UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 22, 2012

Mr. George H. Gellrich, Vice President
Calvert Cliffs Nuclear Power Plant, LLC
Calvert Cliffs Nuclear Power Plant
1650 Calvert Cliffs Parkway
Lusby, MD 20657-4702

SUBJECT: CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2 - AMENDMENT REGARDING DIESEL GENERATOR SURVEILLANCE REQUIREMENT 3.8.1.11 (TAC NOS. ME6831 AND ME6832)

Dear Mr. Gellrich:

The Commission has issued the enclosed Amendment No. 302 to Renewed Facility Operating License No. DPR-53 and Amendment No. 279 to Renewed Facility Operating License No. DPR-69 for the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated August 8, 2011, as supplemented by letters dated January 11, May 7, and July 18, 2012.

These amendments would modify Technical Specification 3.8.1, "AC Sources – Operating," Surveillance Requirement (SR) 3.8.1.11 by revising the required power factor value to be achieved by the diesel generators (DGs) during conduct of the surveillance test. The proposed change would also modify the existing note in SR 3.8.1.11 to allow the DG to not achieve the required power factor if the grid conditions do not permit and the test is performed with DG synchronized with offsite power.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,

Nadiyah S. Morgan, Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-317 and 50-318

Enclosures:
1. Amendment No. 302 to DPR-53
2. Amendment No. 279 to DPR-69
3. Safety Evaluation

cc w/encls: Distribution via Listserv

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CALVERT CLIFFS NUCLEAR POWER PLANT, LLC

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 302
Renewed License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Calvert Cliffs Nuclear Power Plant, LLC (the licensee) dated August 8, 2011, as supplemented by letters dated January 11, May 7, and July 18, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2. of Renewed Facility Operating License No. DPR-53 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 302, are hereby incorporated into the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

George Wilson, Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the License and Technical Specifications

Date of Issuance: August 22, 2012

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CALVERT CLIFFS NUCLEAR POWER PLANT, LLC

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 279
Renewed License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Calvert Cliffs Nuclear Power Plant, LLC (the licensee) dated August 8, 2011, as supplemented by letters dated January 11, May 7, and July 18, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2. of Renewed Facility Operating License No. DPR-69 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 279, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

George Wilson, Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the License and Technical Specifications

Date of Issuance: August 22, 2012

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 302 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-53

AMENDMENT NO. 279 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-69

DOCKET NOS. 50-317 AND 50-318

Replace the following page of the Renewed Facility Operating License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove Page	Insert Page
3	3

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages	Insert Pages
3.8.1-13	3.8.1-13
3.8.1-14	3.8.1-14
3.8.1-15	3.8.1-15
3.8.1-16	3.8.1-16
---	3.8.1-17

rules, regulations, and orders of the Commission, now or hereafter applicable; and is subject to the additional conditions specified and incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady-state reactor core power levels not in excess of 2737 megawatts-thermal in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 302, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications.

(a) For Surveillance Requirements (SRs) that are new, in Amendment 227 to Facility Operating License No. DPR-53, the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment 227. For SRs that existed prior to Amendment 227, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the Surveillance was last performed prior to implementation of Amendment 227.

(3) Additional Conditions

The Additional Conditions contained in Appendix C as revised through Amendment No. 297 are hereby incorporated into this license. Calvert Cliffs Nuclear Power Plant, LLC shall operate the facility in accordance with the Additional Conditions.

(4) Secondary Water Chemistry Monitoring Program

The Calvert Cliffs Nuclear Power Plant, LLC, shall implement a secondary water chemistry monitoring program to inhibit steam generator tube degradation. This program shall include:

a. Identification of a sampling schedule for the critical parameters and control points for these parameters;

b. Identification of the procedures used to quantify parameters that are critical to control points;

C. This license is deemed to contain and is subject to the conditions set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act, and the rules, regulations, and orders of the Commission, now and hereafter applicable; and is subject to the additional conditions specified and incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor steady-state core power levels not in excess of 2737 megawatts-thermal in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 279 are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications.

