Dennis R. Madison Vice President - Hatch Southern Nuclear Operating Company, Inc. Plant Edwin I. Hatch 11028 Hatch Parkway North Baxley, Georgia 31513

Tel 912.537.5859 Fax 912.366.2077



0

Docket Nos.: 50-321

NL-12-1662

SOUTHER

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D. C. 20555-0001

> Edwin I. Hatch Nuclear Plant Licensee Event Report 2012-002-01 <u>Failure of 1C EDG Output Breaker to Close</u> <u>Results in Condition Prohibited by Technical Specifications</u>

Ladies and Gentlemen:

In accordance with the requirements of 10CFR50.73(a)(2)(i)(B), Southern Nuclear Operating Company hereby submits the enclosed revised Licensee Event Report concerning an event of non-compliance with Technical Specification 3.8.1 for the failure of the 1C emergency diesel generator (EDG) output breaker to close when tested during a plant shutdown for refueling.

This letter contains no commitments to the NRC. If you have any questions, please contact Mr. B. D. McKinney at (205) 992-5982.

Respectfully submitted,

Jenno Marchin

D. R. Madison Vice President – Hatch

DRM/SBT/msc

Enclosure: LER 2012-002-01

 cc: Southern Nuclear Operating Company Mr. S. E. Kuczynski, Chairman, President & CEO Mr. D. G. Bost, Executive Vice President & Chief Nuclear Officer Mr. D. R. Madison, Vice President – Hatch Mr. B. L. Ivey, Vice President – Regulatory Affairs Mr. B. J. Adams, Vice President – Fleet Operations RTYPE: CHA02.004

U. S. Nuclear Regulatory Commission Mr. V. M. McCree, Regional Administrator Mr. P. G. Boyle, NRR Senior Project Manager– Hatch Mr. E. D. Morris, Senior Resident Inspector – Hatch

Enclosure

NL-12-1662

Edwin I. Hatch Nuclear Plant - Unit 1

Licensee Event Report 2012-002-01

Failure of 1C EDG Output Breaker to Close Results in Condition Prohibited by Technical Specifications

NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION (9-2007)				APPROVED BY OMB: NO. 3150-0104 EXPIRES: 10/31/2013									
LICENSEE EVENT REPORT (LER)					Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects.resources@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to resoond to. the information collection.								
	1. FACILITY NAME Edwin I. Hatch Nuclear Plant Unit 1							2. DOCKET NUMBER 3 05000 321			3. PAGE 1 OF 7		
4. TITLE	I. TITLE												
	Failure of 1C EDG Output Breaker to Close Results in Condition Prohibited by Technical Specifications												
5. EVENT D	DATE	6. L	ER NUM		7. R	EPORT D	ATE	FACIL	8.	OTHER FAC	ILITIES INVOLVED		
MONTH DAY	YEAR	YEAR	SEQUEN NUMBE		MONTH	DAY	YEAR						
03 10	2012	2012	- 002	- 01	08	16	2012		ITY NAME			DOCKET	NUMBER
9. OPERATING	MODE	11.	THIS RE	PORTIS	SUBMITTE	ED PURSI	JANT T	D THE	REQUIREM	ENTS OF 10	CFR§: (Chec	k all that a	apply)
$\begin{array}{c c} 4 & \begin{array}{ c c c c c } 20.2201(b) \\ 20.2201(d) \\ 20.2203(a)(1) \\ 20.2203(a)(2)(i) \\ 20.2203(a)(2)(i) \\ 20.2203(a)(2)(ii) \\ 20.2203(a)(2)(ii) \\ 20.2203(a)(2)(iv) \\ 20.2203(a)(2)(v) \\ 20.2203(a)(v) \\ 20.2203(a)(v) \\ 20.2203(a)(v) \\ 20.2203(a)(v) \\ 20.2203(a)(v) \\ 20.2203(a)(v$		(i) (ii) (iii) (iv) (v)	$\begin{array}{ c c c c c c c c c c c c c c c c c c c$		$ \begin{bmatrix} 50.73(a)(2)(i)(C) \\ 50.73(a)(2)(ii)(A) \\ 50.73(a)(2)(ii)(B) \\ 50.73(a)(2)(iii) \\ 50.73(a)(2)(v)(A) \\ 50.73(a)(2)(v)(A) \\ 50.73(a)(2)(v)(B) \\ 50.73(a)(2)(v)(C) \\ 50.73(a)(2)(v)(D) \\ \end{bmatrix} $		<ul> <li>50.73(a)(2)(vii)</li> <li>50.73(a)(2)(viii)(A)</li> <li>50.73(a)(2)(viii)(B)</li> <li>50.73(a)(2)(ix)(A)</li> <li>50.73(a)(2)(x)</li> <li>73.71(a)(4)</li> <li>73.71(a)(4)</li> <li>73.71(a)(5)</li> <li>OTHER</li> <li>Specify in Abstract below or in NRC Form 366A</li> </ul>		)(A) )(B) A) ct below				
				1	2. LICENS	SEE CONT	ACT FO	OR THIS	BLER				
FACILITY NAME Edwin I. Hat	tch / Ste		<u> </u>					<u> </u>		91	2-537-588		ea Code)
		13. COM	PLETE C	NE LINE I	FOR EACH			AILUR	DESCRIB	ED IN THIS F	REPORT		
CAUSE	SYSTEM	COMPO	F.	MANU- ACTURER	TOE	TABLE EPIX	C/	USE	SYSTEM	COMPONEN	FACTURER		ORTABLE DEPIX
В	EK	BKF		W120	Ye								
YES (If yes					T EXPECT		$\boxtimes$	NO	SUB	XPECTED MISSION DATE	MONTH	DAY	YEAR
□ YES (If yes, complete 15. EXPECTED SUBMISSION DATE)       □ NO       DATE         ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)       During the performance of the 1C emergency diesel generator (EDG) loss of offsite power (LOSP) logic system functional test (LSFT), on March 10, 2012, at 2350 EST, the EDG output breaker failed to close and supply power to the 1G 4kV bus. This failure resulted in the inability to energize the 1G bus and the safety-related buses fed by this bus. The operating crew attempted to restore normal power to the bus, but was unsuccessful. A DC ground indication was also received when the 1C EDG output breaker failed to close. Troubleshooting revealed a connecting screw on the circuit breaker auxiliary switch making contact between terminals 8 and 10, creating a fault between the positive and negative portions of the DC circuit when the LOSP test permissive was applied to the closing circuit for the 1C EDG output breaker. This short prevented the output breaker closing coil from functioning as required.         The direct cause for the failure of the breaker to close is attributed to an apparent vendor quality issue associated with the alternate supply breaker for the 1G emergency bus that occurred at the vendor facility during the manufacturing/assembly process. The installation of a screw on an auxiliary switch termination that penetrated the insulation of an adjacent lug created a short circuit condition that could only be manifested during an actual or simulated LOSP condition. The breaker was replaced and testing performed to confirm the output breaker "close" permissive functioned as required. The condition was reviewed for applicability to 10 CFR 21 and determined to not be reportable.													

 
 NRC FORM 366A (10-2010)
 LICENSEE EVENT REPORT (LER)
 U.S. NUCLEAR REGULATORY COMMISSION

 CONTINUATION SHEET
 1. FACILITY NAME
 2. DOCKET
 6. LER NUMBER
 3. PAGE

	Z. DUCKET	0. LER NUMBER			3. PAGE		
Edwin I. Hatch Nuclear Plant Unit 1	05000321	YEAR	YEAR SEQUENTIAL NUMBER		2	OF	
	00000021	2012	- 002 -	01		01	1

NARRATIVE

## PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor Energy Industry Identification System codes appear in the text as (EIIS Code XX).

## DESCRIPTION OF EVENT

During the performance of the 1C emergency diesel generator (EDG)(DG) loss of offsite power (LOSP) logic system functional test (LSFT), on March 10, 2012, at approximately 2350 EST, with Unit 1 in a refueling outage in cold shutdown (Mode 4), the EDG output breaker (EK) failed to close and supply power to the 1G 4kV bus. This failure resulted in the inability to energize the 1G bus and the safetyrelated loads fed by this bus. The operating crew attempted to restore normal power to the bus, but was unsuccessful. A DC ground indication was also received when the 1C EDG output breaker failed to close. There was an indicated ground on the positive leg of the 125 VDC battery system associated with this diesel generator. Initial attempts to isolate the ground were unsuccessful until the ground indication cleared during tagout of the circuit containing the undervoltage relay (EK) approximately 4 hours later. Additionally, the 125 VDC control power breaker for the normal and alternate supply breakers to the 1G 4kV bus was also opened during the same interval in which the ground cleared. An undervoltage relay (27GDX) in the closing circuit for the 1C EDG output breaker was subsequently determined to have experienced arcing during the breaker closure failure. Later it was determined that this short damaged the 1C EDG undervoltage relay in addition to preventing the closure of the output breaker. Subsequent to the "breaker close" failure, attempts to re-energize 4kV bus 1G from the normal supply breaker were unsuccessful with that breaker closing and then immediately tripping. The nature of the failure of the 1C EDG output breaker to close was determined to be limited to the condition that resulted in a grounded condition as a result of the relay failure which was subsequent to the electrical short that was created by a fault between the DC positive and negative when the LOSP test permissive was applied to the closing circuit for the 1C EDG output breaker.

