



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

October 1, 2012

Mr. Preston Gillespie
Site Vice President
Oconee Nuclear Station
Duke Energy Carolinas, LLC
7800 Rochester Highway
Seneca, SC 29672-0752

SUBJECT: OCONEE NUCLEAR STATION, UNIT 1, ISSUANCE OF AMENDMENT
REGARDING EXTENSION OF THE REACTOR BUILDING INTEGRATED LEAK
RATE TEST (TAC NO. ME8407)

Dear Mr. Gillespie:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 381 to Renewed Facility Operating License DPR-38, for the Oconee Nuclear Station, Unit 1. The amendment consist of changes to the Technical Specifications (TSs) in response to your application dated April 3, 2012.

This amendment revises the due date for the next integrated leak rate test of the reactor building from December 8, 2013, to March 8, 2015, which aligns with the two-year refueling outage schedule at Oconee Nuclear Station, Unit 1.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

If you have any questions, please call me at 301-415-2901.

Sincerely,

A handwritten signature in cursive script that reads "John P. Boska".

John P. Boska, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-269

Enclosures:

1. Amendment No. 381 to DPR-38
2. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE ENERGY CAROLINAS, LLC

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 381
Renewed License No. DPR-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 1 (the facility), Renewed Facility Operating License No. DPR-38, filed by Duke Energy Carolinas, LLC (the licensee), dated April 3, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B of Renewed Facility Operating License No. DPR-38 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 381, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert J. Pascarelli, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to Renewed Facility
Operating License No. DPR-38
and the Technical Specifications

Date of Issuance: October 1, 2012

ATTACHMENT TO LICENSE AMENDMENT NO. 381

RENEWED FACILITY OPERATING LICENSE NO. DPR-38

DOCKET NO. 50-269

Replace the following pages of the License and the Appendix A Technical Specifications (TSs) with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages

Insert Pages

Licenses

Licenses

License No. DPR-38, page 3

License No. DPR-38, page 3

TSs

Page 5.0-7

Page 5.0-8

TSs

Page 5.0-7

Page 5.0-8

A. Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 2568 megawatts thermal.

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 381 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

C. This license is subject to the following antitrust conditions:

Applicant makes the commitments contained herein, recognizing that bulk power supply arrangements between neighboring entities normally tend to serve the public interest. In addition, where there are net benefits to all participants, such arrangements also serve the best interests of each of the participants. Among the benefits of such transactions are increased electric system reliability, a reduction in the cost of electric power, and minimization of the environmental effects of the production and sale of electricity.

Any particular bulk power supply transaction may afford greater benefits to one participant than to another. The benefits realized by a small system may be proportionately greater than those realized by a larger system. The relative benefits to be derived by the parties from a proposed transaction, however, should not be controlling upon a decision with respect to the desirability of participating in the transaction. Accordingly, applicant will enter into proposed bulk power transactions of the types hereinafter described which, on balance, provide net benefits to applicant. There are net benefits in a transaction if applicant recovers the cost of the transaction (as defined in ¶1 (d) hereof) and there is no demonstrable net detriment to applicant arising from that transaction.

1. As used herein:

- (a) "Bulk Power" means electric power and any attendant energy, supplied or made available at transmission or sub-transmission voltage by one electric system to another.
- (b) "Neighboring Entity" means a private or public corporation, a governmental agency or authority, a municipality, a cooperative, or a lawful association of any of the foregoing owning or operating, or proposing to own or operate, facilities for the generation and transmission of electricity which meets each of

5.0 ADMINISTRATIVE CONTROLS

5.5 Programs and Manuals

The following programs shall be established, implemented, and maintained.

5.5.1 Offsite Dose Calculation Manual (ODCM)

The ODCM shall contain the methodology and parameters used in the calculation of offsite doses resulting from radioactive gaseous and liquid effluents, in the calculation of gaseous and liquid effluent monitoring alarm and trip setpoints, and in the conduct of the radiological environmental monitoring program.

