



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

September 5, 2012

Vice President, Operations  
Entergy Nuclear Operations, Inc.  
Indian Point Energy Center  
450 Broadway, GSB  
P.O. Box 249  
Buchanan, NY 10511-0249

SUBJECT: ACCEPTANCE REVIEW FOR AMENDMENT REGARDING ATMOSPHERIC  
DUMP VALVES - INDIAN POINT NUCLEAR GENERATING UNIT NO. 3 - (TAC  
NO. ME8750)

Dear Sir or Madam:

By letter dated May 23, 2012, Entergy Nuclear Operations, Inc., the licensee, submitted a license amendment request for Indian Point Nuclear Generating Unit No. 3. The proposed amendment would revise the limiting condition for operation (LCO) for Technical Specification (TS) 3.7.4, "Atmospheric Dump Valves (ADVs)," to require that all four ADVs be operable. The application states that the current LCO, which only requires three ADVs to be operable, is non-conservative.

The purpose of this letter is to provide the results of the U.S. Nuclear Regulatory Commission (NRC) staff's acceptance review of this amendment request. The acceptance review was performed to determine if there is sufficient technical information in scope and depth to allow the NRC staff to complete its detailed technical review. The acceptance review is also intended to identify whether the application has any readily apparent information insufficiencies in its characterization of the regulatory requirements or the licensing basis of the plant.

Consistent with Section 50.90 of Title 10 of the *Code of Federal Regulations* (10 CFR), an amendment to the license (including the technical specifications) must fully describe the changes requested, and following as far as applicable, the form prescribed for original applications. Section 50.34 of 10 CFR addresses the content of technical information required. This section stipulates that the submittal address the design and operating characteristics, unusual or novel design features, and principal safety considerations.

The NRC staff reviewed your application and concluded that it did not provide technical information in sufficient detail to enable the NRC staff to complete its detailed review. This informational need was conveyed to you by e-mail dated July 2, 2012, followed by letter dated July 19, 2012. In this letter, the NRC staff identified that the following information was needed to complete its technical review:

1. The atmospheric dump valves (ADVs) are credited in the steam generator tube rupture (SGTR) analysis. The proposed revision to TS Bases Section 3.7.4 states that no failure of a second ADV is postulated. In addition, the SGTR analysis in both the Indian Point Unit 3 Updated Final Safety Analysis and the NRC's safety evaluation for the 4.85 percent stretch power uprate dated March 24, 2005, did not address single active

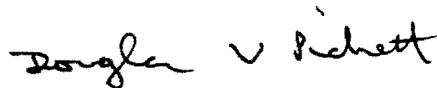
failures of ADVs. Please provide the licensing basis for not assuming a single active failure for the SGTR analysis.

2. The proposed revision to TS Bases 3.7.4 describes how 4 ADVs must be OPERABLE to ensure that at least 3 ADVs are available to conduct a plant cooldown following a SGTR. Please confirm the number of ADVs required for plant cooldown in the analysis of record, and discuss whether it is possible to cool the plant down safely following a SGTR with fewer ADVs.
3. Revision 2 to Indian Point emergency operating procedure 3-E-3 (page 12 of 56) describes available operator actions to vent the faulted steam generator to the atmosphere. While this version of the emergency operating procedure may be out of date, please clarify the conditions under which plant operators would vent the faulted steam generator to atmosphere and whether such actions would be bounded by the dose analysis of record.

By letter dated August 3, 2012, you provided a supplement to this submittal. The NRC staff has reviewed your application, as supplemented, and concluded that the proposed change is administrative in nature because the proposed revision improves alignment between the TSs and the specific Indian Point 3 technical analysis and is more conservative than the existing specification. The staff, therefore, concludes that sufficient information has been provided to permit the staff to initiate its review. You will be advised of any further information needed to support the NRC staff's more detailed licensing review by separate correspondence.

If you have any questions, please feel free to contact me at 301-415-1364.

Sincerely,



Douglas V. Pickett, Senior Project Manager  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-286

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/ra/

Douglas V. Pickett, Senior Project Manager  
Plant Licensing Branch I-1  
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DATED: September 5, 2012

SUBJECT: ACCEPTANCE REVIEW FOR FACILITY OPERATING LICENSE NO. DPR-64  
INDIAN POINT UNIT 3

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