## CAUSE OF EVENT

The direct cause for the failure of the 1C EDG output breaker to close is attributed to an apparent vendor quality issue associated with the 1G 4kV bus alternate supply breaker that occurred at the vendor facility during the manufacturing/assembly process. The 1C EDG output breaker's LOSP permissive has a logic tie through the affected 1G 4kv bus alternate supply breaker's auxiliary switch that contained the quality issue. Troubleshooting revealed a connecting screw on the 1G 4kV bus alternate supply breaker auxiliary switch making contact between terminals 8 and 10, creating a short between the DC positive and negative when the LOSP test permissive was applied to the closing circuit for the 1C EDG output breaker. This condition prevented the closing coil from functioning properly. The apparent manufacturing/assembly error involved the installation of a screw on an auxiliary switch termination of the 1G 4kV bus alternate supply breaker by the vendor that penetrated the insulation of an adjacent lug, thereby creating the noted short circuit condition that could only be manifested during an actual or simulated LOSP condition.

The root cause investigation concluded that the root cause of the "as found" condition was attributed to inadequate vendor testing/inspection and design (associated with the use of 5/16" length termination screws). The vendor had no inspection criteria in place to ensure that wires are formed correctly or that air gaps exist between lugs/wires in close proximity to each other. The discovered latent fault was determined to have originated at the original equipment manufacturer (OEM) during assembly of the 1G 4kV bus alternate supply breaker based upon the apparent stress found on the lug and the fact that there was no prior Hatch work history associated with the auxiliary switch on this breaker.

The root cause investigation also reviewed the plant's receipt inspection, preventive maintenance (PM),

NRC FORM 366A (10-2010) LICENSEE EVENT REPORT (LER) U.S. NUCLEAR REGULATORY COMMISSION CONTINUATION SHEET							
1. FACILITY NAME	2. DOCKET	6. LER NUMBER 3. PAGE					
Edwin I. Hatch Nuclear Plant Unit 1	05000321	YEAR SEQUENTIAL REVISION NUMBER NUMBER 3 OF	7				
		2012 - 002 - 01	'				
they could have played, if any, in identify assessment of the affected auxiliary swit improper connections or frayed leads ex- would move to their required position cal auxiliary switch mounted on the breaker perform the inspection due to its physica access to subcomponents is limited in th PM was determined to be adequate bass received at the plant. It should also be in damaged could not be seen until the scri- out/removed. Such a condition would not The root cause investigation also consid would perform its intended function where function of the breakers installed in the ef- performance of an LSFT which tests the and that the breaker subcomponents fun- components to go to their assumed end in 2010 with another breaker used as the LSFT confirmed that the control circuits and the alternate supply breaker contain the alternate supply breaker components to conditions are present requiring their act breaker was installed, Operations person Having previously successfully complete breaker (this LSFT tests the logic and co successful completion of the breaker PM would function as required with respect to cause investigation concluded that the a breaker would perform its intended funct order to recognize that further testing we vendor quality issue for this single break foreknowledge is not reasonable. Additi Plant Hatch is the same tests and inspec- nuclear stations with similar 4kV vacuum not include additional testing or checks to Plant Hatch. A review of Hatch procedu Post Maintenance Testing Good Practic the plant or industry guidance document breaker be tested upon installation or re a relay or a breaker is removed with no the component in its cubicle once appro- been verified. Changing testing method replaced component without the right co place the plant at undue risk.	ring the "as four ch for damage of isted. It also ind led upon. Thes in such a way the location on the is way. The gue ed on the inform obted that the date of reasonably be ered the adequate of the LOSP performed the LOSP performed of the LOSP performed of the LOSP performed of the LOSP performed to the LOSP performed of the LOSP performed	e expected for a new breaker. acy of the PMT to ensure that the equipment ervice following maintenance activities. The safety switchgear is accomplished by the successful hat ensures the breakers function when required the related logic string(s) to actuate to cause FT was successfully performed during the outage lternate supply breaker during that test. This installed alternate supply breaker and associated owing performance of the planned PM on the1G vendor quality issue, the breaker was installed in letion of the breaker PM ensured the breaker and e supply breaker would function when the mally, once the 1G 4kV bus alternate supply at the breaker would open and close under load. sociated with the 1G 4kV bus alternate supply reconnected with the alternate supply breaker, the open resured that the alternate supply breaker. The roof T to ensure that the 1G 4kV bus alternate supply ned to service was successfully completed. In preknowledge of this apparent isolated latent	t				