Licensee initiated changes to the ODCM:

- a. Shall be documented and records of reviews performed shall be retained. This documentation shall contain:
 1. sufficient information to support the change(s) together with the appropriate analyses or evaluations justifying the change(s), and
 2. a determination that the change(s) do not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations;
- b. Shall become effective after the approval of the Station Manager; and
- c. Shall be submitted to the NRC in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Radioactive Effluent Release Report for the period of the report in which any change in the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (i.e., month and year) the change was implemented.

5.5.2 Containment Leakage Rate Testing Program

A program shall establish the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. The next Unit 1 ILRT following the December 8, 2003 test shall be performed no later than March 8, 2015. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163,

5.5 Programs and Manuals

5.5.2 Containment Leakage Rate Testing Program (continued)

“Performance-Based Containment Leak-Test Program,” dated September 1995. Containment system visual examinations required by Regulatory Guide 1.163, Regulatory Position C.3 shall be performed as follows:

1. Accessible concrete surfaces and post-tensioning system component surfaces of the concrete containment shall be visually examined prior to initiating SR 3.6.1.1 Type A test. These visual examinations, or any portion thereof, shall be performed no earlier than 90 days prior to the start of refueling outages in which Type A tests will be performed. The validity of these visual examinations will be evaluated should any event or condition capable of affecting the integrity of the containment system occur between the completion of the visual examinations and the Type A test.
2. Accessible interior and exterior surfaces of metallic pressure retaining components of the containment system shall be visually examined at least three times every ten years, including during each shutdown for SR 3.6.1.1 Type A test, prior to initiating the Type A test.

The calculated peak containment internal pressure for the design basis loss of coolant accident, P_a , is 59 psig. The containment design pressure is 59 psig.

The maximum allowable containment leakage rate, L_a , at P_a , shall be 0.20% of the containment air weight per day.

Leakage rate acceptance criterion is:

- a. Containment leakage rate acceptance criterion is $\leq 1.0 L_a$. During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria are $\leq 0.60 L_a$ for the Type B and C tests, and $\leq 0.75 L_a$ for Type A tests;

The provisions of SR 3.0.3 are applicable to the Containment Leakage Rate Testing Program.

Nothing in these Technical Specifications shall be construed to modify the testing Frequencies of 10 CFR 50, Appendix J.



UNITED STATES
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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO

AMENDMENT NO. 381 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-38

DUKE ENERGY CAROLINAS, LLC

OCONEE NUCLEAR STATION, UNIT 1

DOCKET NO. 50-269

1.0 INTRODUCTION

By application dated April 3, 2012, Agencywide Documents Access and Management System (ADAMS) Accession No. ML12097A248, Duke Energy Carolinas, LLC (Duke, or the licensee), requested changes to the Technical Specifications (TSs) for the Oconee Nuclear Station, Unit 1 (ONS 1).

The proposed change would allow for a one-time extension to the once per ten years frequency of the ONS 1 reactor containment building leakage rate test (i.e., the Type A test, also known as the integrated leak rate test (ILRT), per Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, Appendix J, "Primary Reactor Containment Leakage Testing For Water-Cooled Power Reactors"). This test is required by Technical Specification (TS) 5.5.2 "Containment Leakage Rate Testing Program." The proposed change would permit the existing ILRT frequency to be extended from ten years to approximately 11.25 years. The proposed change would avoid the necessity of performing a Type A test 13 months prior to the 10th year anniversary of the completion of the last Type A test, which was completed on December 8, 2003. If granted, this revision would extend the period from 120 months (10 years) to no longer than 135 months between successive tests. In terms of refueling outages, this extension would move the performance of the next ILRT from the fall 2012 refueling outage (1EOC27) to the fall 2014 refueling outage (1EOC28).

2.0 REGULATORY EVALUATION

The primary reactor containment, which at ONS 1 is called the reactor building, is the structure that encloses the components of the reactor coolant pressure boundary and serves as an essentially leak-tight barrier against the uncontrolled release of radioactivity to the environment. As described in ONS 1's Updated Final Safety Analysis Report (UFSAR) section 3.8 and section 6.2, the containment is designed to permit leakage monitoring during the life of the plant. A program of testing and surveillance of the reactor building has been developed to provide assurance, during service, of the capability of each containment system to perform its intended safety function. This program consists of the following tests: (1) overall integrated leak rate tests of the reactor building; (2) local leak detection tests of components having resilient seals, gaskets, or sealant compounds that penetrate or seal the boundary of the containment system; (3) local

leak detection and operability tests of containment isolation valves in systems that vent directly to the reactor building atmosphere or the reactor coolant system that must close upon receiving an isolation signal and seal the containment under accident conditions; and (4) operability tests of engineered safeguards systems which under post accident conditions are relied upon to limit or reduce leakage from the containment.

