



NextEra Energy Seabrook, LLC
10 CFR 72.48 Report of Changes, Tests, and Experiments
August 8, 2012

Docket No. 50-443

Docket No. 72-63

SBK-L-12156

ATTN: Document Control Desk
Director, Spent Fuel Project Office,
Office of Nuclear Material Safety and Safeguards
Nuclear Regulatory Commission
Washington, DC 20555-0001

Seabrook Station
10 CFR 72.48 Report of Changes, Tests, and Experiments

10 CFR 72.48(d)(2) requires that a report containing a brief description of changes, tests, and experiments, including a summary of the evaluation of each be submitted at intervals not to exceed 24 months. The enclosure provides a summary report of changes, tests, and experiments performed during the time period of August 1, 2010 through July 31, 2012.

Should you have any questions regarding this submittal, please contact me at (603) 773-7745.

Sincerely,

NextEra Energy Seabrook, LLC

Michael O'Keefe
Licensing Manager

cc:
Regional Administrator, Region I
Project Manager, Project Directorate I-2
Seabrook Senior Resident Inspector

NM5526

Enclosure to SBK-L-12156

Summary Report of Facility Changes, Tests, and Experiments
Completed in Accordance with the Requirements of 10 CFR 72.48
for the Time Period August 1, 2010 through July 31, 2012

10CFR72.48 Evaluation 12-001

Revision to 10 CFR 72.212 Evaluation
Report Dry Fuel Storage Manual
(DFSM) Chapter 2

Summary Description and Purpose: This activity revised the 10 CFR 72.212 Evaluation Report (Dry Fuel Storage Manual (DFSM) Chapter 2) to ensure the loading conditions under the initial Certificate of Compliance (CoC) did not affect the ability of the previously loaded casks to meet the storage or unloading requirements of the newer CoC amendment and to change the 10 CFR 72 licensing basis to NUHOMS® HD System Certificate of Compliance 72-1030, Amendment 1 and NUHOM® HD System Updated Final Safety Analysis Report (UFSAR) 72-1030, Revision 3 for the previously loaded casks and assembled NUHOMS® HSM-H array. Additionally, the revision corrected a typographical error related to the DFS facility pad design.

Evaluation Summary: The 10 CFR 72.48 evaluation performed for this change concluded there was no increase to the probability or consequence of any accident or malfunction, nor is the possibility of a different accident or malfunction created. There is no impact to any design basis limit for a fission product barrier and no evaluation methodology is involved. The evaluation determined that the revision to the 10 CFR 72.212 report could be implemented without prior NRC approval.