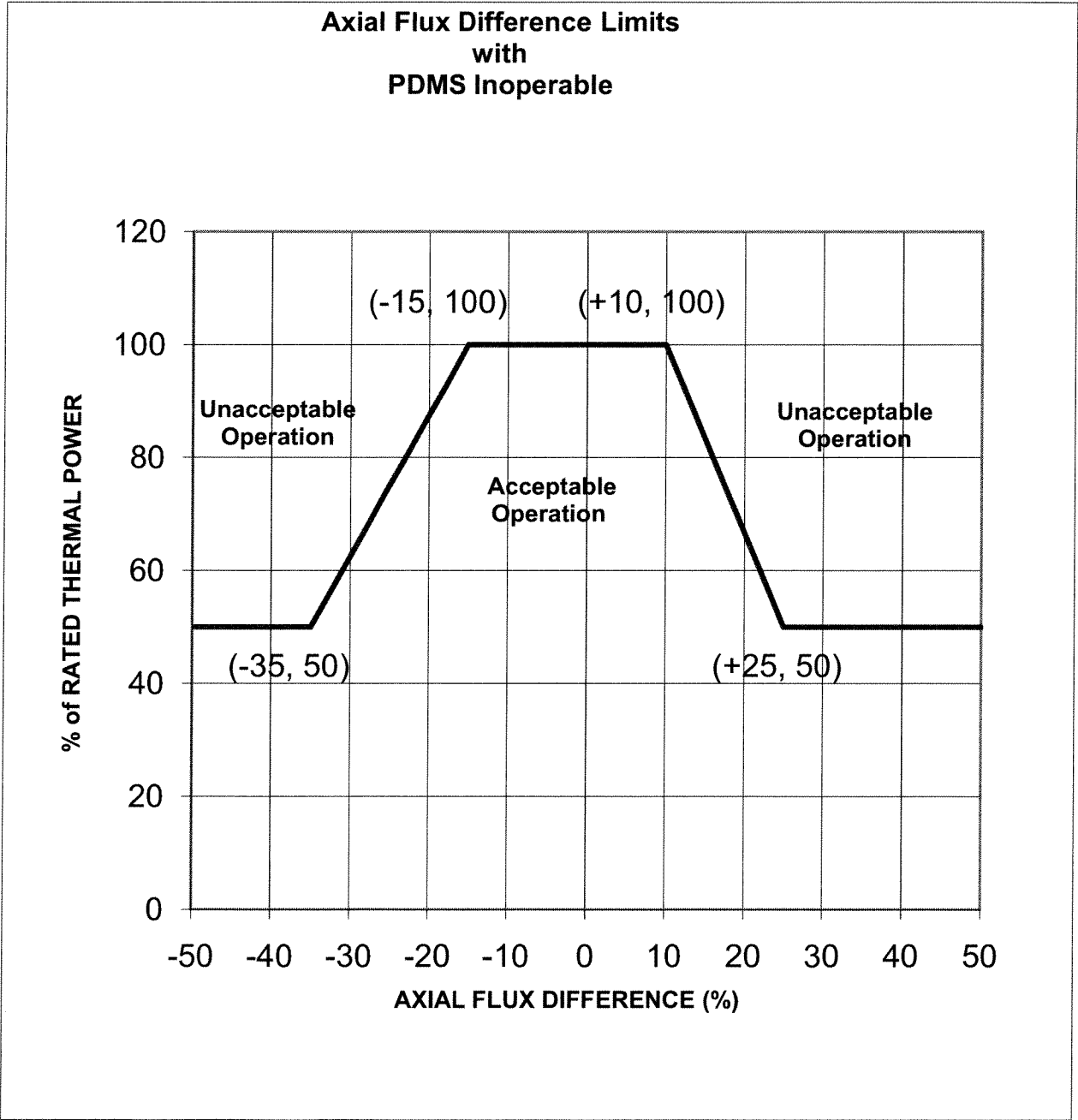
2.9.2 PDMS Alarms:

