

August 30, 2012

NG-12-0346 10 CFR 50.73

U.S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, D.C. 20555-0001

Duane Arnold Energy Center Docket 50-331 License No. DPR-49

Licensee Event Report #2012-004-00

Please find attached the subject report submitted in accordance with 10 CFR 50.73. This letter makes no new commitments or changes to any existing commitments.

Richard L. Anderson Vice President, Duane Arnold Energy Center NextEra Energy Duane Arnold, LLC



NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION						APPROVED BY OMB NO. 3150-0104 EXPIRES: 10/31/2013										
(10-2010) LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block)							Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOLA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202 (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.									
1. FACILI	ITY NAM	E						2. DOCH	2. DOCKET NUMBER					3. PAGE		
Duane	<u>Arnold</u>	l Ener	gy Cente	r				05000	331				1 OF 3			
4. TITLE High Pressure Coolant Injection Declared Inoperable																
ý i	VENT DA			LER NUMBE			REPORT	8. OTHER FACILITIES INVOLVED								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER		MONTH		YEAR	facility N/A	NAME			DOCKET NUMBER			
07	02	12	2012	004	0	08	30	12	facility N/A				docket number 05000			
9. OPERATING MODE 11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)																
	1			201(b)		20.2203(a		50.73(a)(2)(i)(C) 50.73(a)(2)(vii)								
	1						20.2203(a)(3)(ii) 20.2203(a)(4)						73(a)(2)(viii)(A)			
				203(a)(1) 203(a)(2)(i)		50.36(c)(50.73(a)(2)(ii)(B) 50.73(a)(2)(viii)(B) 50.73(a)(2)(iii) 50.73(a)(2)(ix)(A)								
10. POW	ER LEV	EL	20.2	203(a)(2)(ii)		50.36(c)(1)(ii)(A)			50.73(a)(2)(iv)(A) 5				.73(a)(2)(x)			
			20.2	20.2203(a)(2)(iii) 50.36(c)(2			2)	= 50.73(a)(2)(v)(A) = 73.					71(a)(4)			
100%			—	203(a)(2)(iv)		50.46(a)(3)(ii)							71(a)(5)			
				203(a)(2)(v) 203(a)(2)(vi)		50.73(a)(2)(i)(A) 50.73(a)(2)(i)(B)			50.73(a)(2)(v)(C) 50.73(a)(2)(v)(D)				OTHER VOLUNTARY LER			
			20.2			50.75(a)(.		2	(a)	(2)(V)(D)		volu				
NAME					12. LIC	CENSEE C	ONTACT	FOR THIS	LER		TELEPH	ONE NUMBER	(Include Ar	ea Code)		
							TELEPHONE NUMBER (Include Area Code) (319) 851-7900					ea coae)				
Robert J. Murrell, Engineering Analyst (319) 851-7900 13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT																
			DMPONENT	MANU-	REPORTABLE		CAUSE		SYSTEM		COMPONENT MAN		- REPORTABLE			
	<u> </u>			FACTURER	TO EF	PIX						FACTURER	TO EPIX			
14. SUPPLEMENTAL REPORT EXPECTED								15. EXPECTED MONTH			DAY	YEAR				
YES (If yes, complete 15. EXPECTED SUBMISSION DATE)							SUBMISSION DATE									
ABSTDACT (Limit to 1400 angage i.e. approximately 15 single angage typowyitten ling)																

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On July 2, 2012, at 15:05, while operating at 100% power, a spurious isolation of the High Pressure Coolant Injection (HPCI) Outboard Steam Supply Valve and Outboard Torus Suction Valve occurred. The isolation resulted from a trip of the HPCI Steam Leak Detection (SLD) system. Immediate investigations into the event determined that there was no steam leak and the probable cause was from a bad connection in the SLD system. Subsequent investigation identified a broken solid conductor thermocouple extension wire. The wire was damaged earlier in the day as part of maintenance activities investigating erroneous readings on unrelated circuits. The damaged wire was replaced and HPCI was declared operable on July 3, 2012, at 0002. The root cause of this event was an inadequate installation position of a recorder in the control room panel. The recorder was installed in such a manner that the removal of its terminal cover placed excessive stress on the thermocouple wire resulting in its failure.

