



UNITED STATES
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July 26, 2012

Mr. Thomas D. Gatlin
Vice President, Nuclear Operations
South Carolina Electric & Gas Company
Virgil C. Summer Nuclear Station
Post Office Box 88
Jenkinsville, SC 29065

SUBJECT: VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1 (VCSNS) – REQUEST
FOR ADDITIONAL INFORMATION (TAC NO. ME7586)

Dear Mr. Gatlin:

By letters dated November 15, 2011 and January 26, 2012, the South Carolina Electric & Gas Company (SCE&G, the licensee) submitted a license amendment request (LAR) to revise Facility Operating License Number NPF-12 for the Virgil C. Summer Nuclear Station, Unit No. 1. The LAR would permit transition of the fire protection licensing basis from Title 10 of the *Code of Federal Regulations (10 CFR)*, Section 50.48(b), to 10 CFR 50.48(c), "National Fire Protection Association Standard NFPA 805" (NFPA 805).

The U.S. Nuclear Regulatory Commission (NRC) staff, including the technical areas of fire protection, probabilistic risk assessment licensing, and health physics and human performance, has reviewed the licensee's information and has also participated in an audit of that information from June 4 to June 8, 2012. The staff has determined that additional information is needed to complete the review, as discussed below. A draft version of these requests for additional information (RAIs) was transmitted on June 8, 2012 (Agencywide Documents Access and Management System No. ML12168A001).

The NRC staff requests that SCE&G respond to these RAIs within 60 calendar days from the date of this letter. Please note that the staff's review is continuing and further RAIs may be developed.

Sincerely,

A handwritten signature in cursive script that reads "Robert E. Martin".

Robert E. Martin, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-395

Enclosure: Request for Additional Information

cc w/encl: Distribution via Listserv

REQUESTS FOR ADDITIONAL INFORMATION
LICENSE AMENDMENT REQUEST TO ADOPT
PERFORMANCE-BASED NATIONAL FIRE PROTECTION ASSOCIATION
STANDARD 805 FOR FIRE PROTECTION
FOR VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1
DOCKET NO. 50-395

By letters dated November 15, 2011 and January 26, 2012, the South Carolina Electric & Gas Company (SCE&G, the licensee) submitted a license amendment request (LAR) to revise Facility Operating License Number NPF-12 for the Virgil C. Summer Nuclear Station, Unit No. 1 (VCSNS). The LAR would permit transition of the fire protection licensing basis from Title 10 of the *Code of Federal Regulations (10 CFR)*, Section 50.48(b), to 10 CFR 50.48(c), "National Fire Protection Association Standard NFPA 805" (NFPA 805).

The staff, including the technical areas of fire protection, probabilistic risk assessment licensing, and health physics and human performance, has reviewed the information provided by the licensee and has also participated in an audit of that information from June 4 to June 8, 2012. The staff has determined that additional information is needed to complete the review as discussed below. An email with draft versions of these requests for additional information (RAIs) was transmitted on June 8, 2012 (ADAMS No. ML12168A001). Some of the draft RAIs were determined to no longer be necessary and they are noted as deleted below in order to maintain a numbering scheme consistent with the draft RAIs. The abbreviations in the following RAIs stand for the following: MP [Monitoring Program], SSD [Safe Shutdown], FPE [Fire Protection Engineering], FM [Fire Modeling], PRA [Probabilistic Risk Assessment], and RR [Radioactive Release].

MP RAI 01

National Fire Protection Association Standard 805, "Performance-Based Standard for Fire Protection for Light Water Reactor Electric Generating Plants", 2001 Edition, (NFPA 805), Section 2.6 "Monitoring" states that "a monitoring program shall be established to ensure that the availability and reliability of the fire protection systems and features are maintained and to assess the performance of the fire protection program in meeting the performance criteria" and that "Monitoring shall ensure that the assumptions in the engineering analysis remain valid."

Specifically, NFPA 805, Section 2.6 states that (2.6.1) "Acceptable levels of availability, reliability, and performance shall be established;" (2.6.2) "Methods to monitor availability, reliability, and performance shall be established; the methods shall consider the plant operating experience and industry operating experience;" and (2.6.3) "If the established levels of availability, reliability, or performance are not met, appropriate corrective actions to return to the established levels shall be implemented. Monitoring shall be continued to ensure that the corrective actions are effective."

Enclosure

Section 4.6.2, "Overview of Post-Transition NFPA 805 Monitoring Program" of the Transition Report states that the NFPA 805 monitoring program will be implemented "after the safety evaluation issuance as part of the fire protection program transition to NFPA 805" (Table S-2, Implementation Items, Item 4 of the Transition Report). The Transition Report also states that a monitoring program consistent with the NRC approved version of FAQ 10-0059 will be implemented.

The staff noted that the information provided in Section 4.6.2 of the Transition Report is insufficient for the staff to complete its review of the monitoring program, and as such, is requesting that the following additional information be provided:

- a. A description of the process by which structures, systems, and components (SSCs) and programmatic elements will be identified for inclusion in the NFPA 805 monitoring program, including the approach to be applied to any fire protection SSCs that are already included within the scope of the Maintenance Rule program.
- b. A description of the process that will be used to assign availability, reliability, and performance goals to SSCs and programmatic elements within the scope of the monitoring program including the approach to be applied to any SSCs and programmatic elements for which availability, reliability, and performance goals are not readily quantified.
- c. A demonstration of how the monitoring program will address response to programmatic elements that fail to meet performance goals (example discrepancies identified in programmatic areas such as combustible controls programs).
- d. A description of how the monitoring program will address fundamental fire protection program elements.
- e. A description of how the guidance in EPRI Technical Report (TR)1006756, "Fire Protection Equipment Surveillance Optimization and Maintenance Guide" if used, will be integrated into the monitoring program.

Note that such changes permitted by NFPA Section 3.2.3 require NRC approval with an appropriate justification in accordance with 10 CFR 50.48 (c)(2)(vii).

- f. A description of how periodic assessments of the monitoring program will be performed taking into account, where practical, industry wide operating experience including whether this process will include both internal and external assessments and the frequency at which these assessments will be performed.
- g. A confirmation that periodic NFPA 805 assessments (audits) of the fire protection program will be conducted under the existing Fire Protection Quality Assurance Program. If not, describe the process that will be used to conduct these assessments.

SSD RAI 01

License Amendment Request (LAR) Section 4.2.1.1, "Compliance with NFPA 805 NSCA (Nuclear Safety Capability Assessment) (Section 2.4.2)" states: "The NSCA methodology review evaluated the existing NSCA methodology against the guidance provided in NEI 00-01, "Guidance for Post-Fire Safe Shutdown Analysis", Revision 1, Chapter 3, "Deterministic Methodology," as discussed in Appendix B-2 of NEI 04-02, "Guidance for Implementing a Risk Informed Performance Based Fire Protection Program under 10 CFR 50.48(c)."

NEI 00-01, Revision 2 is the current version cited in Regulatory Guide 1.205, "Risk Informed Performance-Based Fire Protection for Existing Light-Water Nuclear Power Plants", Revision 1. The LAR references section lists Revision 1 and 2. Provide a gap analysis on the differences between the alignments using NEI 00-01, Rev. 1, as the basis for transitioning the NFPA 805 nuclear safety capability as indicated in NEI 04-02, versus using NEI 00-01, Revision 2, which is the current version cited in Regulatory Guide 1.205, Revision 1.

SSD RAI 02

LAR Section 4.2.1.2, "Safe and Stable Conditions for the Plant" states that "For the plant to be in a safe and stable condition, it may not be necessary to perform a transition to cold shutdown as currently required under 10 CFR 50, Appendix R. Therefore, the unit may remain at or below the temperature defined by a hot standby/hot shutdown plant operating state for the event."

If there are any fire areas where it is necessary to perform a transition to cold shutdown, provide a detailed discussion of why the transition to cold shutdown is needed, and what time limitations are required for achieving cold shutdown. If yes, describe the fire areas/zones along with a list of cold shutdown systems and equipment for NFPA 805 compliance. Define the word "event" as it applies to the above statement.

SSD RAI 03

LAR Section 4.2.1.2, "Safe and Stable Conditions for the Plant," states that "If evacuation of the Main Control Room (MCR) was required due to a significant fire in the Control Complex, the Control Room Evacuation Panel (CREP) is designed to provide the instrumentation and controls to maintain Hot Standby, as a Primary Control Station (PCS)."

LAR Attachment G provides a list of recovery actions (RAs) required to be taken in the plant to maintain hot standby conditions. Describe any instrumentation and controls such as local equipment operation or instrument indications, not part of the CREP, used to maintain Hot Standby that are not listed as RAs in Attachment G.

SSD RAI 04

Deleted

SSD RAI 05

LAR Section 4.2.1.2, "Safe and Stable Conditions for the Plant (Results)" states that "Some systems, such as the Chemical and Volume Control System (CVCS), serve multiple goals of coolant inventory addition and boric acid addition for long term reactivity control. Following the initial coping/assessment period at the start of a fire, the operators will maintain safe and stable conditions as follows:"

Define the term "initial coping/assessment period" as it is used in the above section. During this period, describe if and how any operators will be performing any safe shutdown operator RAs as a result of the fire. Describe what guidance documents and procedures operators use to determine when to exit the initial coping/assessment period.

SSD RAI 06

LAR Section 4.2.1.3, "Establishing Recovery Actions" states "The discussion below provides the methodology used to define and assess the Recovery Actions necessary to support the goals of the NFWA 805 Nuclear Safety Capability Assessment (NSCA) for VCSNS. This process was initially based on FAQ 07-0030 (ML110070485) and consists of the following steps:"

Describe what this process is currently based on.

SSD RAI 07

LAR, Attachment G, "Recovery Actions Transition" states that "Based on the definition provided in RG 1.205, and the additional guidance provided in FAQ 07-0030, the following location is considered taking place at the primary control station:"

A list of the actions is in the Results of Step 1 Section of the LAR.

RG 1.205, Section 2.4 states: "The staff has identified two cases where operator actions taken outside the MCR may be considered as taking place at a primary control station. These two cases involve dedicated shutdown or alternative shutdown controls, which have been reviewed and approved by the NRC. In either case, the location or locations become primary when command and control is shifted from the MCR to these other locations."

- a. Describe whether all of the actions in both cases have been reviewed and approved by the NRC and include the references indicating approval, including excerpts of statements of explicit NRC approval.
- b. Describe whether the location or locations of all of the actions become primary when command and control is shifted from the MCR to these other locations.
- c. Describe whether the actions in both cases meet the criteria in RG 1.205, Section 2.4 a. and b.