(a) For Surveillance Requirements (SRs) that are new, in Amendment 201 to Facility Operating License No. DPR-69, the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment 201. For SRs that existed prior to Amendment 201, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the Surveillance was last performed prior to implementation of Amendment 201.

(3) Less Than Four Pump Operation

The licensee shall not operate the reactor at power levels in excess of five (5) percent of rated thermal power with less than four (4) reactor coolant pumps in operation. This condition shall remain in effect until the licensee has submitted safety analyses for less than four pump operation, and approval for such operation has been granted by the Commission by amendment of this license.

(4) Environmental Monitoring Program

If harmful effects or evidence of irreversible damage are detected by the biological monitoring program, hydrological monitoring program, and the radiological monitoring program specified in the Appendix B Technical Specifications, the licensee will provide to the staff a detailed analysis of the problem and a program of remedial action to be taken to eliminate or significantly reduce the detrimental effects or damage.

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.8.1.8	Verify interval between each sequenced load block is within ± 10% of design interval for the load sequencer.	31 days
SR 3.8.1.9	----------NOTE---------- All DG starts may be preceded by an engine prelube period. ---------- Verify each DG starts from standby condition and achieves, in ≤ 10 seconds, voltage > 4060 V and frequency > 58.8 Hz, and after steady state conditions are reached, maintains voltage ≥ 4060 V and ≤ 4400 V and frequency of > 58.8 Hz and ≤ 61.2 Hz.	184 days
SR 3.8.1.10	Verify manual transfer of AC power sources from the normal offsite circuit to the alternate offsite circuit.	24 months

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.8.1.11	----------NOTE---------- 1. Momentary transients outside the load and power factor limits do not invalidate this test. 2. If performed with the DG synchronized with offsite power, the surveillance test shall be performed at the required power factor. However, if grid conditions do not permit, the power factor limit is not required to be met. Under this condition, the power factor shall be maintained as close to the limit as practicable. ---------- Verify each DG, operating at a power factor of $\leq$ 0.84 for DG 1A and $\leq$ 0.83 for DGs 1B, 2A, and 2B, operates for $\geq$ 60 minutes while loaded to $\geq$ 4000 kW for DG 1A and $\geq$ 3000 kW for DGs 1B, 2A, and 2B.	24 months
SR 3.8.1.12	Verify each DG rejects a load $\geq$ 500 hp without tripping.	24 months

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.8.1.13	Verify that automatically bypassed DG trips are automatically bypassed on an actual or simulated required actuation signal.	24 months
SR 3.8.1.14	Verify each DG: a. Synchronizes with offsite power source while loaded upon a simulated restoration of offsite power; b. Manually transfers loads to offsite power source; and c. Returns to ready-to-load operation.	24 months

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.8.1.15	----------NOTE---------- All DG starts may be preceded by an engine prelube period. ----------	
	Verify on an actual or simulated loss of offsite power signal in conjunction with an actual or simulated Engineered Safety Feature actuation signal:	24 months
	a. De-energization of emergency buses;	
	b. Load shedding from emergency buses;	
	c. DG auto-starts from standby condition and:	
	1. energizes permanently connected loads in $\leq$ 10 seconds,	
	2. energizes auto-connected emergency loads through load sequencer,	
	3. maintains steady state voltage $\geq$ 4060 V and $\leq$ 4400 V,	
	4. maintains steady state frequency of $\geq$ 58.8 Hz and $\leq$ 61.2 Hz, and	
	5. supplies permanently connected and auto-connected emergency loads for $\geq$ 5 minutes.	