$DNBR \text{ Warning Setpoint} = 2\% DNBR \text{ Margin}$

$DNBR \text{ Alarm Setpoint} = 0\% DNBR \text{ Margin}$

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 17

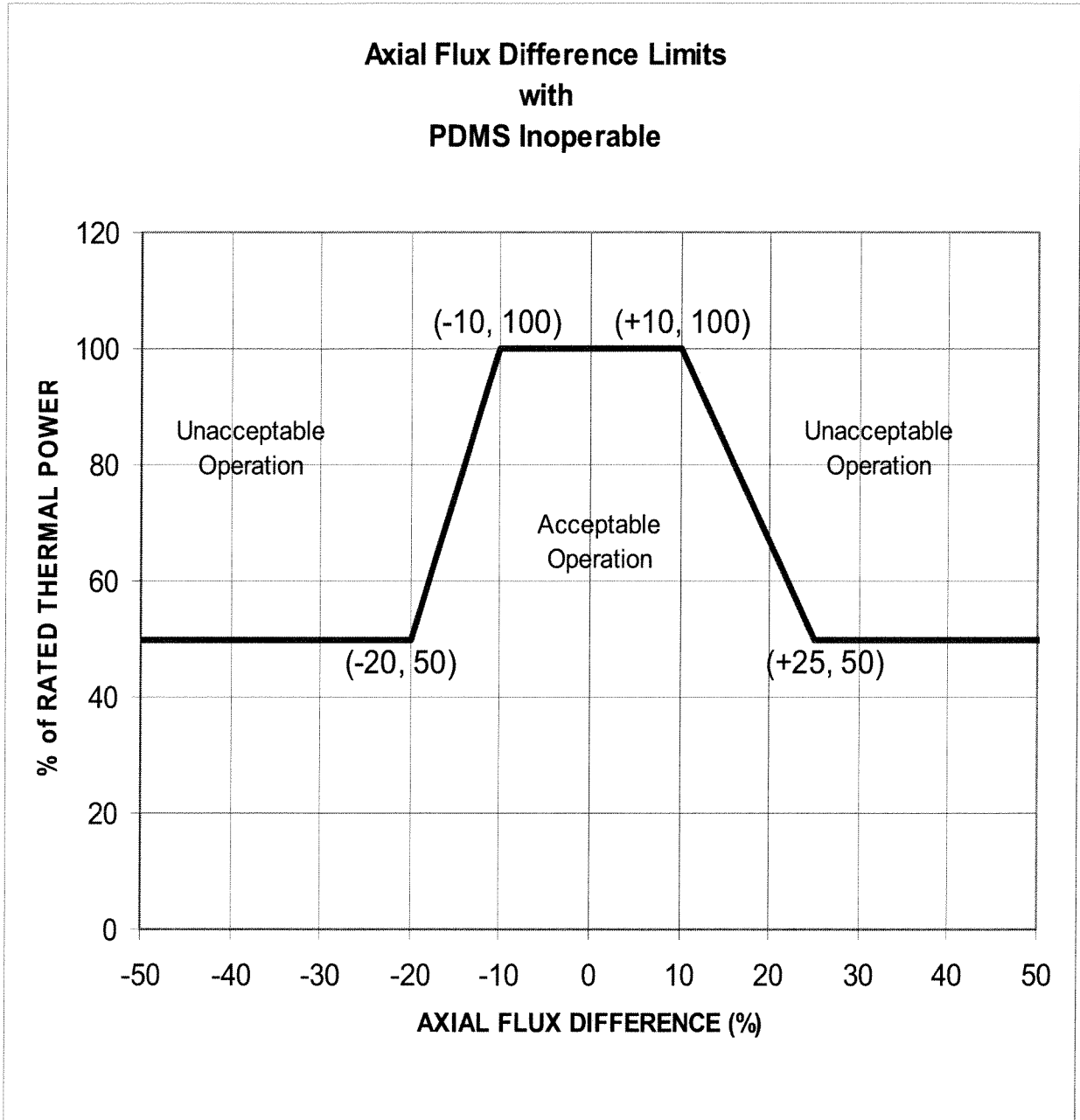
Figure 2.8.1.a:
Axial Flux Difference Limits
as a Function of Rated Thermal Power
(Full Cycle)



CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 17

Figure 2.8.1.b:
Reduced Axial Flux Difference Limits
as a Function of Rated Thermal Power

(Cycle burnup ≥ 18000 MWD/MTU)



CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 17

- 2.10 Reactor Trip System (RTS) Instrumentation (LCO 3.3.1) - Overtemperature ΔT Setpoint Parameter Values
- 2.10.1 The Overtemperature ΔT reactor trip setpoint K_1 shall be equal to 1.325.
 - 2.10.2 The Overtemperature ΔT reactor trip setpoint T_{avg} coefficient K_2 shall be equal to 0.0297 / °F.
 - 2.10.3 The Overtemperature ΔT reactor trip setpoint pressure coefficient K_3 shall be equal to 0.00181 / psi.
 - 2.10.4 The nominal T_{avg} at RTP (indicated) T' shall be less than or equal to 588.0 °F.
 - 2.10.5 The nominal RCS operating pressure (indicated) P' shall be equal to 2235 psig.
 - 2.10.6 The measured reactor vessel ΔT lead/lag time constant τ_1 shall be equal to 8 sec.
 - 2.10.7 The measured reactor vessel ΔT lead/lag time constant τ_2 shall be equal to 3 sec.
 - 2.10.8 The measured reactor vessel ΔT lag time constant τ_3 shall be less than or equal to 2 sec.
 - 2.10.9 The measured reactor vessel average temperature lead/lag time constant τ_4 shall be equal to 33 sec.
 - 2.10.10 The measured reactor vessel average temperature lead/lag time constant τ_5 shall be equal to 4 sec.
 - 2.10.11 The measured reactor vessel average temperature lag time constant τ_6 shall be less than or equal to 2 sec.
 - 2.10.12 The f_1 (ΔI) "positive" breakpoint shall be +10% ΔI .
 - 2.10.13 The f_1 (ΔI) "negative" breakpoint shall be -18% ΔI .
 - 2.10.14 The f_1 (ΔI) "positive" slope shall be +3.47% / % ΔI .
 - 2.10.15 The f_1 (ΔI) "negative" slope shall be -2.61% / % ΔI .

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 17

2.11 Reactor Trip System (RTS) Instrumentation (LCO 3.3.1) - Overpower ΔT Setpoint Parameter Values

- 2.11.1 The Overpower ΔT reactor trip setpoint K_4 shall be equal to 1.072.
- 2.11.2 The Overpower ΔT reactor trip setpoint T_{avg} rate/lag coefficient K_5 shall be equal to 0.02 / $^{\circ}F$ for increasing T_{avg} .
- 2.11.3 The Overpower ΔT reactor trip setpoint T_{avg} rate/lag coefficient K_5 shall be equal to 0 / $^{\circ}F$ for decreasing T_{avg} .
- 2.11.4 The Overpower ΔT reactor trip setpoint T_{avg} heatup coefficient K_6 shall be equal to 0.00245 / $^{\circ}F$ when $T > T''$.
- 2.11.5 The Overpower ΔT reactor trip setpoint T_{avg} heatup coefficient K_6 shall be equal to 0 / $^{\circ}F$ when $T \leq T''$.
- 2.11.6 The nominal T_{avg} at RTP (indicated) T'' shall be less than or equal to 588.0 $^{\circ}F$
- 2.11.7 The measured reactor vessel ΔT lead/lag time constant τ_1 shall be equal to 8 sec.
- 2.11.8 The measured reactor vessel ΔT lead/lag time constant τ_2 shall be equal to 3 sec.
- 2.11.9 The measured reactor vessel ΔT lag time constant τ_3 shall be less than or equal to 2 sec.
- 2.11.10 The measured reactor vessel average temperature lag time constant τ_6 shall be less than or equal to 2 sec.
- 2.11.11 The measured reactor vessel average temperature rate/lag time constant τ_7 shall be equal to 10 sec.
- 2.11.12 The $f_2(\Delta I)$ "positive" breakpoint shall be 0 for all ΔI .
- 2.11.13 The $f_2(\Delta I)$ "negative" breakpoint shall be 0 for all ΔI .
- 2.11.14 The $f_2(\Delta I)$ "positive" slope shall be 0 for all ΔI .
- 2.11.15 The $f_2(\Delta I)$ "negative" slope shall be 0 for all ΔI .

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 17

2.12 Reactor Coolant System (RCS) Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits (LCO 3.4.1)

2.12.1 The pressurizer pressure shall be greater than or equal to 2209 psig.

2.12.2 The RCS average temperature (T_{avg}) shall be less than or equal to 593.1 °F.

2.12.3 The RCS total flow rate shall be greater than or equal to 386,000 gpm.

2.13 Boron Concentration

2.13.1 The refueling boron concentration shall be greater than or equal to the value given in the Table below (LCO 3.9.1). The reported value also bounds the end-of-cycle requirements for the previous cycle.

2.13.2 To maintain $k_{eff} \leq 0.987$ with all shutdown and control rods fully withdrawn in MODES 3, 4, or 5 (TRM TLCO 3.1.g Required Action B.2 and TRM TLCO 3.1.k.2), the Reactor Coolant System boron concentration shall be greater than or equal to the values given in the Table below.

COLR Section	Conditions	Boron Concentration (ppm)
2.13.1	Refueling (prior to initial criticality)	1678
2.13.2	a) prior to initial criticality	1747
2.13.2	b) all other times in life	1992