SSD RAI 08

LAR Section 4.2.1.2, "Safe and Stable Conditions for the Plant (Results)" states that "Reactor coolant system (RCS) pressure control is maintained by the ability to increase pressure by an emergency bus supplied pressurizer heater bank or by control of the charging rate and by the ability to reduce pressure by pressurizer power operated relief valve (PORV) operation."

LAR Attachment B, Section 3.1.2.2, "Pressure Control Systems" states that "Although utilization of the pressurizer heaters and/or auxiliary spray reduces operator burden, neither component is required to provide adequate pressure control. Pressure reductions are made by allowing the RCS to cool/shrink, thus reducing pressurizer level/pressure. Pressure increases are made by initiating charging/makeup to maintain pressurizer level/pressure. Manual control of the related pumps is acceptable."

- a. Based on the performance-based analyses, describe whether there are any fire scenarios where both pressurizer heaters and auxiliary spray would not be available to support reactor pressure control nuclear safety performance criteria (NSPC). If there are any scenarios, provide a discussion of how pressure control NSPC will be achieved.
- b. Describe whether there are any fire areas where long term pressure control would require the use of solid plant conditions. If so, describe how the operators have been trained in performing these actions under post-fire conditions. Describe the procedures that control post-fire solid plant conditions.
- c. Provide a justification for the generic statement in Appendix B, Section 3.1.2.2 that states "Manual control of the related pumps is acceptable."
- d. When pressurizer PORV operation is used for reducing RCS pressure, describe how it is analyzed as "required" for safe shutdown.

SSD RAI 09

LAR Table S-1, "Plant Modifications Committed", indicates that a modification will provide instrument air auto start capability for the Diesel Driven Air Compressor (XAC0014).

Explain the process used to analyze possible fire damage to instrument air equipment such as air filters that could create an air leak and affect the instrument air system pressure and/or capacity.

SSD RAI 10

Deleted

SSD RAI 11

LAR Table 4-2 "NEI 04-02 Improvements, Post Transition Alignment (3.3.2 B Common Power Source Cables)" states "The NFWA 805 transition project has analyzed common power supplies required to be energized for the NSCA function to ensure compliance with NEI 00-01. Cases

where breaker coordination has been determined to be insufficient, entries will be made into the Corrective Action Program as a part of NFPA 805 implementation.”

- a. Provide the status for the cases where breaker coordination was determined to be insufficient for the current electrical system. Include in the discussion when compliance to the common power supply requirement will be achieved.
- b. Describe whether entries were made into the Corrective Action Program (CAP) for these cases, including compensatory measures for the current license, if necessary.

SSD RAI 12

LAR 4.2.1.2 “Safe and Stable Conditions for the Plant, Safe and Stable Summary Description” states “An important part of maintaining RCS inventory is maintaining Reactor Coolant Pump (RCP) seal integrity. RCP seal cooling is maintained by either the charging pump seal injection path or the Component Cooling (CC) flow to the RCP thermal barrier heat exchanger. Modifications are planned (see Table S-1 in Attachment S) to provide a redundant seal injection system that is independent of the existing system and not affected in the problem fire areas. Second, a new seal material is planned (see Table S-1 in Attachment S) so that the loss of seal cooling does not lead to significant loss of RCS inventory. Until new seal materials are installed, procedures for seal cooling interruptions are in place to address the issue as a part of the existing appendix R analysis.”

During the NFPA 805 audit, discussions indicated that there is the possibility that the new RCP seal installation may be staggered such that one or more seals may be installed during the implementation window, and the remaining seals installed post-transition.

If all of the new seals are not scheduled to be installed during the transition period, describe whether the procedures for seal cooling interruptions are planned to be part of the approved NFPA 805 license.

SSD RAI 13

LAR Section 4.2.1.2, “Safe and Stable Conditions for the Plant, (Safe and Stable Summary Description)” under the bullet discussing Support Systems states “Systems typically not credited (but potentially available) include instrument air, secondary side support, industrial cooling, and other plant systems not associated with a safety function.”

- a. The above discussion addresses systems not typically credited. Provide the areas where any of these systems are credited and the analysis that documents that they would be available.
- b. Describe how the probabilistic risk assessment (PRA) modeled these systems.

SSD RAI 14

Describe the basis for Section 4.5.2, “Proper Polarity Three Phase Hot Shorts on alternating current (AC) Power Conductors” of the “NFPA 805 and Fire PRA Circuit Analysis, Task 4.4”, TR

07800-009.

Describe whether the 13 open items in Appendix C, Circuit Analysis Opens Table, of “NFPA 805 and Fire PRA Circuit Analysis, Task 4.4”, TR 07800-009, have been entered into the CAP and whether they have been completed.

SSD RAI 15

LAR Section 1.2, “Historical Perspective and Discussion”, states that “Molded case circuit breakers should be periodically exercised and inspected and on a rotating refueling outage basis, should be sample tested to confirm drift remains in acceptable limits.”

Describe how the above testing and exercising of the molded case breakers is procedurally controlled for breakers that are credited for meeting NFPA 805.

SSD RAI 16

Section 2.0 “Scope” of “Safe Shutdown Separation Fire Protection Engineering Evaluations,” TR 0780E-001, states “The scope of this TR is for the fire protection engineering evaluations (FPEEs) that are related to the NSCA and Safe Shutdown Separation requirements...”

Describe whether all open items, such as reworking existing fire protection systems, have been completed. If not, describe how they are being tracked and provide the latest schedule for completion. Describe whether items that are not in compliance with the current license have been entered into the CAP and confirm that compensatory measures have been implemented.

SSD RAI 17

The alternate shutdown RAs of Attachment 6 “(CREP Shutdown Transition (PCS) and Recovery Actions)”, of “Nuclear Safety Capability Assessment Report Fire Shutdown Analysis” TR08620-312, differ from the alternate RAs in LAR Table G-1, “Recovery Actions and Activities Occurring at the Primary Control Stations.”

Describe which document is correct, explain the apparent discrepancy, and describe any corrections made.

SSD RAI 18

NEI 04-02 Table B-3, “Fire Area Transition” Attachment 14, Fire Area Assessment Table (Table B-3); Attachment 14, Fire Area IB08, and LAR, Attachment C, Fire Area IB08, discuss water from fire suppression activities draining to adjacent fire zones in the 412-foot main floor area.

Describe whether the 412-foot main floor area has been analyzed for this additional water draining into the area. Describe if there are similar areas where water from fire suppression activities could affect other areas and whether they have been analyzed.

SSD RAI 19

Section 4.4.4.3.2, "Chilled Water Mechanical Chillers" of TR O8620-015 "Nuclear Safety Equipment Report", Part 1, states that "A RA [recovery action] is available to start the B chiller or the C chiller on B train at the respective chiller control panel after taking the transfer switch to local."

This RA does not appear on LAR Table G-1. Discussions of RAs not in Table G-1 are found throughout TR O8620-015. Explain this discrepancy.

SSD RAI 20

Section 4.3.1.3.5.c, "Heating Ventilation and Air Conditioning (HVAC) Systems" of TRO8620-015 "Nuclear Safety Equipment Report", Part 1, states that "Plant-specific evaluations are necessary to determine which HVAC systems are essential to safe shutdown equipment operations."

- a. Describe whether all HVAC systems evaluations been completed.
- b. Provide a list of all HVAC systems that have been determined to be "essential to safe shutdown equipment operations."
- c. Describe the actions, modifications and procedures that have been identified and include a schedule for completion.

FPE RAI 01

Incipient Detection is described as a necessary modification in LAR Attachment S for electrical cabinets. FAQ 08-0046 (NUREG 6850, "EPRI/NRC-RES Fire PRA Methodology for Nuclear Power Facilities," Supplement 1) provides detailed discussion of the elements and limitations that are necessary to credit incipient detection systems for Fire PRA (FPRA).

Provide more details regarding system design features, NFPA code(s) of record, installation, acceptance testing, setpoint control, alarm response procedures and training, and routine inspection, testing, and maintenance that will be implemented to credit the new incipient detection system. Provide details regarding availability and reliability. Describe whether this installation and the credit that will be taken will be in compliance with each of the method elements, limitations and criteria of NUREG/CR- 6850 Supplement 1, Chapter 13, or will there be deviations. Provide justification for any deviations.

FPE RAI 02

LAR Attachment S "Plant Modifications Committed" Table S-1 for ECR1553, identifies a modification to "improve availability and reliability of station communication system(s) during fire scenarios."

Provide more detail with regard to what specific modifications are being done. Describe whether these modifications are assumed to be in place when determining feasibility for RAs.

Describe whether there are open items in the feasibility evaluation identifying the need to complete these modifications.

Table S-1 indicates that communications is “implicitly considered in credit for operator actions.” Describe how communications was evaluated during RA feasibility evaluation and in the FPRA (establishment of Human Error Probabilities (HEP) and Human Reliability Analysis (HRA)).

FPE RAI 03

LAR Attachment S “Plant Modifications Committed” ECR50810 is supposed to “provide mitigation strategies to address fire initiators or limit fire propagation.”

- a. Provide more detail for this modification(s).
- b. Describe the modification, including location.
- c. Describe the applicable design standards.
- d. Describe the elements of defense-in-depth (DID) that are being satisfied.
- e. This modification is identified as being in the Fire PRA. Describe specifically what is “in the Fire PRA”.

FPE RAI 04

The LAR states in Table B-1 Section 3.2.3(2), Compensatory Actions will be revised and updated incorporating NFPA 805 insights.

Describe what compensatory actions will be revised and whether actions, impairment duration limits, or reporting requirements will be created or modified. If yes, provide a detailed list of those changes being made, and the justification for those changes in compensatory measures being considered. Describe whether the Technical Requirements Manual (TRM) (or comparable plant document) is expected to transition or will it be superseded.

FPE RAI 05

The LAR states in Table B-1 Section 3.3.1.2 states that “controls of limited duration” for untreated wood will be put in place...Attachment L1 identifies the need to use non-treated wood in limited quantities in an attempt to address unique situations.

Provide a description of the process that would control these deviations. Describe whether the future process will use the “Plant Change Evaluation” defined in NFPA 805 to evaluate the impact to the Fire Protection Program (FPP). Provide a specific description of engineering and administrative procedure changes that the fire protection engineer (FPE) will use to control these elements of the requirement 3.3.1.2. Describe whether temporary non-compliances of limited duration will be considered program non-compliances requiring compensatory measures. Explain how instances of untreated wood will be controlled such that the assumptions of the FPRA will remain valid.

FPE RAI 06

The LAR states in Table B-1 Section 3.3.9 that “A Fire Hazard Evaluation of the Transformer area considered drainage alternatives to that cited in this section. (Table S-2, Item 2).”