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.8.1.16	For the LCO 3.8.1.c AC electrical sources, SR 3.8.1.1, SR 3.8.1.2, SR 3.8.1.3, SR 3.8.1.5, SR 3.8.1.6, and SR 3.8.1.7 are required to be performed.	In accordance with applicable Surveillance Requirements

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 302 TO RENEWED

FACILITY OPERATING LICENSE NO. DPR-53

AND AMENDMENT NO. 279 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-69

CALVERT CLIFFS NUCLEAR POWER PLANT, LLC

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-317 AND 50-318

1.0 INTRODUCTION

By application dated August 8, 2011, as supplemented by letters dated January 11, May 7, and July 18, 2012 (Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML11222A020, ML12012A218, ML12132A086, and ML12202A013 respectively), the Calvert Cliffs Nuclear Power Plant, LLC (the licensee) submitted a request for changes to the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2 (Calvert Cliffs), Technical Specifications (TSs). The requested changes would modify TS 3.8.1, "AC Sources – Operating," Surveillance Requirement (SR) 3.8.1.11 by revising the required power factor value to be achieved by the diesel generators (DGs) during conduct of the surveillance test. The proposed change would also modify the existing note in SR 3.8.1.11 to allow the DG to not achieve the required power factor if the grid conditions do not permit and the test is performed with DG synchronized with offsite power.

The supplements dated January 11, May 7, and July 18, 2012, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the Nuclear Regulatory Commission (NRC or the Commission) staff's original proposed no significant hazards consideration determination as published in the Federal Register on November 29, 2011 (76 FR 73729).

2.0 REGULATORY EVALUATION

2.1 System Description

Calvert Cliffs has two Class 1E DGs for each unit. These DGs provide a dependable standby onsite electrical power source that is capable of supplying the essential electrical loads necessary to safely shutdown the plant and maintain it in a safe shutdown condition under all design bases accidents. Three of the four Class 1E DGs (DGs 1B, 2A and 2B) have a nominal continuous rating of 3000 kilo-Watt (kW), while the fourth DG (DG 1A) has a nominal continuous rating of 5400 kW. Each DG is aligned to a separate safety-related 4.16 kV electrical bus and is capable

of providing sufficient power for the required engineered safety features (ESFs) loads for that unit. A DG will start automatically on a safety injection signal or on a 4.16 kV degraded or undervoltage signal. After coming to rated speed and voltage, if the 4.16 kV breakers that supply offsite power to the electrical bus are open, the DG output breaker will automatically close onto the 4.16 kV electrical bus. The ESF electrical loads will then automatically sequence onto the DG in time to ensure vital functions are maintained in the event of a design basis accident.

2.2 Proposed TS Changes

SR 3.8.1.11 of TS 3.8.1 is required to be performed once every 24 months to provide verification that each DG can operate for a period of at least 60 minutes with an electrical load slightly greater than the calculated electrical loads experienced under worst case accident conditions. In order to demonstrate the ability of each DG to perform its safety function, SR 3.8.1.11 requires testing to be performed to a power factor value that reflects the design basis loading experienced during worst case accident conditions.

In the license amendment request (LAR), the licensee stated that during the review activities in connection with Temporary Instruction 2515/176, "Emergency Diesel Generator Technical Specification Surveillance Requirements Regarding Endurance and Margin Testing," the identified power factor value currently contained in SR 3.8.1.11 was inconsistent and less restrictive than the calculated design basis power factors corresponding to the calculated DG loadings, as noted in the following table:

Diesel Generator	SR 3.8.1.11 Current Power Factor Values	Design Basis Calculation Power Factor Values
DG 1A	0.85	0.84
DG 1B	0.85	0.83
DG 2A	0.85	0.83
DG 2B	0.85	0.83

The licensee stated that the power factor values listed above for the design basis calculations represent the most bounding power factor values for the various design basis events. The bounding power factor value calculation was based on the bounding maximum kW loading of each DG. For DG 1A, the maximum kW loading occurred during loss of offsite power (LOOP) without an accompanying accident and resulted in a power factor value of 0.84. For all other DGs (1B, 2A, and 2B), maximum kW loading occurred during a LOOP with an accompanying large break loss-of-coolant accident (LOCA) and resulted in a power factor value of 0.83. The LAR seeks to replace the current power factor value listed in SR 3.8.1.11 with the power factor values obtained in the design basis calculation.

