LIST OF ATTACHMENTS TO FIRSTENERGY'S MOTION FOR SUMMARY DISPOSITION OF CONTENTION 4 (SAMA ANALYSIS SOURCE TERMS)
(Attachments provided pursuant to ¶ G.3 of the Board's June 15, 2011 Initial Scheduling Order)

Attachment	Description
1	FirstEnergy's Statement of Material Facts on Which There is No Genuine Issue to be Heard (July 26, 2012)
2	Joint Declaration of Kevin O'Kula and Grant Teagarden in Support of FirstEnergy's Motion for Summary Disposition of Contention 4 (SAMA Analysis Source Terms) (July 26, 2012)
3	*Curriculum Vitae* of Dr. Kevin R. O'Kula
4	*Curriculum Vitae* of Mr. Grant A. Teagarden
5	Letter from B. Allen, FirstEnergy, to NRC Document Control Desk, "Correction of Errors in the Davis-Besse Nuclear Power Station, Unit No. 1, License Renewal Application (TAC No. ME4613) Environmental Report Severe Accident Mitigation Alternatives Analysis, and License Renewal Application Amendment No. 29" (July 16, 2012)
6	Letter from K. Byrd, FirstEnergy, to NRC Document Control Desk, "Reply to Request for Additional Information for the Review of the Davis-Besse Nuclear Power Station, Unit No. 1, License Renewal Application (TAC No. ME4613), Environmental Report Severe Accident Mitigation Alternatives Analysis, and License Renewal Application Amendment No. 10 (June 24, 2011)
7	Letter from B. Allen, FirstEnergy, to NRC Document Control Desk, "Reply to Supplemental Request for Additional Information for the Review of the Davis-Besse Nuclear Power Station, Unit No. 1, License Renewal Application (TAC No. ME4613) Environmental Report Severe Accident Mitigation Alternatives Analysis" (Sept. 1, 2011)
8	NUREG-1465, *Accident Source Terms for Light-Water Nuclear Power Plants* (Feb. 1995)
9	Excerpt from NRC Office of Nuclear Regulatory Research, *Draft for Comment, Reactor Risk Reference Document*, NUREG-1150, Vol. 1 (Feb. 1987)
10	Excerpts from NUREG-1150, "Severe Accident Risks: An Assessment for Five U.S. Nuclear Power Plants" (Dec. 1990)
11	Excerpt from Policy Statement on Severe Reactor Accidents Regarding Future Design and Existing Plants, 50 Fed. Reg. 32,138 (Aug. 8, 1985)
12	Excerpt from NUREG-1437, "Generic Environmental Impact Statement for License Renewal of Nuclear Plants," Vol. 1 (May 1996)
13	Excerpt from Final Rule: Environmental Review for Renewal of Nuclear Power Plant Operating Licenses, 61 Fed. Reg. 28,467 (June 5, 1996), amended by 61

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	Fed. Reg. 66,537 (Dec. 18, 1996)
14	NEI 05-01, Rev. A, "Severe Accident Mitigation Alternatives (SAMA) Analysis Guidance Document" (Nov. 2005)
15	Final License Renewal Interim Staff Guidance LR-ISG-2006-03: Staff Guidance for Preparing Severe Accident Mitigation Alternatives Analyses" (Aug. 2007)
16	NUREG/BR-0184, "Regulatory Analysis Technical Evaluation Handbook, Final Report" (January 1997)
17	NUREG/BR-0058, "Regulatory Analysis Guidelines of the U.S. Nuclear Regulatory Commission, Revision 4" (August 2004)
18	U.S. Nuclear Regulatory Commission, *Probabilistic Risk Assessment (PRA)*. http://www.nrc.gov/about-nrc/regulatory/risk-informed/pra.html)
19	NUREG/CR-6613, "Code Manual for MACCS2: User's Guide," Vol. 1 (May 1998)
20	Excerpt from Electric Power Research Institute ("EPRI") Report 1020236, "MAAP4 Applications Guidance: Desktop Reference for Using MAAP4 Software, Revision 2" (2010)
21	Fauske & Associates, LLC, *MAAP (Modular Accident Analysis Program)*, http://www.fauske.com/pdf/MAAP.pdf
22	Letter from Gary M. Holahan, NRC, to Theodore U. Marston, EPRI (Dec. 4, 2001)
23	Excerpt from EPRI Report 1013492, "Probabilistic Risk Assessment Compendium of Candidate Consensus Models" at 2-3 (2006)
24	Excerpt from NEA Committee on the Safety of Nuclear Installations, NEA/CSNI/R(2007)16, *Recent Developments in Level 2 PSA and Severe Accident Management* (Nov. 2007)
25	Excerpt from Kenneth D. Kok, Ed., *Nuclear Engineering Handbook* (2009)
26	NUREG-1437, Supp. 47, "Generic Environmental Impact Statement for License Renewal of Nuclear Plants: Columbia Generating Station – Final Report, Vol. 2, App. F (Apr. 2012)
27	NUREG-1437, Supp. 45, "Generic Environmental Impact Statement for License Renewal of Nuclear Plants: Regarding Hope Creek Generating Station and Salem Nuclear Generating Station, Units 1 and 2, Vol. 2, App. G (Mar. 2011)
28	F. Eltawila, U.S. NRC, "NRC Source Term Research – Outstanding Issues and Future Directions," European Review Meeting on Severe Accident Research, Karlsruhe, Germany, June 12-14, 2007, Slide 2