It is not apparent how Table S-2, Item 2 addresses this requirement. Provide a detailed description of the Fire Hazard Evaluation of the Transformer area drainage alternatives. Provide adequate justification for any alternative compliance approaches.

FPE RAI 07

LAR Table B-1 Section 3.3.7.2 identifies that the outdoor high pressure flammable gas storage is evaluated in TR0780E-006. There is no bulk gas storage in this evaluation. Clarify this discrepancy.

FPE RAI 08

LAR Attachment L2 requests approval for “existing wiring in suspended ceilings.” Describe what fire area(s) constitute “...suspended ceilings is limited in risk significant areas important to the NSCA, FPRA and NPO analysis.” The request states that “...wiring is specified to be within metal conduits, cable trays, armored cable, or rated for plenum use.” Wiring that is in cable trays without metal tops and bottoms does not meet the requirements.

The current justification in Approval Request L2 is not adequate. A request to comply with chapter 3 using performance-based methods [in accordance with 10 CFR 50.48(c)(2)(vii)] must satisfy criteria for risk, DID and safety margins (SM). The current request only addresses DID and SM. Provide either a qualitative or quantitative risk evaluation that demonstrates the ability to meet the acceptance criteria.

Explain whether this describes the “as-built” condition or the “as designed” condition. Describe the types of cables, the types of service, and the fire protection features (e.g. detection) that are currently installed above the suspended ceilings.

FPE RAI 09

LAR Attachment L3 “Electrical Cable Construction” does not identify the as-built condition of currently installed “existing non-compliant” cables.

Describe the cable flame spread or other cable construction standards that were used for the currently installed cables. Describe whether these standards are addressed in FAQ 06-0022 and whether the current installation is different from the NFPA 805 requirement. Describe the hazards that are being presented and whether those elements are alternatively meeting the intent of the requirement.

FPE RAI 10

LAR Attachment L4 “Bulk Gas Storage” presents an alternative method to meet a NFPA 805 requirement.

A request for approval of the use of a performance-based method to meet a requirement in NFPA 805 Chapter 3 must contain an evaluation of risk, DID and SMs. The current request only addresses DID and SM.

The justification should include details regarding the extent of the hazard, the quantity and capacity of the tanks, a description of the refilling process and a description of the features that reduce the hazards associated with this alternative method. Since the tanks may form a missile hazard, include a discussion of the substantial buildings and structures between the tanks and any equipment required to meet the nuclear safety performance criteria.

FPE RAI 11

LAR Attachment request L5 “Fire Brigade Notification” identifies the need to allow for delayed fire brigade response based on verification of a “direct visual contact with the fire”.

Describe how this delay will be factored into the assumptions for time-to-damage and non-suppression probability for the FPRAs assumptions.

FPE RAI 12

LAR Attachment request L9 “Hose Station” identifies that pressure and flow rates are not reduced by pressure reducers and requests that the NRC approve this configuration. Provide justification why not reducing water pressure at hose stations is acceptable. Include a description of the system including the minimum and maximum calculated pressures and flows, and the impact to personnel and equipment of not having the pressure reducing valves installed. In addition also include a description of any training provided to plant personnel using the hose stations at this higher pressure.

FPE RAI 13

LAR Attachment request L11 appears to ask for approval of all existing fire detector layouts throughout the plant in accordance with NFPA 72E. The fire alarm and detection system was upgraded to NFPA 72, however, fire detection device layout was not upgraded and remained in accordance with NFPA 72E.

Describe whether the intention of this request is for the NRC staff to approve the individual locations of the detectors throughout the plant as being compliant with NFPA 72E as opposed to NFPA 72. If this is the intention, provide a gap analysis of the differences between the detector location requirements under NFPA 72E and NFPA 72 from the respective code years. For any location requirements that the detectors will not meet under the updated code, provide a justification for not updating the detector locations in accordance with the newer code.

FPE RAI 14

Table B-1 compliance statement CA – is defined as either 1. Clarification of the requirement or as a 2. Submitted request for approval. Approvals are listed in Attachment L and presented as (C)(2)(vii)'s.

NEI 04-02 Section 4.3.1 provides specific compliance statements for each Chapter 3 attribute. Two of the means to comply are:

- Complies with Clarification - Items that are not in 'literal compliance' with the requirement as listed in NFPA 805 but should be transitioned as complies, and
- 10 CFR 50.48(c)(2)(vii) allows licensees to use performance-based methods to demonstrate compliance with NFPA 805 Chapter 3 requirements.

These two compliance strategies are combined into "Complies by Alternative (CA)." This is interpreted as some CA's in table B-1 are merely editorial clarifications and others require separate NRC approval as a 10 CFR 50.48 (c)(2)(vii) listed in Attachment L.

In an effort to ensure clarity, provide a listing of each Chapter 3 requirement that uses "CA" as the compliance strategy and explicitly delineate which strategy is being chosen. If the strategy is intended to be an editorial change then explicitly state the editorial clarification or change. If the strategy is a request for approval then state which approval in Attachment L is being credited for compliance in the specific NFPA 805 Chapter 3 requirement.

FPE RAI 15

Table B-1 identifies Chapter 3 elements where credit is being taken for "Previous NRC Approval (CNRC)". Compliance statements are identified in Table B-1, but the referenced Safety Evaluation Reports (SERs) are not identified. The results summary only indicates "...were previously found to be acceptable and has been summarized in the defined evaluation." NEI 04-02 Section 4.3.1 and associated Figure 4-2 delineate the method to identify those requirements where previous approval has been used. The Section 4.3.1 and the simplified flowchart identify the need to provide verbatim excerpts from the submittal documents and the approval documents in the compliance basis field.

For those requirements in Table B-1 of the LAR where CNRC credit was used, provide appropriate SER references and excerpted submittal and approval statements of explicit NRC approval. Provide statements regarding the continued validity of the original basis of approval.

FPE RAI 16

In the definition of the Power Block, Attachment I, Table I-1, the Yard is identified as a power block structure included in the FPP. Confirm that this designation includes the following structures/features:

- Condensate Storage Tank
- Refueling Water Storage Tank
- Manholes
- Diesel Generator Fuel oil Tanks
- Auxiliary Boiler Oil Storage Tanks
- Transformers

If any of these are not included provide justification for their exclusion.

FPE RAI 17

In plant partitioning for FPRA, confirm that no active fire barriers (e.g. water curtains), thermal wraps, ERFBS, or fire resistant coatings are used to credit partitioning boundaries. If any of these fire protection features are credited in this manner, identify what type, fire area / zone location, and duration of rating credited.

FPE RAI 18

The LAR states in Table B-1, Section 3.11.3, that NFPA 101, "Life Safety Code", is exempted from the scope of the NRC review per 10 CFR 50.48 C.2 (i) regarding Life Safety.

The NFPA 805 requirement relates to the features of penetrations in fire barriers, specifically doors and dampers. Identify the strategy of compliance with Section 3.11.3 Fire Barrier Penetrations with regard to NFPA 101.

FM RAI 01

NFPA 805, Section 2.4.3.3, states: "The PSA [probabilistic safety assessment] approach, methods, and data shall be acceptable to the AHJ [authority having authority] ... " The NRC staff noted that fire modeling comprised the following:

- The Consolidated Fire and Smoke Transport Model (CFAST) was used to calculate control room (CR) abandonment times.
- CFAST was used to calculate Hot Gas Layer (HGL) temperature for damage determinations in selected fire zones throughout the plant.
- Fire Dynamics Tools (FDTs) was used to calculate Zone of Influence (ZOI) dimensions in selected fire zones throughout the plant.

Section 4.5.1.2, "FPRA Quality" of the Transition Report states that fire modeling was performed as part of the FPRA development (NFPA 805, Section 4.2.4.2). Reference is made to Attachment J, "Fire Modeling V&V," for a discussion of the acceptability of the fire models that were used.

Specifically regarding the acceptability of CFAST for the CR abandonment time study:

- a. Provide the input files in electronic format for 6 selected CFAST runs that were conducted, i.e., the input files for the cases with the highest heat release rate (HRR) in Tables 6, 8, 10, 13, 15 and 17 in the Control Room Risk Calculation Report (Section 8 of Calculation No. DC0780B-100, Fire Modeling: CB17.01).
- b. Due to presence of a large number of cabinets and control boards, the effective volume of the MCR will be less than if calculated using the length, width and height. Explain why the presence of this equipment was not considered in the volume estimation.

- c. In Section 4.3.3 of Calculation No. DC0780B-100, it is discussed how the horizontal natural ventilation flow areas are determined. It appears that a characteristic opening fraction from the study of Klote and Milke was used to calculate the average opening fraction for every door. Explain why an average characteristic value for all opening gaps is used instead of the actual door gaps.

Explain why this approach is conservative and describe the results of the parametric analysis to show the dependence of abandonment time on the ventilation opening area, if such an analysis was performed.

- d. Explain why the default drop-off and zero flow pressure values in CFAST were used. Explain how these values are consistent with the plant-specific HVAC data.
- e. Calculation No. DC0780B-100 mentions (page 16) that the fire location was chosen such that it produces worst case fire scenarios. Explain the justification for this assertion.

In addition, it is noted (page 21) that the fire location for all simulations is at floor level in the center of the room. It is stated that the central location of the fire limits the effects of the walls on the fire growth and the plume behavior. It is not clear to the NRC staff what is meant by 'limits the effects of the walls on fire growth and plume behavior.' Explain this statement in greater detail.

- f. The abandonment time analysis has not considered fire spread from one cabinet to adjacent cabinets. However, Section 9.2 and Table 20 of Calculation No. DC0780B-100 discuss the phenomenon of fire propagating to adjacent cabinets. Provide justification for not considering a scenario involving multiple cabinet fires in the calculation for time to abandonment.
- g. It is assumed that the HRR of the transient fires is bounded by the large fixed ignition source fire (electrical cabinets with unqualified cable, fire in multiple cable bundles).
 - i. The growth time for the cabinet fire is 12 minutes, whereas according to FAQ-52, the average growth time for transient fires could be 0, 2 or 8 minutes, depending on the actual transient combustible. Provide additional justification for the assumption that the large fixed ignition source fire is bounding for transient fires.
 - ii. Though the HGL temperature and optical density is not dependent on the location of the fire, targets are used to assess the radiant heat flux criterion. It would seem that the location of a transient fire could affect these calculated target heat fluxes. Provide additional justification for not postulating any transient fires for the purpose of calculating CR abandonment.
- h. The report provides results of a sensitivity study for the times to abandonment for different arrival times of the fire brigade with and without forced ventilation. Provide the results from any additional sensitivity studies performed to address fire location, ambient temperature, etc.