Pursuant to 10 CFR 50.54(o), the primary containment is subject to the requirements of Appendix J to Part 50, "Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors." These test requirements provide for preoperational and periodic verification by tests of the leak-tight integrity of the primary reactor containment, and systems and components which penetrate containment of water-cooled power reactors, and establish the acceptance criteria for these tests. The purposes of the tests are to assure that (a) leakage through the primary reactor containment and systems and components penetrating primary containment shall not exceed allowable leakage rate values as specified in the technical specifications or associated bases; and (b) periodic surveillance of reactor containment penetrations and isolation valves is performed so that proper maintenance and repairs are made during the service life of the containment, and systems and components penetrating primary containment. Appendix J describes three types of tests. Type A tests are tests intended to measure the primary reactor containment overall integrated leakage rate. Type B tests are tests intended to detect local leaks and to measure leakage across pressure-containing or leakage-limiting boundaries such as containment penetrations whose design incorporates resilient seals, gaskets, or sealant compounds, including doors with resilient seals or gaskets. Type C tests are tests intended to measure leakage rates for containment isolation valves.

Appendix J contains two options, A and B, either of which can be chosen for meeting the requirements of Appendix J. Option A contains prescriptive requirements that must be met at fixed intervals. Option B contains performance-based requirements, is less prescriptive, utilizes risk-based insights, and allows the licensee the flexibility to adopt cost-effective methods, including setting test intervals, for implementing the safety objectives underlying the requirements of Appendix J.

On October 30, 1996, the NRC issued a license amendment which approved the use of Option B for Type A tests for ONS 1. NRC Regulatory Guide (RG) 1.163 "Performance-Based Containment Leak-Rate Testing Program," dated September 1995 (ADAMS Accession No. ML003740058) was developed as a method acceptable to the NRC staff for implementing Option B. RG 1.163 states that the Nuclear Energy Institute (NEI) guidance document NEI 94-01, Revision 0, "Industry Guideline for Implementing Performance-Based Option of 10 CFR Part 50, Appendix J," provides methods acceptable to the NRC staff for complying with Option B with four exceptions, which are described in RG 1.163. Extensions in test intervals are allowed using the methods in Section 9, "Determining Performance-Based Test Interval For Type A Tests," of NEI 94-01, based upon two consecutive, periodic Type A tests and consideration of performance factors as described in Section 11.3, "Plant-Specific Testing Program Factors." Section 9.2.3 of NEI-94-01 still requires that Type A testing be performed at a frequency of at least once per 10 years based on acceptable performance history. However, NEI 94-01, Section 9.1, says that "Consistent with standard scheduling practices for Technical Specifications Required Surveillances, intervals for recommended Type A testing given in this section may be extended by up to 15 months. This option should be used only in cases where refueling schedules have been changed to accommodate other factors."

10 CFR Part 50, Appendix J, Option B, "Performance Based Requirements," states that a Type A test shall be conducted at a periodic interval based on historical performance of the overall containment system. ONS 1 TS 5.5.2 requires that leakage rate testing be performed as required by 10 CFR Part 50, Appendix J, Option B, as modified by approved exemptions, and in accordance with the guidelines contained in RG 1.163.

A Type A test is an overall ILRT of the containment structure. NEI 94-01, Revision 0, specifies an initial test interval of 48 months, but allows an extended interval of 10 years, based upon two consecutive successful tests. As described earlier, there is also a provision for extending the test interval an additional 15 months, but this "should be used only in cases where refueling schedules have been changed to accommodate other factors." The most recent two Type A tests at ONS 1 have been successful, so the current interval requirement is 10 years.