This condition was reported under 10 CFR 50.72(b)(3)(v)(D), any event or condition that at the time of discovery could have prevented the fulfillment of the safety function of the structures or systems that are needed to mitigate the consequences of an accident. Reference EN#48064.

The safety significance of this event was minimized by the fact that all other Emergency Core Cooling Systems (ECCS) remained fully operable during the time that HPCI was inoperable.

NRC FORM 366A (10-2010)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

	OULEET
CONTINUATION	
CONTINUATION	JILLI

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3	. PAGE		
Duane Arnold Energy Center		YEAR	SEQUENTIAL REV NUMBER NO.					
	05000 - 331	2012	004	0	2	OF 3		

NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

I. Description of Event:

On July 2, 2012, at 15:05, while operating at 100% power, a spurious isolation of the High Pressure Coolant Injection (HPCI) (System Code BG) Outboard Steam Supply Valve and Outboard Torus Suction Valve occurred. The isolation resulted from a trip of the HPCI Steam Leak Detection (SLD) system. Immediate investigations into the event determined that there was no steam leak and the probable cause was from a bad connection in the SLD system. Subsequent investigation identified a broken solid conductor thermocouple extension wire. The wire was damaged earlier in the day as part of maintenance activities investigating erroneous readings unrelated circuits. The damaged wire was replaced and HPCI was declared operable on July 3, 2012, at 0002.

There were no structures, systems or components inoperable at the start of this event that contributed to the event.

II. Assessment of Safety Consequences:

During the period HPCI was inoperable, all other Emergency Core Cooling Systems were operable, and therefore fully capable of mitigating the consequences of an accident had it occurred.

This event did result in a safety system functional failure.

III. Cause of Event:

A root cause evaluation was completed for this event. The root cause of this event was an inadequate installation position of a recorder in the control room panel. The recorder was installed in February 2007 in such a manner that the removal of its terminal cover placed excessive stress on the thermocouple wire resulting in its failure.

IV. Corrective Actions:

On July 03, 2012, all maintenance activities associated with replacing the failed thermocouple wire were completed, and HPCI was declared operable.

The following actions are planned to address the root cause of this event:

Installation of signs inside the cabinet containing the SLD thermocouple wires warning of the potential fragility of the wires in the vicinity of the terminal covers for the associated recorders.

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

1. FACILITY NAME	2. DOCKET		3. PAGE						
Duane Arnold Energy Center		YEAR	SEQUENTIAL NUMBER	REV NO.					
	05000 - 331	2012	004	0	3 OF 3				

NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

Completion of an inspection of other thermocouple wires associated with recorders installed by the Engineering Change Package that installed the recorder identified as the root cause of this event.

Revision of the model Work Order associated with work inside the cabinet containing the SLD thermocouple wires to require maintenance technicians be briefed on the susceptibility of the SLD wires to break.

V. Additional Information:

Previous Similar Occurrences:

A review of License Event Reports from the past 5 years did not identify any previous similar occurrences where HPCI was declared inoperable due to an invalid SLD isolation.

EIIS System and Component Codes:

BJ - High Pressure Coolant Injection System (BWR)

Reporting Requirements:

This event is being reported as an Event or Condition that Could Have Prevented Fulfillment of a Safety Function, 10CFR50.73(a)(2)(v)(D). Additionally, this event was reported under 50.72(b)(3)(v)(D), any event or condition that at the time of discovery could have prevented the fulfillment of the safety function of the structures or systems that are needed to mitigate the consequences of an accidentjidiekdihfjiejfieikdbobmurrelll. Reference EN#48064.