Regarding the acceptability of the PSA approach, methods, and data in general as discussed in the paragraph above:

- i. Calculation DC0780B-001 suggests that an opening of 0.5-in. is assumed below every door if the leakage area is not known, as per NUREG-6850 Section F.2. It appears that NUREG-6850 prescribes this assumption only for HGL calculations. Clarify whether this assumption was used for every fire modeling calculation or just for HGL calculations. Also, the methodology used in the MCR risk calculation seems to differ from this approach. Provide further justification for this approach to door leakage areas.
- j. It is not clear how the dimensions of a “transient zone” are determined. The report states that the transient zones are larger compared to the traditional ZOI dimensions. Clarify how the dimensions for the transient zones are determined.
- k. On page 12 of Calculation DC0780B-001 (bullet No. 4 of the second paragraph described under the engineering judgment scenario), it is assumed that some fire suppression activity will be initiated before fire spreads from one transient zone to others. Explain how this is ensured when determining the transient zone size.
- l. Based on the last paragraph of page 13 (Calculation DC0780B-001), it appears that the decision to perform detailed fire modeling for a transient zone was not based on some numerical screening threshold, but was based on the judgment of the PRA analyst. Clarify what criteria were used to screen areas where fire modeling would be performed. Include a justification for why the criteria are acceptable.
- m. Page-22 (Calculation DC0780B-001) describes the method, by which targets have been identified. It is stated that if the specific location of the target is not known, it has been assigned to all the scenarios. Clarify how the location of the target was determined for every scenario that it has been assigned to and how the guidance on fire location discussed on page 27 has been applied for this case.
- n. Clarify what is meant by “bounding vertical and horizontal distances were used for calculating detection time” (page 27 of Calculation DC0780B-001). It is not clear how these values were calculated for a room that has numerous obstructions that could significantly delay detection and activation. Provide examples from specific analyses in which these activation times are calculated.

Also, elaborate on how the suppression time was conservatively estimated as stated in Section 6.2 2(b) of Calculation DC0780B-001.

- o. What are the criteria used in the generic methodology described in Calculation DC0780B-001 for determining when a more complicated fire model is necessary for evaluation of a given problem (e.g., prediction of sprinkler activation in a space with complicated geometry).
- p. On page 27 of Calculation DC0780B-001 there is discussion about uncertainty of fire location. Location factor is not explicitly discussed here. Provide additional information about how the generic methodology treats fires located close to walls or corners.

- q. On page 54 of Calculation DC0780B-001 there is a discussion about the targets. It is reported that since there is approximately 10% unknown cable and the rest is known to be thermoset, the whole plant is considered to have only thermoset cable. Provide additional justification why it is not necessary to address this 10% of unknown cable more conservatively, if any non-thermoset cables are located in the fire zones for which detailed fire modeling was conducted.
- r. On page 60 of Calculation DC0780B-001, there is discussion about the assumption of an 'incubation time' of ten minutes from hotwork fire initiation to propagation to the next tray. This same approach is used on page 63 in Section E.4 on Junction Boxes. Describe whether this is the methodology utilized and if so, provide additional justification for this assumed time period.
- s. On page 26 of Calculation DC0780F-096 for Fire Modeling Fire Area CB10 VFDR, it is discussed that this staging area also requires that no combustibles be placed within one foot of the adjacent walls, so that wall effects of the fire source do not have to be accounted for. Provide additional justification for this critical dimension of one foot of separation, beyond which wall and corner effects can be ignored. Expand the response to this question for the other fire areas where this critical separation distance is utilized (CB12, CB18 and IB11).
- t. During the walkdown of fire zone IB25, the staff reviewed the analysis of the fire scenarios in IB25.01.02 subzone, which credited fire suppression/detection. The staff has the following questions about the analysis performed in this fire zone:
 - During the audit, the staff noted that it is discussed in the detailed fire modeling report that standard response sprinklers are used in the fire zone and therefore a Response Time Index (RTI) of 130 (m-s)^{0.5} was used for the analysis. The licensee justified this value for the RTI by way of reference to NUREG 1805, "Fire Dynamics Tools", "which provides a generic RTI value of 130 (m-s)^{0.5} for standard response heads with a fusible link." However, in Chapter 10 of NUREG 1805, there is a note about selecting the RTI of a sprinkler element which states, "the actual RTI should be used when the value is available." Provide justification for the RTI value chosen for this analysis and describe how that value compares with the RTI of the actual sprinklers in the fire zone. Apply this response to any additional fire zones where detailed fire modeling was conducted to address credited suppression systems.
 - For at least one of the fire scenarios in this zone, a sprinkler activation calculation was performed to determine if/when a sprinkler would actuate. In this fire zone, there are sprinklers and smoke detectors at ceiling level and additional sprinklers at an intermediate level several feet below the ceiling. The staff has the following questions about how that configuration may impact the existing analysis:
 - o The FDT calculation for sprinkler activation assumes that the fusible link is located a short distance (a few inches) below a flat ceiling extending in all directions. The current analysis credits intermediate level sprinklers activating, however, these heads are located several feet below the ceiling

overhead. In addition, these intermediate level sprinklers were not observed to have “heat collectors” over the heads. Provide justification for the assumption that this configuration does not affect the conclusions of the analysis

- o During the audit, the staff was informed that the sprinkler system is a pre-action system and therefore requires a signal from the smoke detection system in order for water to flow to the heads. Therefore, it is important for the analysis to include the timing of smoke detection activation with respect to intermediate level sprinkler activation. Provide further clarification about the timing between smoke detector activation and intermediate-level sprinkler activation.
- u. During the audit, the staff walked down fire zone CB10, which is one of the performance-based fire modeling zones. In this zone a “transient staging area” has been proposed in the fire modeling analysis. In this southwest corner of the compartment, the staff observed three vertical pipes directly adjacent to the staging area. One of the pipes is steel, but the other two are copper pipes that are jacketed with some insulation that is non-metallic. It was not obvious what the material flammability characteristics of this insulation are and if it could be ignited from an adjacent transient fire. Provide additional information about this insulation and how it could affect the conclusions of the analysis.
- v. During the audit, the staff walked down fire zone IB11, which is one of the performance-based fire modeling zones. In this zone a “transient staging area” has been proposed in the fire modeling analysis. This staging area is identified as being located 4.5-ft from a nearby VFDR cable tray. However, during the walkdown, the staff observed several exposed cables extending from this variance from deterministic requirement (VFDR) tray in the direction of the staging area. The staff confirmed that the licensee measured the distance to the staging area from the edge of the VFDR cable tray, rather than this exposed cable. Provide additional information about this configuration and how it could affect the conclusions of the analysis.
- w. During the audit, the staff discussed the fire modeling analysis performed in fire zone CB4. Part of this analysis utilized CFAST to calculate the HGL temperature, which was used to determine whether damage would propagate outside the originating transient zone. Since CFAST is a zone fire model, the HGL temperature that is calculated is an average temperature for the upper gas layer in the compartment. In reality, a higher HGL temperature will be observed closer to the fire source than far away from the fire source. The staff request that the licensee provide additional information to confirm that the fire postulated will not lead to damage beyond the originating transient zone.

It is of interest that the limits of applicability, with respect to room dimension aspect ratios for CFAST, were slightly exceeded for the L/H ratio. Explain why this is acceptable.

- x. Provide the CFAST input files in electronic format for 9 selected fire zones (that were walked down during the site audit on June 5, 2012) that utilized detailed fire modeling

calculations, i.e., the input files for CB10, CB12, CB18, IB11, CB15, CB04, CB06, AB01 (21.02), and IB25 (01.02).

FM RAI 02

NFPA 805, Section 2.5, requires damage thresholds be established to support the performance-based approach. Thermal impact(s) must be considered in determining the potential for thermal damage of SSCs. Appropriate temperature and critical heat flux criteria must be used in the analysis.

Section C.10 of Calculation DC0780B-001 includes a discussion about the targets. It is reported that there is approximately 10% unknown cable and the rest is known to be thermoset, and the whole plant is considered to have only thermoset cable.

Provide the following information:

- a. For the 10% of unknown cabling, characterize the installed thermoset and thermoplastic cabling in the power block. Specifically with regard to the critical damage threshold temperatures and critical heat flux threshold as described in NUREG/CR-6850. Provide a statement regarding the extent of installed thermoplastic cable insulation.
- b. If necessary, explain how raceways with a mixture of thermoset and thermoplastic cables were treated in terms of damage thresholds and HRR and fire propagation.
- c. If thermoplastic cabling is present, discuss the impact on ZOI size due to increased HRR and fire propagation.
- d. If thermoplastic cabling is present, discuss self-ignited cables and their impact to additional targets created.
- e. Explain if and how covered, partially covered, or holes in closed raceways affected the damage thresholds of cables used in the analysis.
- f. Describe how the damage thresholds for non-cable components (i.e., pumps, valves, electrical cabinets, etc.) were determined. Describe whether there were any non-cable components that were assigned damage thresholds different from those for cables and, if so, explain how those thresholds have been evaluated for damage and ignition.
- g. If more targets are identified, describe the impact to core damage frequency (CDF) and large early release frequency (LERF), as well as changes in CDF (Δ CDF) and changes in LERF (Δ LERF) for those fire zones affected.

FM RAI 03

NFPA 805, Section 2.7.3.2, "Verification and Validation," states: "Each calculational model or numerical method used shall be verified and validated through comparison to test results or comparison to other acceptable models."

Section 4.5.1.2, "FPRA Quality" of the Transition Report states that fire modeling was performed as part of the FPRA development (NFPA 805 Section 4.2.4.2). Reference is made to Attachment J, "Fire Modeling V&V," for a discussion of the verification and validation (V&V) of the fire models that were used.

Furthermore Section 4.7.3 "Compliance with Quality Requirements in Section 2.7.3 of NFPA 805" of the Transition Report states that, "Models and numerical methods used in support of compliance with 10 CFR 50.48(c) were verified and validated as required by Section 2.7.3.2 of NFPA 805."