The last ONS 1 ILRT was completed on December 8, 2003. The next ILRT, per TS 5.5.2, is required to be performed no later than December 8, 2013. The next ONS 1 refueling outage (1EOC27) is scheduled for fall of 2012. Therefore, the next Type A test would have to be performed thirteen months less than 10 years after the most recent one, because the following refueling outage (1EOC28) would be approximately 131 months after the most recent Type A test, and the additional 15 month extension allowed by NEI 94-01 does not apply. Thus, the licensee is requesting a TS change to add 15 months to the test interval.

The proposed TS change does not involve any other changes to licensing commitments or acceptance criteria.

As additional background, the NRC staff has issued licensing amendments to a significant number of reactor units which extended, on a one-time basis, their Type A test intervals to 15 years, based primarily on probabilistic risk assessment arguments. The licensee's proposed request for ONS 1 is also on a one-time basis, but only increases the Type A test interval to 135 months (11.25 years). The licensee cited Nine Mile Point Nuclear Station (NMPNS) Unit 1, Vermont Yankee Nuclear Power Station (VYNPS), Arkansas Nuclear One (ANO) Unit No. 2, and Palisades Nuclear Plant, as precedents in obtaining NRC approval of license amendment requests similar to the one proposed for ONS 1.

3.0 TECHNICAL EVALUATION

3.1 Method of Review

The NRC staff compared the licensee's proposal against regulatory criteria.

3.2 Technical Specifications Change

The current TS 5.5.2 "Containment Leakage Rate Testing Program," reads as follows:

A program shall establish the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines of Regulatory Guide 1.163, "Performance-Based Containment Leakage-Rate

Program," dated September 1995. Containment system visual examinations required by Regulatory Guide 1.163, Regulatory Position C.3, shall be performed as follows:...

The licensee's proposed request would modify TS 5.5.2 as follows:

A program shall establish the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. **The next Unit 1 ILRT following the December 8, 2003 test shall be performed no later than March 8, 2015.** This program shall be in accordance with the guidelines of Regulatory Guide 1.163, "Performance-Based Containment Leakage-Rate Program," dated September 1995. Containment system visual examinations required by Regulatory Guide 1.163, Regulatory Position C.3, shall be performed as follows:...

The NRC staff has issued one exemption to ONS 1 for 10 CFR 50, Appendix J, that is still valid. That exemption allows the licensee to pressurize the door seals on the reactor building personnel air lock for the leakage test, rather than pressurizing the entire air lock. See ADAMS Accession No. ML003670061.

3.3 Containment Leak-Tight Integrity Considerations

The reactor containment leakage test program requires the licensee to perform ILRT, also called a Type A test, and local leakage tests (LLRTs) known as Type B and Type C tests. The Type A test measures the overall leakage rate of the primary reactor containment. Type B tests are primarily intended to detect leakage paths and measure leakage rates for primary reactor containment penetrations. Type C tests are intended to measure containment isolation valve leakage.

The ONS 1 TS 5.5.2 currently requires that the licensee follow Option B of Appendix J, in accordance with the guidelines in RG 1.163. The licensee stated that this will require the next Type A test to be within 10 years after the last Type A test, which was performed on December 8, 2003. The licensee has requested an extension of the next Type A test interval not to exceed 15 months.

As discussed previously, NEI 94-01, Revision 0, allows an additional 15 months to be added on to the 10-year interval at the discretion of the licensee, but with the restriction that it "should be used only in cases where refueling schedules have been changed to accommodate other factors." The purpose of this restriction is to prevent a licensee from arbitrarily adding the 15 months on to every testing interval, which would effectively change the interval permanently to 135 months. This extension does not apply to ONS 1 in this case, since the licensee would have the ability to perform the Type A test during the Fall 2012 refueling outage. However, the use of a 15 month extension was endorsed by the NRC staff through the issuance of RG 1.163 and its endorsement of the guidance in NEI 94-01, Revision 0.

The proposed revision would avoid the necessity of performing a Type A test 13 months prior to the 10th year anniversary of the completion of the last Type A test on December 8, 2003, and would also extend the period from 120 months (10 years) to no longer than 135 months between

successive tests. In terms of refueling outages, this extension would move the performance of the next ILRT from ONS 1 outage #27 (1EOC27) to ONS 1 outage #28 (1EOC28).