In summary the direct cause of the 1C EDG output breaker failing to close upon demand was attributed to an apparent vendor quality issue associated with an auxiliary switch attached to the 1G 4kV bus alternate supply breaker that occurred at the vendor facility during the manufacturing/assembly process. The root cause of the "as found" condition was attributed to inadequate vendor testing/inspection and the use of the 5/16" length termination screws in the design. The successful completion of the 1G 4kV bus alternate

NRC FORM 366A (10-2010) LICENSEE EVENT REPORT (LER) U.S. NUCLEAR REGULATORY COMMISSION CONTINUATION SHEET								
1. FACILITY NAME	2. DOCKET	6. LER NUMBER 3. PAGE						
Edwin I. Hatch Nuclear Plant Unit 1	05000321	YEAR SEQUENTIAL REVISION NUMBER NUMBER						
		2012 - 002 - 01						
testing to assure the intended safety func the known condition of the alternate supp limited nature of the apparent manufactu	ction of the affect oly breaker at the ring defect in the ndition could on	ciated LSFT provided adequate post maintenance cted components was not compromised based on e time it was installed. Due to the unique and e 1G 4kV bus alternate supply breaker auxiliary ly be manifested and thereby revealed during an <u>ENT</u>						
occurred and existed that was prohibited EDG to be operable during the preceding switch quality issue had a receipt inspect visual inspection and complete preventive included checks of the cam, links, contine PM was developed based on direction from performing similar PM and inspections in by the vendor quality issue, functional test did not identify the presence of the short manufacturing/assembly quality issue wat CFR 21 and determined to not be reported. The affected (later to be the1G 4kV bus at related 1E 4kV switchgear as the normal conditions. When installed in this breaked connected to DC negative and as a result difference in voltage. The LOSP LSFT for thereafter with no problems noted.	by the Technic operating cycl ion performed i e maintenance uity tests and re- om the vendor t their shop. Be sts consistent w circuit condition as present. This able per 10 CFF alternate supply supply breaker r location, the s t did not affect or the 1A EDG w	) breaker was initially installed in the safety , which is fed by the 1A EDG during LOSP shorted points within this cubicle logic are both the breaker function since there was no was performed March 2004 and every 24 months						
supply breaker until the Unit 1 2010 refue performed on March 11, 2010, the affect at that time. Following the PM on the bre switch, and performance of a hi-pot test of accordance with the normal functional test with no problems noted. The breaker con- then placed into the 1G 4kV switchgear a noted that the circuit containing the short and loading of the EDG that is performed affected logic string be in the circuit. Dur condition manifested itself when the short the 1C EDG output breaker from closing latent condition has existed since March the 1C EDG output breaker in the event of Unit 1 2012 refueling outage at a time wh the condition and associated timeline wa existed for a period of time greater than to The discovery of this event occurred dur 1C during the Unit 1 refueling outage dur required to be operable. The 1A and 1B its emergency bus. During the preceding	eling outage. Meed breaker was eaker that include on the auxiliary st requirements ntaining the app as the alternate ed connection was and re-energized the circuit was and re-energized 16, 2010, and wo of an LOSP cor- then the 1C EDC s performed, ar hat allowed by ing the perform ring which time EDGs were op g operating cycl 4 hours to 101 h	ance of routine TS surveillance testing of EDG the 1C EDG was not one of the EDGs erable at the time the 1C EDG failed to tie to e, the 1A EDG was inoperable on three nours; not considering the very brief periods of						