Regarding the V&V of fire models:

- a. It has been mentioned in both Generic Methodology calculations (DC0780B-001 and DC0780C-001) that a tool called VC Summer Fire Modeling Database has been developed using Microsoft® Access. Explain how it was developed along with the underlying mathematical bases for this package and the supporting spreadsheets that were used to perform the FDT calculations. In addition, explain how this package and the supporting spreadsheets were verified.
- b. Provide technical details to demonstrate that the fire models have been applied within the validated range of input parameters, or to justify the application of the model outside the validated range reported in NUREG-1824 or other V&V basis documents.
- c. In Attachment J of the LAR, the normalized parameter L_f/H_c was calculated and reported to be within range for the four fire areas that had performance-based calculations, per Section 4.2.4.1 of NFPA 805. The most recent draft of NUREG 1934, "Nuclear Power Plant Fire Modeling Application Guide," which explains how to calculate these parameters, corrects a clerical mistake for this normalized parameter. This parameter should be $(L_f+H_f)/H_c$ (to account for fire elevation). Please confirm that this normalized parameter was calculated correctly and if not, describe whether the new value is within the parameter's range of applicability. Provide justification for the use of the fire model in cases when the new parameter value is outside the validation range.
- d. It appears that these normalized parameters were only calculated for fire zones where the methodology described in Section 4.2.4.1 of NFPA 805 was used. Provide a discussion of the normalized parameters for all fire zones where fire modeling was conducted.
- e. During the audit, the staff observed that the spreadsheets included in Attachment 1 (Supporting Documents) of the reports for the zones where detailed fire modeling was performed show hand calculations. The equation that was used to calculate the plume temperature at a specified height shown on some printouts (e.g. one of the printouts in the report for zone AB01.21.02) is different from that shown on the 09_Plume_Temperature_Calculations FDT spreadsheets. Provide the origin of the version of Heskestad's equation that was used and explain how this equation was verified and validated.

FM RAI 04

NFPA 805, Section 2.7.3.3, "Limitations of Use," states: "Acceptable engineering methods and numerical models shall only be used for applications to the extent these methods have been subject to verifications and validation. These engineering methods shall only be applied within the scope, limitations, and assumptions prescribed for that method"

Section 4.7.3, "Compliance with Quality Requirements in Section 2.7.3 of NFPA 805," of the Transition Report states that "Engineering methods and numerical models used in support of compliance with 10 CFR 50.48(c) were and are used with the same limitations and assumptions supported by the V&V for the methods as required by Section 2.7.3.3 of NFPA 805."

Regarding the limitations of use, identify uses, if any, of the fire modeling tools outside the limits of applicability of the method and for those cases explain how the use of the fire modeling approach was justified. An example of this limit of applicability issue can be referenced in RAI 01, item v.

FM RAI 05

NFPA 805, Section 2.7.3.4, "Qualification of Users," states: "Cognizant personnel who use and apply engineering analysis and numerical models (e.g., fire modeling techniques) shall be competent in that field and experienced in the application of these methods as they relate to nuclear power plants, nuclear power plant fire protection, and power plant operations."

Section 4.5.1.2, "FPRA Quality" of the Transition Report states that fire modeling was performed as part of the Fire PRA development (NFPA 805 Section 4.2.4.2). This requires that qualified fire modeling and PRA personnel work together. Furthermore, Section 4.7.3, "Compliance with Quality Requirements in Section 2.7.3 of NFPA 805," of the Transition Report states:

"Cognizant personnel who use and apply engineering analysis and numerical methods in support of compliance with 10 CFR 50.48(c) was competent and experienced as required by Section 2.7.3.4 of NFPA 805.

This requirement will continue to be met by adherence to SCE&G procedures and project management of contractor support staff. For personnel performing fire modeling or FPRA development and evaluation, VCSNS and contract personnel developed and maintained project instructions to be used by individuals assigned various tasks, to ensure consistency of the engineering and PRA products. These instructions were developed by personnel with intimate knowledge and experience in the task subject matter. Task specific instructions were developed to identify and document required training and mentoring to ensure individuals are appropriately qualified per the requirements of NFPA 805 Section 2.7.3.4 to perform assigned work."

Regarding qualifications of users of engineering analyses and numerical models:

- a. Describe what constitutes the appropriate qualifications for the VCS, Unit 1 staff and consulting engineers to use and apply the methods and fire modeling tools included in the engineering analyses and numerical models.

- b. Describe the process/procedures for ensuring the adequacy of the appropriate qualifications of the engineers/personnel performing the fire analyses and modeling activities post-transition.
- c. Explain the communication process between the fire modeling analysts and PRA personnel to exchange the necessary information and any measures taken to assure the fire modeling was and will be performed adequately.

FM RAI 06

NFPA 805, Section 2.7.3.5, "Uncertainty Analysis," states: "An uncertainty analysis shall be performed to provide reasonable assurance that the performance criteria have been met." NFPA 805, Section 1.6.9, defines "Completeness Uncertainty" as "uncertainty in the predictions of the model due to model scope limitations. This uncertainty reflects an unanalyzed contribution or reduction of risk due to limitations of the available analytical methods."

NFPA 805, Section 1.6.40, defines "Model Uncertainty" as uncertainty in the predictions of a model related to the equations in the model being correct, whether or not they are appropriate to the problem being solved, and whether or not they are sufficiently complete."

NFPA 805, Section 1.6.43, defines "Parameter Uncertainty" as uncertainty in the predictions of a model due to uncertainties in the numerical values of the model parameters." Section 4.7.3, "Compliance with Quality Requirements in Section 2.7.3 of NFPA 805," of the Transition Report states that "The impact of important uncertainties on the Fire PRA results was established using extensive, well formulated sensitivity studies to provide reasonable assurance that the performance criteria have been met as outlined in Section A of 2.7.3.5 of NFPA 805."

Regarding the uncertainty analysis for fire modeling:

- a. Describe how the uncertainty associated with the fire model input parameters (compartment geometry, radiative fraction, etc.) was accounted for in the analysis?
- b. Describe how the "model" and "completeness" uncertainties were accounted for in the analysis. An example of completeness of an uncertainty issue is described in FM RAI 01, item b.

FM RAI 07

Section 4.5.2.1, "Fire Modeling Approach" of the Transition Report discusses the performance-based approach via fire modeling. Reference is made to Attachment J, "Fire Modeling V&V," for a discussion of the acceptability of the fire models that were used.

Directly after this reference to Attachment J, there is a note, which states, "At VCSNS, the use of the fire modeling option (see NFPA 805, Section 4.2.4.1) to disposition potential variances from deterministic requirements follows a predefined process documented in the NFPA 805 project instructions or SCE&G design guides. The objective of these documents is to provide the framework for the use of fire modeling both during the NFPA 805 transition and in the future while the plant operates under NFPA 805 licensing basis. Consistent with the fire modeling

requirements in NFPA 805, these documents allow for the use of fire models that are verified and validated within a range of applications. Consequently, the fire models available for use at VCSNS when operating under NFPA 805 are not limited to the ones selected for supporting the transition. Fire models that are verified and validated (e.g., FDS, CFAST, FDTs, etc.) and are exercised within the corresponding application range may be used in the future following the process outlined in the project instructions and in accordance with the requirements of NFPA 805.”

The staff does not agree that a fire modeling methodology, which has not been previously reviewed by the NRC, can be used for a future plant change evaluation. The note in the Transition Report should be revised to only allow the future use of fire model methodologies that have been reviewed by the NRC for use in performance-based fire modeling applications in accordance with NFPA 805 Section 4.2.4.1.

FM RAI 08

NFPA 805, Section 2.4.4.3, "Safety Margins," states: "The plant change evaluation shall ensure that sufficient safety margins are maintained. The deterministic approach for meeting the performance criteria shall be deemed to satisfy this safety margins requirement."

NFPA 805, Section 4.2.4.1.5, "Protection of Required Nuclear Safety Success Path(s)" discusses the required safety margins in the context of the difference between the maximum expected fire scenario (MEFS) and the limiting fire scenario (LFS). This margin is to account for uncertainties and unknowns in the analytical process and to ensure adequate defense-in-depth is provided and nuclear safety performance criteria are met and maintained.

Provide a discussion of the uncertainties and margin that exist for fire areas utilizing the performance-based approach and a technical justification for how the analysis meets the regulatory requirements.

FM RAI 09

During the audit and based on review of the detailed fire modeling reports supporting the performance-based fire modeling approach in fire areas CB10, CB12, CB18 and IB11, the staff learned that the use of a detailed administrative controls methodology in the performance-based fire modeling analyses is being credited.

However, it is not clear whether this approach has adequately considered all of the necessary aspects to ensure that a post-transition administrative controls program will be implemented such that the technical assumptions included in the fire modeling calculations are not violated. For instance, it is not clear whether a height limitation will be included in an administrative procedure to ensure that combustible materials stored in transient staging areas do not exceed the values used in the analysis.

Provide a discussion on the bases used to develop the performance-based fire modeling scenarios and a description of and a commitment for how the post-transition procedures will maintain a program that is consistent with the fire modeling calculations.

In addition, provide a discussion of the DID concept that was applied in areas where performance-based fire modeling was used such that administrative control procedures are not the only form of protection in a given fire area.

PRA RAI 01

Describe how the evaluation includes the possible increase in HRR caused by the spread of a fire from the ignition source to other combustibles. For example, on page 60 of 73 in the Generic Fire Methodology report, a qualitative argument is made that the second cable tray will not get ignited for a fire in the initial cable tray from a hot work induced cable tray fire. Perform a quantitative calculation to verify this conclusion. The same type of conclusion is drawn for a junction box fire. Perform the same type of quantitative verification. If you are unable to support these qualitative arguments, provide a sensitivity study as to the importance of these conclusions to your PRA results.

PRA RAI 02

Transient fires should at a minimum be placed in locations within the plant physical access units (PAUs) where conditional core damage probabilities (CCDPs) are highest for that PAU, i.e., at "pinch points." Pinch points include locations of redundant trains or the vicinity of other potentially risk-relevant equipment, including the cabling associated with each. Transient fires should be placed at all appropriate locations in a PAU where they can threaten pinch points. Hot work should be assumed to occur in locations where hot work is a possibility, even if improbable (but not impossible), keeping in mind the same philosophy. Describe how transient and hot work fires are distributed within the PAUs. In particular, identify the criteria that determines where an ignition source is placed within the PAUs. Also, if there are areas within a PAU where no transient or hot work fires are located since those areas are considered inaccessible, define the criteria used to define "inaccessible." Note that an inaccessible area is not the same as a location where fire is simply unlikely, even if highly improbable.

PRA RAI 03

Discuss the calculation of the frequencies of transient and hot work fires. Characterize the use of the influence factors for maintenance, occupancy, and storage, noting if the rating "3" is the most common, as it is intended to be representative of the "typical" weight for each influence factor. It is expected that the influence factor for each location bin associated with transient or hot work fires will utilize a range of influence factors about the rating "3," including the maximum 10 (or 50 for maintenance) and, if appropriate, even the rating "0." Note that no PAU may have a combined weight of zero unless it is physically inaccessible, administrative controls notwithstanding. In assigning influence factor ratings, those factors for the Control/Auxiliary/Reactor Building are distinct from the Turbine Building; thus, the influence factor ratings for each location bin are to be viewed according to the bin itself.