The proposed TS change also seeks to modify the existing note in SR 3.8.1.11, which presently reads as follows: "Momentary transients outside the load and power factor limits do not invalidate this test." The revised note would also allow a DG to not achieve the required power factor value during performance of this SR when certain grid conditions would prohibit the DG from safely achieving the power factor limit. In this instance, the DG will be operated at a power factor as close to the limit as practicable. The revised note would read as follows:

NOTE

1. Momentary transients outside the load and power factor limits do not invalidate this test.

2. If performed with the DG synchronized with offsite power, the surveillance test shall be performed at the required power factor. However, if grid conditions do not permit, the power factor limit is not required to be met. Under this condition, the power factor shall be maintained as close to the limit as practicable.

In the LAR, the licensee provided an example of a situation when the required power factor value could not be safely achieved. The high voltage at the grid (offsite power source) or low load conditions can result in the emergency electrical bus voltage to be high. In this condition, with the DG synchronized to the grid, additional field excitation would be needed at DG to achieve the required power factor value which would result in the emergency bus voltage exceeding an upper acceptable voltage limit. In this situation, the tested power factor value would be maintained as close as possible to the required power factor value.

2.3 Regulatory Requirements and Guidance Documents

The construction permits for Calvert Cliffs were issued by the Atomic Energy Commission (AEC) on July 7, 1969, and the operating licenses were issued on July 31, 1974 for Unit No. 1 and August 13, 1976 for Unit No. 2. The AEC published the final rule that added 10 CFR Part 50, Appendix A, "General Design Criteria [GDC] for Nuclear Power Plants," in the *Federal Register* (36 FR 3255) on February 20, 1971, with the rule effective on May 21, 1971. As stated in SECY-92-223, dated September 18, 1992, the Commission decided not to apply the Appendix A GDC to plants with construction permits issued prior to May 21, 1971. The Calvert Cliffs Updated Final Safety Analysis Report (UFSAR) states that the plant was designed and constructed to meet the intent of the GDC published in July 1967. The plant GDC is discussed in the UFSAR Appendix 1C, "AEC Proposed General Design Criteria for Nuclear Power Plants."

GDC 17, "Electric power systems," of Appendix A, "General Design Criteria for Nuclear Power Plants," to Title 10 of the *Code of Federal Regulations* (CFR) Part 50 requires, in part, that nuclear power plants have onsite and offsite electric power systems to permit the functioning of structures, systems, and components (SSCs) that are important to safety. As stated in the LAR, Calvert Cliffs meets the requirements of GDC 17. Each DG is designed such that each onsite power source provides sufficient capacity and capability to permit the functioning of SSCs that are important to safety.

GDC 18, "Inspection and Testing of Electric Power Systems," requires, in part, that electric power systems important to safety shall be designed to permit appropriate periodic inspection and testing of important areas and features, such as wiring, insulation, connections, and switchboards, to assess the continuity of the systems and the condition of their components. As stated in the LAR, Calvert Cliffs meets the requirements of GDC 18. Each DG is designed to have adequate features to permit appropriate periodic inspection and testing of the operability of the DG system as a whole.

NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants: LWR Edition," Section 8.3.1, "AC Power Systems (Onsite)," provides guidance to

determine whether the AC onsite power system satisfies the requirements of GDCs 17 and 18, and will perform its intended functions during all plant operating and accident conditions.

10 CFR 50.36(c)(3), "Technical Specifications," requires, in part, that TSs include SRs relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the limiting conditions for operation will be met.

Calvert Cliffs is a combustion engineering plant that adopted Standard Technical Specifications (NUREG-1432, Revision 1) by Amendment Nos. 227 and 201, dated May 4, 1998.

3.0 TECHNICAL EVALUATION

The derivation of proposed power factor values in SR 3.8.1.11 was not provided in the LAR. By letter dated January 11, 2012, the licensee responded to the NRC staff's request for additional (RAI) question regarding the derivation of the worst case revised power factor values. The licensee provided highest loading values of various class 1E DGs in terms of kW, kilo-voltampere-reactive (kVAR), kilo-voltampere (kVA), and power factor, as calculated in Main Steam Line Break (MSLB), Large Break LOCA (LBLOCA), Small Break LOCA (SBLOCA), and LOOP (NORMAL) design basis scenarios.