10-2010)		•	<b>:</b> R) 0.0.100	LCAR REG	OLATO		NRC FORM 366A (10-2010) LICENSEE EVENT REPORT (LER) U.S. NUCLEAR REGULATORY COMMISSION CONTINUATION SHEET							
1. FACILITY NAME	2. DOCKET		6. LER NUMBER			3. PAGE								
		YEAR	SEQUENTIAL NUMBER	REVISION		0.1710.2								
Edwin I. Hatch Nuclear Plant Unit 1	05000321	2012	- 002 -	01	5	OF	7							
ranging from approximately 8 hours to 12 inoperability during monthly surveillance to 1A and 1B EDGs concurrently inoperable fact that the 1C EDG could not tie to its e- were periods of time during the preceding inoperable. A review was performed to d from having periods of time when only on safety function during a design basis acci The methods and models used to analyze during the plant lifetime in the form of the results and consequences associated wit in the FSAR. The SAFER/GESTR-LOCA average planar heat generation rate at th concluded that the peak clad temperature cause perforation of the fuel cladding. As LOCA, and no fuel damage would result. cold shutdown condition and maintained low pressure coolant injection (LPCI) pun EDG provides power for the 'A' LPCI pur 1B EDG serves the 'C' and 'D' RHR LPC 'B' LPCI pump, the 'B' core spray pump, EDG can provide the needed low-pressur assumptions in the SAFER/GESTR-LOCA safety function on Unit 1 during the previo consequences of an accident and bring th was maintained.	testing. At no t mergency bus g operating cyc letermine what he diesel was o ident involving e the conseque SAFER/GEST th a LOCA usin A analyses were e most limiting e for the nomina s a result, no cl This analysis in that condition nps or one core np, the 'A' RHF I pumps and th and the 'B' and re pumps and I A analysis for L ous operating c	ime during ne 1A or 1E due to the l le when two the impact perable and a Loss Of ( ences of the R-LOCA and g realistic e e performed power and al or expect adding performed on a long e spray pun SW pump, e 'C' RHRS 'D' RHRSV put 1. For sycle as the	he very brief p the operating BEDG inopera latent equipme o of the three L on nuclear saf d available to p Coolant Accide ELOCA/LOSP nalyses. Thes evaluation meth d with a bound exposure com ted case would forations would that the reacto term basis with p and one LP and the 'A' co SW pump. The N pumps. Eith imp(s) to satisf this reason, th ability to mitig	eriods of cycle wer ble, and ant proble Jnit 1 ED fety would perform th ent (LOCA have bee e analyse hods as d ing maxim bination a d be insuff d be cause r can be th h either tw CI pump. re spray p e 1C EDG her the 1A y the min ere was r ate the	given t m, ther Gs wer I have e requ )/LOSI n refin s prov ocume num and ficient f ed by t prough vo RHI The 1 pump. serve: or 1B imum no loss	he re been ired P. ed ide ented to he t to a R A The s the of								
The 1G 4kV bus also provides normal em which contains the 'B' LPCI injection valv the Unit 2 'A' RHR LPCI valve load cente emergency power supply for the Unit 2 'A kV bus which requires a manual alignment During the previous operating cycle, the approximately 34 hours to 101 hours not monthly surveillance testing. The 1A ED provides the normal emergency power to removing the 1A EDG from service for pla 4kV bus to be realigned to provide the alt load center. The same SAFER/GESTR-I combination of low pressure pumps in or bring the unit to cold shutdown and allow a DBA LOSP/LOCA on Unit 2 and an LO 2 core spray pumps remain operable and reactor to be safely shut down. With the normally be realigned to provide power to emergency power to the Unit 2 'A' LPCI I is out of service for maintenance and the the swing 1B EDG will remain dedicated conditions. Based on the nature of the di Operations personnel would identify this provide the needed direction to manual c	re. The 1E 4kV or that contains A' RHR LPCI van to use this point of the Unit 2 'A' Fanned mainten ternate emerge LOCA analysis der to automati is to be mainta SP on Unit 1 w d will recover U LOCA/LOSP of the Unit 2 2F oad center. Ho 1C EDG output to Unit 1 to ma irect cause for condition on the	bus provid the 'A' LPC leve load ce over source operable o every brief ergency po AHR LPCI ance, plant ncy power for Unit 2 a cally restor ined long te ith the 1A a nit 2 reacto ccurring on 4kV bus ar owever, in t it breaker w intain this u the 1C EDG,	les the normal cl injection valve inter is provide e for the desire n three occasion periods of inop wer to the 1E 4 valve load cent procedures re to the Unit 2 'A assumes the pro- e reactor vess erm in that con and 1C EDGs is r vessel invent of therefore rest he assumed ca vill fail to close unit in a safe co G output break and their proc	emergen ve. The a d by the l ed valve lo ons rangin perability 4kV bus th ter. Prior equired the V RHR LF resence o el invento dition. In inoperable tory and a ving EDG store alter ondition th . For these ondition di e failing the edures we	cy pow Iternate Jnit 2 2 bad cer ag from during nat to e Unit 2 PCI vali f the sa ry and the ca e, both llow th would nate ne 1A E se reas uring L to close build	er to e 2F 4 hter. n 2 2F ve ame to se of Unit e EDG cons .OSP e,								