PRA RAI 04

If any influence factors that were used were outside of the values identified in Table 6-3 of NUREG/CR-6850, identify the values used, identify the PAUs that use these factors, and justify

the assigned factor(s).

PRA RAI 05

Describe the methodology that was used to evaluate DID and that was used to evaluate SMs. The description should include what was evaluated, how the evaluations were performed, and what, if any, actions or changes to the plant or procedures were taken to maintain the philosophy of defense-in-depth or sufficient safety margins.

PRA RAI 06

The Transition Report summarizes equipment available to achieve and maintain safe and stable conditions but provides limited information about how long the facility can be maintained in those conditions using equipment that is free from fire damage and actions that can be taken from the CR. The submittal indicates that safe and stable conditions cannot be maintained indefinitely using this equipment. Provide a discussion about how long safe and stable conditions can be maintained using the undamaged equipment, and actions necessary to maintain these conditions once realignment and/or resupply activity such as refilling fluid tanks or re-aligning systems is required. If safe and stable conditions cannot be maintained indefinitely, evaluate at least qualitatively, and preferably quantitatively, the risk associated with the failure of actions and equipment necessary to extend these conditions indefinitely.

PRA RAI 07

Section 10 of NUREG/CR-6850 Supplement 1 states that a sensitivity analysis should be performed when using the fire ignition frequencies in the Supplement instead of the fire ignition frequencies provided in Table 6-1 of NUREG/CR-6850. Provide the sensitivity analysis of the impact on using the Supplement 1 frequencies instead of the Table 6-1 frequencies on CDF, LERF, Δ CDF, and Δ LERF for all of those bins that are characterized by an alpha that is less than or equal to one. If the sensitivity analysis indicates that the change in risk acceptance guidelines would be exceeded using the values in Table 6-1, justify not meeting the guidelines.

PRA RAI 08

Describe how CDF and LERF are estimated in MCR abandonment scenarios. Describe if any fires outside of the MCR cause MCR abandonment because of loss of control and/or loss of CR habitability. Describe whether "screening" values for post MCR abandonment are used (e.g., conditional core damage probability of failure to successfully switch control to the PCS and achieve safe shutdown of 0.1) or have detailed human error analyses been completed for this activity. Justify any screening value used.

PRA RAI 09

It was recently stated at the industry fire forum that the Phenomena Identification and Ranking Table Panel (PIRT) being conducted for the circuit failure tests from the DESIREE-FIRE and CAROL-FIRE tests may be eliminating the credit for Control Power Transformers (CPTs) (about a factor 2 reduction) currently allowed by Tables 10-1 and 10-3 of NUREG/CR-6850, Vol. 2, as being invalid when estimating circuit failure probabilities. Provide a sensitivity analysis that

removes this CPT credit from the PRA and provide new results that show the impact of this potential change on CDF, LERF, Δ CDF, and Δ LERF. If the sensitivity analysis indicates that the change in risk acceptance guidelines would be exceeded after eliminating CPT credit, please justify not meeting the guidelines.

PRA RAI 10

Attachment W of the LAR provides the Δ CDF and Δ LERF for the VFDRs for each of the fire areas, but the LAR does not describe either generically or specifically how Δ CDF and Δ LERF were calculated. Describe the method(s) used to determine the changes in risk reported in the Tables in Appendix W. The description should include:

- a. A summary of PRA model additions or modifications needed to determine the reported changes in risk. If any of these model additions used data or methods not included in the FPRA Peer Review, describe the additions.
- b. Identification of new operator actions (not including post MCR abandonment which are addressed elsewhere) that have been credited in the change in risk estimates. If such actions are credited, how is instrument failure addressed in the HRA.

PRA RAI 11

Describe whether the peer reviews for both the internal events PRA (IEPRA) and FPRAs consider the clarifications and qualifications from Regulatory Guide (RG) 1.200, Revision 2, "An Approach for Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities," March 2009 to the American Society of Mechanical Engineers/American Nuclear Society (ASME/ANS) PRA Standard. If not, provide a self-assessment of the PRA model for the RG 1.200 clarifications and qualifications and indicate how any identified gaps were dispositioned.

PRA RAI 12

Identify if any VFDRs in the LAR involved performance-based evaluations of wrapped or embedded cables. If applicable, describe how wrapped or embedded cables were modeled in the FPRA including assumptions and insights on how the PRA modeling of these cables contributes to the VFDR delta-risk evaluations.

PRA RAI 13

Identify any plant modification (implementation item) in Attachment S of the LAR that has not been completed but has been credited directly or indirectly in the change-in-risk estimates provided in Attachment W. When the effect of a plant modification item has been included in the PRA before the modification has been completed, the models and values used in the PRA are necessarily estimates based on current plans. The as-built facility after the modification is completed may be different than the plans. Add an implementation item that, upon completion of all PRA credited implementation items, verifies the validity of the reported change-in-risk. This item should include the plan of action should the as-built change-in-risk exceed the estimates reported in the LAR.

PRA RAI 14

Identify any changes made to the IEPRA or FPRA since the last full-scope peer review of each of these PRA models that are consistent with the definition of a "PRA upgrade" in ASME/ANS-RA-Sa-2009, as endorsed by Regulatory Guide 1.200. Also, address the following:

- a. If any changes are characterized as a PRA upgrade, identify if a focused-scope peer review was performed for these changes consistent with the guidance in ASME/ANS-RA-Sa-2009, as endorsed by Regulatory Guide 1.200, and describe any findings from that focused-scope peer review and the resolution of these findings for this application.
- b. If a focused-scope peer review has not been performed for changes characterized as a PRA upgrade, describe what actions will be implemented to address this review deficiency.
- c.

PRA RAI 15

FSS-A4-01: According to DC000340-001, the multi-compartment analysis (MCA) postulated that a fire must burn 20 minutes to damage equipment across spatial separation or into an adjacent compartment. Provide justification for this assumption. Ensure that you discuss why damage may not occur in less than the time assumed, in this case 20 minutes.

PRA RAI 16

FSS-A4-02: According to the plant disposition, the individual fire zones have been updated to address the peer review concern that suppression credit is not adequately justified. The plant disposition indicates that this discussion has been added, but provides no technical response about the various types of suppression included, the criteria for its credit, and how it is credited. As a result, provide a technical response which addresses these issues for your crediting of fire suppression in your FPRA.

PRA RAI 17

FSS-C7-01: The peer review finding indicates that dependencies between automatic and manual suppression systems are not addressed in the FPRA. The plant disposition indicates this is addressed, but provides no technical response. Provide a technical justification that the dependency between automatic and manual suppression is addressed in the FPRA.

PRA RAI 18

FSS-D3-01: The peer review indicated that in many cases detailed fire modeling was not performed, leading to conservative results. The plant disposition indicates that a set of scenarios in the auxiliary building (AB) had only received preliminary screening analysis at the time of the peer review, yet now has been done. No technical response has been provided in the plant disposition. As a result, describe those scenarios which were added to evaluate the AB, characterizing their ignition, damage, suppression, CCDP, and CDF/LERF.

PRA RAI 19

FSS-D8-01: The peer review found that credit for detection and suppression was not documented and thus rendered a Not Met to the corresponding SR. Your response only references dispositions to other facts and observations (F&Os) which also need additional technical discussion added. Provide a general technical description of the extent of crediting these fire protection features.

PRA RAI 20

FSS-D9-01: According to the plant disposition, the discussion on smoke effects treatment in the FPRA was expanded as a result of the peer review finding. Provide a more complete technical response identifying the short term damage which you included in your FPRA.

PRA RAI 21

FSS-G2-01: The plant disposition indicates that damage to sensitive electronics has been incorporated in the FPRA. Indicate those components that meet the criteria for sensitive electronics and justify why this damage is not assumed beyond the CR and relay room as stated in Appendix V. For example, in many cases, switchgear has electronic undervoltage, overvoltage, under/over frequency and other types of sophisticated relays that are just as temperature sensitive as control modules. In your response, indicate what other electrical cabinets are installed in plant areas other than the MCR and relay room. As appropriate, provide an assessment on the additional effects of sensitive electronics on your FPRA.

PRA RAI 22

FSS-H5-01: The peer review identified that parameter uncertainty evaluations were not performed. Counter to the plant disposition as stated in Appendix V, discussions during the audit revealed that these evaluations were not performed. Since this analysis is needed to achieve CCII, perform these studies and provide the results to achieve CCII. If CCI is all that is desired, justify the adequacy of CCI. In this justification, explain why CCI does not lead to underestimation of Δ CDF, Δ LERF, CDF, and LERF.

PRA RAI 23

CS-A10-01: Provide a discussion on when routing is assumed. In particular, what is the routing assumed between cable trays in a single PAU or between PAUs. Describe how the assumed routing is treated when establishing fire PRA targets.

PRA RAI 24

PP-B2-01: This fact and observation (F&O) is listed as a suggestion, but it is the only F&O describing PP-B3 which is attributed a "Not Met" by the peer review. The peer review suggestion identifies that spatial separation as a partitioning feature is not justified for certain cases. The justification is not provided in the plant partitioning document. Provide a description

of the configurations for those configurations identified in the F&O, and justify spatial separation as a partitioning feature. The plant disposition does not address the partitioning, yet indicates that the multi-compartment analysis takes into consideration scenarios which accounts for the interactions across this spatial separation.

PRA RAI 25

ES-A6-01: Item 4, pg. 14 of the FPRA plant final report indicates that at a maximum 2 spurious operations are included. With respect to ES-A6, this implies that CCII was not met. The peer review rating in table V-16 confirms that CCI was assigned, but the F&O ES-A6-01 is not provided in the LAR. Provide this F&O and the plant disposition. Justify the achieved capability category.

PRA RAI 26

For F&O PRM-A4-01, summarize how, for a given fire scenario, the appropriate initiator is selected (due to impacted equipment) and the related mitigation system fault tree logic is valid.

PRA RAI 27

F&O PRM-A4-02 identifies improper modeling of multiple spurious operation (MSO) scenarios. The disposition only states the model was reviewed for accuracy, but does not identify if the errors flagged by the peer review team are corrected, if other errors were corrected, etc. Clarify the disposition of this item.

PRA RAI 28

F&O PRM-A4-04 identifies MSO modeling errors based specifically on a sample and not a 100% review. The disposition of this item identifies correction of errors and a review of MSOs for additional issues. Describe the scope of the review performed to determine the extent of condition.