The leak-tight integrity of the penetrations and isolation valves are verified through Type B and Type C LLRTs and the overall leak-tight integrity and structural integrity of the primary containment is verified through a Type A test (ILRT) as required by 10 CFR 50, Appendix J. These tests are performed at the design-basis accident pressure. The testing frequency for Type B and Type C tests is not affected by the proposed amendment and will continue to be performed in accordance with NEI 94-01, Revision 0, as endorsed by RG 1.163.

The NRC staff concludes that the licensee's program for measuring containment leakage is being conducted in accordance with the requirements of TS 5.5.2. The regulatory criteria is to perform the next Type A test by December 8, 2013. However, the NRC staff finds that the results of the licensee's containment leakage test program supports a 15 month extension.

3.4 Containment Pressure Boundary Evaluation

The first ONS 1 ILRT was performed on August 2, 1971. The last ONS 1 ILRT was completed on December 8, 2003. In between, there were six other ILRTs performed. The results of previous ILRT testing has confirmed that the ONS 1 containment structure leakage is acceptable, with considerable margin by meeting the TS acceptance criterion of 0.2% of containment air weight at the design basis loss of coolant accident pressure (L_a). The last ILRT was completed on December 8, 2003, after the installation of the replacement steam generators and closure of the construction opening made in the containment structure to support the replacement of the steam generators. There have been no failed ILRTs at ONS 1.

No modifications that require a Type A test are planned prior to 1EOC28 refueling outage, when the next Type A test will be performed under this proposed change. Any modifications to the containment that affect the containment leakage integrity prior to the next scheduled Type A test would be subject to the special testing requirements of NEI 94-01, Section 9.2.4, "Containment Repairs and Modifications."

There have been no pressure or temperature excursions in the containment which could have adversely affected containment integrity. There is no anticipated addition or removal of plant hardware within the primary containment which could affect leak-tightness.

Containment penetration (Type B and C) testing is being performed in accordance with Option B of 10 CFR 50, Appendix J at the frequency required by the TSs. The licensee stated that a review of the Type B and Type C test results from the December 2003 through June 2011 and their comparison with the allowable leakage rate was performed. The total penetration leakage on a minimum path basis was less than 11.72% of the leakage acceptance criteria.

ONS 1 has established procedures for performing visual examination of the accessible surfaces of the containment for detection of structural problems. RG 1.163, Regulatory Position C.3, specifies that these examinations should be conducted prior to initiating a Type A test and during two other outages before the next Type A test if the interval for the Type A test has been extended to 10 years, in order to allow for early detection of evidence of structural deterioration. These visual examinations have been completed, with no significant defects noted to date.

It is noted that a visual inspection is also conducted in accordance with the Containment Inservice Inspection requirements, per 10 CFR 50.55a(b)(2), in accordance with Subsection IWE of Section XI of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code.

The NRC staff finds that a Type A test has been done as required following repairs and modifications to the containment that affect the containment leakage integrity, and that there are no such modifications planned prior to the next planned Type A test. Therefore, the results of the staff's containment pressure boundary evaluation support a 15 month extension.

3.5 Conclusion

Based on the foregoing evaluation, the NRC staff finds that there are no significant increases in risk or reductions in safety resulting from the requested test extension, beyond those already considered in the establishment of the intervals allowed by RG 1.163 and NEI 94-01, Revision 0. Further, the ONS 1 containment has a good recent leakage rate history. Therefore, the NRC staff concludes that the requested TS change, increasing the Type A test interval one time to 135 months, is acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the South Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding, which was published in the *Federal Register* on July 10, 2012, 77 FR 40651. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor(s): Brian E. Lee
Dan Hoang
John Boska

Date: October 1, 2012

October 1, 2012

Mr. Preston Gillespie
Site Vice President
Oconee Nuclear Station
Duke Energy Carolinas, LLC
7800 Rochester Highway
Seneca, SC 29672-0752

SUBJECT: OCONEE NUCLEAR STATION, UNIT 1, ISSUANCE OF AMENDMENT
REGARDING EXTENSION OF THE REACTOR BUILDING INTEGRATED LEAK
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A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

If you have any questions, please call me at 301-415-2901.

Sincerely,

/RA/

John P. Boska, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-269

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