DG	EVENT	kW	kVAR	kVA	Power Factor
1A	MSLB	3248.7	2325.9	3995.5	0.81
	LBLOCA	3298.2	2370.5	4061.7	0.81
	SBLOCA	3239.4	2325.3	3987.6	0.81
	NORMAL	3504.4	2302.9	4193.4	0.84
1B	MSLB	2381.9	1595.8	2867	0.83
	LBLOCA	2567.7	1699.9	3079.4	0.83
	SBLOCA	2423.7	1639.3	2926	0.83
	NORMAL	2397.3	1460.1	2807	0.85
2A	MSLB	2412.1	1649.1	2921.9	0.83
	LBLOCA	2598.1	1753.9	3134.8	0.83
	SBLOCA	2454	1692.6	2981.1	0.82
	NORMAL	2441.9	1500.1	2865.9	0.85
2B	MSLB	2873.5	1897.6	3443.5	0.83
	LBLOCA	2911	1964.5	3511.8	0.83
	SBLOCA	2864.1	1897	3435.4	0.83
	NORMAL	2768.6	1644.9	3220.4	0.86

The licensee stated that the criterion for determining which power factor to use in the SR 3.8.1.11 was based on using the highest design basis kW load in each scenario. For DGs 1B, 2A, and 2B, the LBLOCA scenario had the highest kW values, so the associated power factor (0.83) was selected for each of these DGs. For DG 1A, the LBLOCA scenario had a higher kVAR, but its kW was not the largest kW. Since the NORMAL scenario exhibited the highest kW value, the kVAR associated with the NORMAL scenario was used to determine a power factor of 0.84 for DG 1A. Coupling the highest 2370.5 kVAR with the highest 3504.5 kW was not chosen for DG 1A, since it was considered to be outside of the design basis scenario. As the power factor values for each DG are based on the largest kW values in the above table, the NRC staff finds that the proposed power values in SR 3.8.1.11 are acceptable and conservative.

The NRC staff has reviewed the information provided by the licensee in its LAR and supplemental letters and finds that the proposed change to the Note of SR 3.8.1.11 is reasonable and in accordance with NUREG-1432, "Standard Technical Specifications Combustion Engineering Plants," SR 3.8.1.14, Note 3. Therefore, the NRC staff finds the proposed change to the Note of SR 3.8.1.11 acceptable.

3.1 NRC Staff Findings

Based on the above evaluation, the NRC staff finds the proposed changes to the Calvert Cliff TS would not impact the continued availability of the EDGs to shut down and maintain the reactor in a safe condition after an anticipated operational occurrence or a postulated design-basis accident. Furthermore, the NRC staff concludes that with the proposed TS changes, the licensee would continue to meet the requirements of 10 CFR 50.36, GDC 17, and GDC 18. Therefore, the NRC staff finds the proposed changes acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Maryland State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes SRs. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (76 FR 73729). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: V. Goel

Date: August 22, 2012

August 22, 2012

Mr. George H. Gellrich, Vice President
Calvert Cliffs Nuclear Power Plant, Inc.
Calvert Cliffs Nuclear Power Plant
1650 Calvert Cliffs Parkway
Lusby, MD 20657-4702

SUBJECT: CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2 - AMENDMENT REGARDING DIESEL GENERATOR SURVEILLANCE REQUIREMENT 3.8.1.11 (TAC NOS. ME6831 AND ME6832)

Dear Mr. Gellrich:

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A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,
/ra/
Nadiyah S. Morgan, Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-317 and 50-318

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cc w/encls: Distribution via Listserv

ADAMS Accession No. ML12214A275 *See memo dated 7/24/2012

OFFICE	LPLI-1/PM	LPLI-1/LA	EEEB/BC*	STSB/BC	OGC	LPLI-1/BC
NAME	NMorgan	KGoldstein	JAndersen	RElliott	LSubin	GWilson
DATE	8/2/12	8/2/12	7/24/2012	8/3/12	8/21/12	8/22/12

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