PRA RAI 29

For F&O PRM-A4-05, the disposition of this item only states that a particular set of logic was modified and included in the fault tree appropriately. Discuss how each technical point raised by the peer review team was addressed, as well as how the extent of condition was determined given that the peer review only identified an example.

PRA RAI 30

Deleted

PRA RAI 31

IGN-B5-01. According to the peer review finding, a qualitative discussion of the sources of uncertainty is needed to address this standard support requirement (SR). The peer review cites Appendices U and V of NUREG/CR-6850 as guidance for this issue. The plant disposition

solely identifies the distributions used for the calculation, but does not address the peer review comment. Identify the assumptions and sources of uncertainty associated with the fire frequency analysis as required by the supporting requirement, IGN-B5, of the ASME/ANS PRA Standard.

PRA RAI 32

Deleted

PRA RAI 33

For F&O HRA-C1-01, describe how dependencies were removed from the charging pump swap in the recovery action.

PRA RAI 34

For F&O HRA-C1-02, provide the basis for the time required to perform a manual action inside and outside the CR for a fire, including validation or reference to an accepted methodology. Describe when and where the basis will be documented.

PRA RAI 35

Disposition in Attachment V, Table V-18 for supporting requirement SF-A4-01 indicates plant procedures for fire brigade drills and natural emergencies have not been updated to address seismically induced fire. Confirm that procedures are now updated. The supporting requirement in the ASME/ANS Standard also requires a qualitative assessment of the potential that a seismically induced fire, or spurious operation of fire suppression systems, might compromise a post-earthquake plant response. Provide this assessment based on the updated procedures.

PRA RAI 36

Finding in Attachment V, Table V-18 for supporting requirement ES-A4-01 establishes a deficiency in the linkage between the Component-BE table in the FRANX database and computer aided fault tree analysis (CAFTA) for spurious operation of pressurizer spray valves. The response is not clear as to whether the spray valve failure mode was in the model (and the finding is invalid) or whether the model was corrected. Clarify this response. In addition, the disposition addresses one particular scenario the peer reviewer discovered and provides small confidence this problem will not appear again for other components. Describe the root cause of this problem and provide verification that this issue will not be repeated for other scenarios and that all pertinent components included in the FRANX database are linked to the CAFTA fault tree.

PRA RAI 37

Finding in Attachment V, Table V-18 for supporting requirements ES-B1-01 and ES-B1-03 identify apparent significant errors in the underlying data used in quantification of fire risk. Specifically, the peer review identified records referring to nonexistent basic events in the PRA

model, and PRA model basic events representing potentially legitimate fire failure modes which were never identified as being impacted. Concerns were also identified as to the manual methods of generating these files, as opposed to a database query method. While the disposition of these F&Os identifies a review of the mapping results and addition of documentation to any unmapped basic event, there is no discussion as to the impact of any corrections to the risk results. Further, the staff assumes that the original databases were reviewed, but the errors still were not found. It is not clear if the review conducted to resolve the F&Os was somehow more comprehensive than prior reviews to ensure identification of errors. Discuss the impact of these F&Os on the FPRA due to model changes resulting from the re-review of the mapping.

PRA RAI 38

Finding in Attachment V, Table V-18 for supporting requirement ES-B3-01 identifies that containment penetrations screened in the IEPRAs based on small size were not fully reviewed for inclusion in the FPRA. It also identifies differences in the CDF model and the LERF model used for the FPRA, and changes needed to screening criteria for containment penetrations. The disposition only states that additional penetrations have been identified, but does not discuss either that the FPRA model has been revised to correct the issues or provide any basis for why the existing FPRA LERF model is adequate. It does not discuss if the CDF and LERF models remain different, nor does it discuss any changes to screening criteria. Discuss how this finding was addressed, including identifying if changes to the model were made, and justify the technical adequacy of the LERF model used for the FPRA.

PRA RAI 39

Finding in Attachment V, Table V-18 for supporting requirement ES-B4-01 establishes a deficiency in modeling of support equipment whose fire-induced failure could adversely affect primary equipment. The disposition addresses one particular finding associated with power dependency and provides small confidence this problem will not appear again for other support equipment. Describe the root cause of this problem and provide verification that this issue will not be repeated for other support equipment listed in SR ES-B4.

PRA RAI 40

Disposition in Attachment V, Table V-18 for supporting requirement CS-A8 identifies tasks to address the item, but there is nothing to indicate these items have been completed or what the resulting model impacts were, or if not completed why the deficiency is not significant to this application. Provide additional details for this item.

PRA RAI 41

Finding in Attachment V, Table V-18 for supporting requirement CS-B1-01 identifies open items in Attachment D of a licensee report need to be addressed. The disposition does not state that these open items are addressed. Clarify the disposition of this item.

PRA RAI 42

For F&O DA-02, the acceptable basis for using plant data only is that the quantity of data is sufficient to characterize the parameter value and its uncertainty. Provide confirmation that this is the basis for accepting plant data as a sole source and ignoring generic evidence, as required to conform to capability category II of the standard. Illustrate that this “does not impact the development of a FPRA” by performing a sensitivity analysis to demonstrate that the 3 cases identified by the peer review as having significant differences would not impact the FPRA results.

PRA RAI 43

F&O DA-03 identifies that “non-fatal” common cause failures were added to the model, but does not affect the FPRA. If there are more than two components, then a fire-induced failure of one component combined with a non-fatal common cause failure (CCF) of the remaining components would now be a possible fire failure scenario. Describe the basis as to why this resolution does not impact the FPRA.

PRA RAI 44

For F&O DA-08, describe whether the first issue was resolved via a RA or data screening. Provide a description of the basis for the resolution. For the second issue, provide justification that a common suction path that may lead to steam binding, air binding or debris clogging of both pumps does not exist. Provide justification for the CCF data used to address the second issue. For the third issue, data for this failure mode is in the Idaho National Engineering and Environmental Laboratory (INEEL) CCF database. Provide justification for why it was not used. For the fourth issue, describe whether CCF for emergency diesel generator (EDG) fuel transfer pumps is included in the station blackout SBO model.

PRA RAI 45

For F&O HR-02, describe whether mis-calibration common cause events were added to the model.

PRA RAI 46

For F&O HR-03, provide the justification and reference for the time window used in the HRA calculation for feed and bleed actions.

PRA RAI 47

For F&O HR-05, provide a description of the revision made to the HRA calculation to address these issues, including the basis for assigning the level of dependency.

PRA RAI 48

For F&O HR-06, characterize the human actions reviewed with the operators and summarize the results of the interviews.

PRA RAI 49

For F&O HR-08, provide a description of the time-reliability models that were performed for human failure events (HFEs) with short time windows. Describe whether this is the same model used in the FPRA HRA. Provide a basis for why only one HRA probability had to be updated.

PRA RAI 50

For F&O HR-01-2007, SR HR-01 requires that once the overall HRA has been completed, the plant should perform a review of their HEPs for internal consistency with respect to scenario, context, procedures and timing. Describe when this requirement will be incorporated into plant guidance. If it will not be incorporated into plant guidance, provide a basis for the statement, "This review is performed at each HRA update, although it is not currently a specific requirement in the guideline."

PRA RAI 51

For F&O QU-04, provide a description of the changes made to the PRA guidance to ensure multiple operator action strings are evaluated for dependence after each change in the PRA HRA.

PRA RAI 52

For F&O QU-06, provide a basis for the conclusion that performing these updates after each major revision does not have a negative impact on the FPRA.

PRA RAI 53

For F&O QU-07, the disposition does not address the documentation portion. Describe what actions have been taken to ensure that insights about the contributors to risk, key plant features that impact the results, any unique or specific modeling approaches that influence the results, and results of parametric uncertainty are included in this and future results summaries.

PRA RAI 54

For F&O L2-02, it is not clear from the disposition whether the PRA model includes early containment over pressure failures as discussed in NUREG/CR-6595, "An Approach for Estimating the Frequencies of Various Containment Failure Modes and Bypass Events". Describe what is so unique about the containment that this failure mode can be discounted. Provide a summary discussion of the plant-specific basis as to why this failure mode cannot occur (or is very unlikely to occur), and include a discussion of fire-induced core damage sequences and their relation to this containment failure mode.

PRA RAI 55

For F&O HR-04-GA, summarize the discussions referenced in the response that 1) Discuss the specific factors considered when evaluating the dependency between actions; and, 2) Indicate

the basis for assigning the dependency levels for the second and subsequent actions in a set, especially for the LD and ZD dependencies.

PRA RAI 56

For F&O DA-01-GA, provide a markup of the revised data update guideline so that the reviewer may understand the redefined process and rules used.

PRA RAI 57

Deleted

PRA RAI 58

For F&O QU-01-GA, describe whether there are any cutsets containing multiple maintenance actions that were inappropriate. If so, describe what action was taken.

PRA RAI 59

Deleted

PRA RAI 60

Deleted

PRA RAI 61

For F&O QU-05-GA, describe whether the definition of “significant” in your quantification guidelines is consistent with Section 2 of the Standard. Describe whether the definition of “significant” impacted guidance for documenting assumptions and sources of uncertainty as well as the review of significant cutsets and accident sequences. If so, describe how. If not, describe why not.

PRA RAI 62

Finding in Attachment U, for supporting requirement AS-01 identifies several deficiencies for the interfacing systems loss of coolant accident (ISLOCA) analysis centering around giving credit for successful prevention of core damage and large releases. Specifically, the F&O identified 1) crediting success of high pressure injection and recirculation given depressurization and makeup long term from an external source, 2) probabilistic treatment of low pressure piping failures with inadequate documentation, 3) non-piping failure modes not significant, 4) flooding impacts not considered, 5) open-ended makeup without sump recirculation, 6) credit of recirculation and containment cooling. The disposition of the finding states only that large pipe breaks were added to the analysis as going directly to core damage. This does not appear to address the above issues. Provide additional information that fully addresses the scope of the F&O.

PRA RAI 63

Finding in Attachment U, for supporting requirement SY-07 identifies the condensate storage tank (CST) capacity as not satisfying the 24-hour mission time, but no backup or alternate source being modeled. The disposition of this item only states that the licensee documented why this was not required. A further assessment identified (AS-01-GA of Attachment U-2 of the submittal) that the resolution was not sufficient. However, the disposition of this item does not appear to provide any further basis for this critical assumption. Provide additional justification for this item.

PRA RAI 64

Disposition in Attachment U, Table U-2 for supporting requirement IE-01-GA indicates that the finding is not adequately addressed. The ISLOCA frequency calculated using the variance treatment resulted in a factor of twenty higher than the baseline; however, this updated frequency was not included in the model. Confirm that ISLOCA treatment, including frequency calculation and analysis, was revised following the RG 1.200 gap assessment and addresses the weaknesses for the ISLOCA model identified during the peer review.