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29	TID-14844, "Calculation of Distance Factors for Power and Test Reactors" (Mar. 1962)
30	Excerpt from Regulatory Guide 1.183, "Alternative Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Reactors" (Jan. 2000)
31	Fauske & Associates, Inc. Technical Bulletin No. 1295-1, "BWR MSIV Leakage Assessment: NUREG-1465 vs. MAAP 4.0.2"
32	Excerpt from Regulatory Guide 1.174, "An Approach for Using Probabilistic Risk Assessment in Risk-Informed Decisions on Plant-Specific Changes to the Licensing Basis," Rev. 2 at 7 (May 2011)
33	D. Harrison, NRC, Chief, PRA Licensing Branch, *Perspectives on PSA Technology and Applications,* Slide 5, *Fire PRA China PSA Workshop*, January 10, 2010
34	John R. Lehner et al., Brookhaven National Laboratory, "Benefit Cost Analysis of Enhancing Combustible Gas Control Availability at Ice Condenser and Mark III Containment Plants, Final Letter Report" (Dec. 2002)
35	Excerpt from WASH-1400 (NUREG-75/014), "Reactor Safety Study: An Assessment of Accident Risks in U.S. Commercial Nuclear Power Plants" (1975).
36	Frank J. Rahn and Robert E. Henry, "Release and Dispersion of Radioactivity from Reactor Fuel Research and Analytical Results Leading to Reductions in Radiological Source Terms," Appendix 1A to American Nuclear Society Position Statement No. 65 on "Realism in the Assessment of Nuclear Technologies" (June 2004)
37	Excerpt from NUREG-1437, Supp. 9, "Generic Environmental Impact Statement for License Renewal of Nuclear Plants: Regarding Catawba Nuclear Station, Units 1 and 2 – Final Report at 5-9 to 5-10 (Dec. 2002)
38	Memorandum from A. Rubin to M. Cunningham, "Telecommunication with Duke Energy Corporation in Support of Generic Safety Issue (GSI) 189, 'Susceptibility of Ice Condenser and BWR Mark III Containments to Early Failure from Hydrogen Combustion During a Severe Accident'" (Oct. 8, 2002)
39	NUREG-1935, "State-of-the-Art Reactor Consequence Analyses (SOARCA) Report, Draft Report for Public Comment" (Jan. 2012)
40	Excerpt from NUREG/CR-2239, "Technical Guidance for Siting Criteria Development" (1982).
41	NUREG/CR-7110, Vol. 1, "State-of-the-Art Reactor Consequence Analyses Project: Peach Bottom Integrated Analysis" (Jan. 2012)
42	NUREG/CR-7110, Vol. 2, "State-of-the-Art Reactor Consequence Analyses Project: Surry Integrated Analysis" (Jan. 2012)

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43	NUREG/BR-0359, "Modeling Potential Reactor Accident Consequences," (Jan. 2012) (Attach. 43).
44	K. Vierow, Y. Liao, J. Johnson, M. Kenton, and R. Gauntt, "Severe accident analysis of a PWR station blackout with the MELCOR, MAAP4, and SCDAP/RELAP5 Codes," *Nuclear Engineering and Design 234*, at 129-145 (2004)
45	International Atomic Energy Agency, *IAEA International Fact Finding Expert Mission of the Fukushima Dai-ichi NPP Accident Following the Great East Japan Earthquake and Tsunami* at 33-35 (June 2011)
46	Excerpt from Energy Research, Inc., ERI/NRC 02-202, "Accident Source Terms for Light-Water Nuclear Power Plants: High Burnup and Mixed Oxide Fuels" (Nov. 2002)
47	Excerpt from NUREG-1503, "Final Safety Evaluation Report Related to Certification of the ABWR Standard Design," Vol. 1 (July 1994)
48	Excerpt from NUREG-1793, "Final Safety Evaluation Report Related to Certification of the AP1000 Standard Design," Vol. 1 (Sept. 2004)