	<b>FINUATION</b>				
1. FACILITY NAME	2. DOCKET		6. LER NUMBER		3. PAGE
dwin I. Hatch Nuclear Plant Unit 1	05000321	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	6 OF
		2012	- 002 -	01	
hours from the occurrence of the event. cooling flow path before reaching the co- into service. Once RHR shutdown coolin cold shutdown and maintained in that co During the previous operating cycle, whe one of these EDGs that would remain op 1. Additionally, based on the discussion	nditions necess ng has been pla ondition. en either the 1A perable and cap	ary to allow aced into se or 1B EDC able of per	v shutdown coo ervice the react G was inoperat forming its safe	bling to be for will be l ble there w ety function	placed prought to as always n on Unit
be no loss of safety function on Unit 2. I same conditions present the Unit 2 'A' R power source operable which also assur nature of the direct cause for the 1C ED provide the needed direction to allow clo thereby providing additional margin in th condition for either unit.	on both units wi In the event that HR LPCI load of res there is no lo G output breake osure of the 1C	then the 1A the swing enter woul oss of safe or failing to EDG output	EDG was inop 1B EDG was i d continue to h ty function on l close, Operation the breaker in a r	erable, the noperable ave its em Jnit 2. Bas ons procee natter of 1	ere would with the lergency sed on the dures to 2 hours
Based on the fact that the safety function impact this would have on Unit 2 should operating cycle, there was always one U operable on Unit 2 such that this event w the case the event was determined to be	the events des Jnit 1 EDG oper vould not result	cribed actu able and a in a loss of	ally occur durir dequate low pr f function on ei	ng the prev essure pu	<i>r</i> ious mps
CORRECTIVE ACTIONS					
The 1G 4kV bus alternate supply breake that had previously contained the appare confirm the 1C EDG output breaker "clos the Unit 1 emergency 4kV switchgear, tw seven spare breakers (not installed) hav manufacturing/assembly quality issue wa conclusion that the "as found" condition of are to inspect the remaining safety relate outages depending on the ability to remo and on the operating Mode of the units. vendor revealed no similar conditions or subsequent root cause evaluation, this of breaker failure to function.	ent latent manuf se" permissive f velve 4kV break e been inspecte as not present of of the auxiliary ed Unit 1 and 2 ove the breaker A search of ind failures in the i	acturing/as unctioned a ers on the ed. This in on these br switch was breakers in from servio lustry opera ndustry. B	ssembly quality as required. E Unit 2 emerge spection confir eakers, which o an isolated co o upcoming sys ce based on the ating experience ased on the ini	r issue was leven 4kV ncy switch med that the continues ndition. C tem outag e impact it e and con tial observ	s performed to breakers on gear and ne to support the urrent plans les or refueling would have tact with the ations and the
<ul> <li>Additional corrective actions that have b action program that include:</li> <li>Reviewing OEM manufacturing proce associated with manufactured vacuur</li> <li>Revising plant procedure to ensure ¼ are established to ensure air gaps ex</li> </ul>	edures / dedicat m breakers are 4 inch length te	ion plan to electrically rminal scre	ensure that ea isolated ews are used a	ch wire ter	mination
ADDITIONAL INFORMATION					
Other Systems Affected: None					

Failed Components Information: Master Parts List Number: 1R22-S007 Manufacturer: Westinghouse

EIIS System Code: EK Reportable to EPIX: Yes

NRC FORM 366A         LICENSEE EVENT REPORT (LER)         U.S. NUCLEAR REGULATORY COMMISSION           (10-2010)         CONTINUATION SHEET         U.S. NUCLEAR REGULATORY COMMISSION									
1. FACILITY NAME 2. DOCKET 6. LER NUMBER 3. PAGE									
Edwin I. Hatch Nuclear Plant Unit 1	05000321	YEAR SEQUENTIAL REVIS							
		2012 - 002 - 0	1						
Model Number: 50DHP-VR-250U       Root Cause Code: B         Type: Vacuum Breaker       EIIS Component Code: BKR         Manufacturer Code: W120       Commitment Information: This report does not create any new permanent licensing commitments.									
Previous Similar Events: None									