PRA RAI 65

Finding in Attachment U, Table U-2 for supporting requirement SY-01-GA indicates F&O TH-03 has not been resolved from the original peer review. Table U-1 does not list TH-03 as a finding. Provide the finding associated with TH-03 from the original peer review and the disposition including justification for why room heatup and credit for local operator actions are not modeled in the PRA for electrical equipment rooms IB-63-01 and AB-63-01. In addition, justify how the equipment in the room is deemed operational within the PRA mission time if the temperature in the room exceeds design conditions.

PRA RAI 66

FSS-B2-01: It is not clear from the MCR abandonment document what the criteria are for CR abandonment for main control board (MCB) fires. Secondly, from the MCB scenarios the conditional core damage probability (CCDP) does not appear correct for the specified number of panels as given in Appendix B of the Fire Risk Quantification Task 14 document. For example, for scenario CB 17.01 MCB 11-10-1, 11-9-1, and 11-8-1, 2, 3, and 4 panels are failed respectively, yet the CCDP from the quantification document specified that the CCDP is largest when only 2 panels are damaged. Also for scenarios CB17.01 MCB 18-18-1, 18-17-1, 18-16-1, and 18-15-1 the CCDP is the same for each scenario even though 1, 2, 3, and 4 panels are damaged respectively. Provide the criteria for CR evacuation for MCB fires, and justify the CCDPs provided for the various number of panels damaged. The response should take into account a further examination of the CCDPs for MCB fires than identified in this question to evaluate if the CCDP problem is more extensive than discussed in this question. Provide updated CDF, LERF, Δ CDF, and Δ LERF values for MCB scenarios.

PRA RAI 67

In Table W-3 of the LAR, Fire areas CB10, CB12, and IB11 contain VFDRs, yet the Δ CDF and

Δ LERF are listed as N/A. Since all VFDRs are supposed to have their deltas calculated so that they can be summed and compared to R.G. 1.174, "An Approach for Using Probabilistic Risk Assessment in Risk-Informed Decisions on Plant-Specific Changes to the Licensing Basis," guidelines, explain the N/A.

PRA RAI 68

In the Generic Fire Methodology report, the transient zone describes the zone of damage from the fixed or transient ignition source. According to page 12 of 73 of this report, the transient zone bounds the effect of flame spread or propagation since an extended range of 4 feet beyond the transient zone boundaries was examined for PRA targets. Explain how the transient zone boundary takes into account fire growth over time, or propagation into adjacent transient zones via secondary combustible fires. Should the finding be that the 4 feet margin does not account adequately for these issues, the PRA should be updated accordingly.

PRA RAI 69

On page 34 of 73 of the Generic Fire Methodology report, it is assumed that the growth of electrical motor fires to peak is 6 minutes. Since it is acknowledged that no basis for this can be found in the literature, a sensitivity study should be performed evaluating different growth rates and its effects on the risk. A justification should be provided that the full expected range of the growth time has been explored in this sensitivity study.

PRA RAI 70

Deleted

PRA RAI 71

On page 64 of 73, of the Generic Fire Methodology report for transient fires, a HRR of 15kW and a time of 5 minutes is assumed to damage all targets within the ZOI. Justify that the 15 kW bounds all scenarios, (i.e. that the 15kW is the minimum transient HRR necessary to cause damage for all scenarios). If not, then adjust the minimum HRR for those scenarios which experience damage at less than 15kW and update the FPRA.

PRA RAI 72

On pg 66 of 73 of the Generic Fire Methodology report, per incipient detection systems, the report indicates that credit for incipient detection systems apply to components in low voltage cabinets in selected areas, and to other components expected to demonstrate an incipient fire growth stage. It is further assumed that those low voltage cabinets to which credit is applied do not have fast acting components in them. FAQ 46 only allows credit for low voltage cabinets, providing those components inside are not a part of those fast acting components identified in the FAQ. A review of those cabinet components should be performed and the model modified if any of those components in the cabinets are fast acting. Discuss any variations from FAQ 46 in your treatment of incipient detection credit in the FPRA.

PRA RAI 73

For the SUPP credit on the incipient detection system (ϵ_2), 3.5 minutes is allowed for suppression. The assumption is that this is the time for a fire to grow 2.5 feet above the cabinet. Describe the height of target cables above the cabinets in which incipient detection is installed. Describe whether 2.5 feet is the minimum height. Describe the assumed scenario, taking into account the assumed HRR and indicating its source.

PRA RAI 74

In Step 3 on page F-2 of the LAR, it is stated that, "The inclusion of MSOs in the FPRA is still needed". Clarify that the LAR means that, although screening is not needed, MSOs are still included in the PRA as directed by FAQ 07-0038.

PRA RAI 75

On page 9 of 60 in the CR Risk Calculation report, the HRR profile decays in 5 minutes. Provide justification for this decay time.

PRA RAI 76

A fire brigade arrival time of 10 minutes is assumed in the non-suppression probability for CR abandonment in the CR Risk Calculation. Describe how the 10 minutes compares to the time identified in pre-fire plans for fire brigade arrival to the CR during a fire or during a fire brigade drill. If the 10 minutes is unable to be justified, provide a more realistic time and indicate the impact on the PRA results.

PRA RAI 77

The Feb 2011 follow-on fire peer review F&Os are identified in Table V-18 as "DRAFT". Confirm the final versions are consistent with what is summarized in Table V-18 and that there are no additional F&Os.

PRA RAI 78

The LAR states that the results of the 2005 review were "the VCSNS PRA [internal event PRA] was found to meet CC-II or better for 211 of the 271 SRs from the ASME PRA Standard, but 45 of the elements were found to either not meet the requirement or to meet the requirements at a CC-I level." Describe whether all 60 of the less than CC-II level SR's are included in the tables in Appendix U. If not, describe which ones are missing and why. Add any missing SRs to tables in Appendix U.

PRA RAI 79

Attachment U, table V-18 does not identify a status of the F&Os as was done in Table U-1 for the IEPRAs peer review F&Os. In some instances the disposition states that the FPRA has been

revised but does not confirm if the results presented in the submittal are based on the revised model. In other instances, the disposition indicates that the tasks are not complete. Clarify the status of each item in Table V-18.

PRA RAI 80

Confirm that the revised internal events model which addresses the F&Os was used in the FPRA.

PRA RAI 81

Attachment U, Table U-1 supporting requirement SY-01 addresses modeling of component cooling water support systems. The licensee's disposition indicates that service water and AC/DC power were added to the model; however, does not address loss of instrument air. Clarify how instrument air supports component cooling water system and how this relationship is modeled in the PRA.

PRA RAI 82

In Fire Risk Evaluation A.1.2.33 in the analysis of Power to Bus XMC, it states that those scenarios which are qualitatively or quantitatively screened do not contribute to the Δ CDF or Δ LERF. It also states that those unscreened scenarios are treated in a bounding approach. Describe the meaning of quantitatively screened, and indicate if single or multi-compartment contributions to CDF, LERF, Δ CDF, Δ LERF are screened. If these contributions are screened, quantitatively justify that these screened portions are insignificant with respect to the CDF, LERF, Δ CDF and Δ LERF acceptance guidelines. Otherwise, include the results in the baseline risk values and Δ CDF/ Δ LERF. Perform a review of this issue for the full plant analysis.

PRA RAI 83

In compartment AB01.03 of Attachment A of the Fire Modeling Scoping Report, pump fires from oil spills are assigned a HRR of 767 kW, regardless if the oil spill is 10% or 100% capacity of the pump. Explain why the HRR does not distinguish between oil spill sizes, or provide an updated analysis adjusting the HRR. Examine the rest of the pump oil spills to verify that this distinction is made in the compartment analyses, and correct if necessary.

PRA RAI 84

FSS-D7-01 specifies that outlier experience was not examined for fire detection and suppression systems. The plant disposition indicates that generic values are larger than plant specific values and no outlier behavior exists. Justify the extent of the review of plant specific behavior. Provide a discussion of the plant-specific results and a discussion of the comparison to the generic values.

PRA RAI 85

The plant disposition for UNC-A2-01, 02, and 03 indicate that the peer review finding has been addressed. Provide a technical discussion or analysis, indicating how this issue has been

addressed. In addition, provide a justification for those sensitivity studies that are not consistent with expected results as well as insights gained.

RR RAI 01

For areas where containment/confinement is relied upon:

- a. Liquid
 1. Describe how the qualitative/quantitative assessment addressed capacities of sumps, tanks, transfer pumps, etc., as appropriate.
 2. Describe if there are plant features that may divert the effluent flow that were not taken into account (e.g., Aux. Bld. roll-up doors).

- b. Gaseous
 1. Describe if there are plant features that can bypass the planned filtered/monitored ventilation pathway that have not been accounted for.

RR RAI 02

For areas where containment/confinement is not available describe whether the assessment credits operator actions.

July 26, 2012

Mr. Thomas D. Gatlin
Vice President, Nuclear Operations
South Carolina Electric & Gas Company
Virgil C. Summer Nuclear Station
Post Office Box 88
Jenkinsville, SC 29065

SUBJECT: VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1 (VCSNS) – REQUEST FOR ADDITIONAL INFORMATION (TAC NO. ME7586)

Dear Mr. Gatlin:

By letters dated November 15, 2011 and January 26, 2012, the South Carolina Electric & Gas Company (SCE&G, the licensee) submitted a license amendment request (LAR) to revise Facility Operating License Number NPF-12 for the Virgil C. Summer Nuclear Station, Unit No. 1. The LAR would permit transition of the fire protection licensing basis from Title 10 of the *Code of Federal Regulations (10 CFR)*, Section 50.48(b), to 10 CFR 50.48(c), "National Fire Protection Association Standard NFPA 805" (NFPA 805).

The U.S. Nuclear Regulatory Commission (NRC) staff, including the technical areas of fire protection, probabilistic risk assessment licensing, and health physics and human performance, has reviewed the licensee's information and has also participated in an audit of that information from June 4 to June 8, 2012. The staff has determined that additional information is needed to complete the review, as discussed below. A draft version of these requests for additional information (RAIs) was transmitted on June 8, 2012 (Agencywide Documents Access and Management System No. ML12168A001).

The NRC staff requests that SCE&G respond to these RAIs within 60 calendar days from the date of this letter. Please note that the staff's review is continuing and further RAIs may be developed.

Sincerely,

/RA/

Robert E. Martin, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-395

Enclosure: Request for Additional Information

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