



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 12, 2012

Mr. Mano Nazar
Executive Vice President and
Chief Nuclear Officer
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

SUBJECT: ST. LUCIE PLANT, UNIT 1 – CORRECTION TO AMENDMENT NO. 213
REGARDING EXTENDED POWER UPRATE (TAC NO. ME5091)

Dear Mr. Nazar:

On July 9, 2012, the Nuclear Regulatory Commission issued Amendment No. 213 to Renewed Facility Operating License No. DPR-67 for the St. Lucie Plant, Unit No. 1. This amendment was in response to your application dated November 22, 2010, as supplemented by letters dated February 25, 2012, March 3, March 14, March 22, April 1, April 21, May 17, May 18, May 19 (three letters), May 24, May 27, May 31 (two letters), June 16, June 22, July 5, July 8, July 22, August 5, August 8, August 12, August 18, August 25 (two letters), August 31, September 2 (two letters), September 8 (two letters), September 22, September 23, September 27, September 29, September 30, October 10, October 14, October 20, October 21, October 27, October 31 (six letters), November 1, November 23, November 29, December 1, December 2, December 14, December 27, 2011, January 2, 2012, January 10, January 14, January 25, February 11, February 21, February 29 (three letters), March 6 (two letters), March 8, March 15, March 16, March 22, March 26, 2012.

The amendment increased the licensed core power level for St. Lucie Unit 1 from 2070 megawatts thermal (MWt) to 3020 MWt. This represents a net increase in the core thermal power of approximately 11.85 percent, including a 10-percent power uprate and a 1.7-percent measurement uncertainty recapture, over the then current licensed thermal power level and is defined as an extended power uprate.

After issuance, it became apparent that some pages were incorrect. License page 3 had not been updated with new maximum power level and a coefficient listed in Table 5.6-1 on page 5-6g had not been replaced with a corrected version.

M. Nazar

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The corrected pages are enclosed. We regret any inconvenience this may have caused. If you have any questions regarding this matter, please call me at (301) 415-2788.

Sincerely,

A handwritten signature in black ink, appearing to read "Tracy J. Orf". The signature is fluid and cursive, with the first name "Tracy" being the most prominent part.

Tracy J. Orf, Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-335

Enclosure: Corrected Pages

cc: Distribution via Listserv

applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

A. Maximum Power Level

FPL is authorized to operate the facility at steady state reactor core power levels not in excess of 3020 megawatts (thermal).

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 213 are hereby incorporated in the renewed license. FPL shall operate the facility in accordance with the Technical Specifications.

Appendix B, the Environmental Protection Plan (Non-Radiological), contains environmental conditions of the renewed license. If significant detrimental effects or evidence of irreversible damage are detected by the monitoring programs required by Appendix B of this license, FPL will provide the Commission with an analysis of the problem and plan of action to be taken subject to Commission approval to eliminate or significantly reduce the detrimental effects or damage.

C. Updated Final Safety Analysis Report

The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on March 28, 2003, describes certain future activities to be completed before the period of extended operation. FPL shall complete these activities no later than March 1, 2016, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.

The Updated Final Safety Analysis Report supplement as revised on March 28, 2003, described above, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following issuance of this renewed license. Until that update is complete, FPL may make changes to the programs described in such supplement without prior Commission approval, provided that FPL evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

D. Sustained Core Uncovery Actions

Procedural guidance shall be in place to instruct operators to implement actions that are designed to mitigate a small-break loss-of-coolant accident prior to a calculated time of sustained core uncovery.

TABLE 5.6-1
Minimum Burnup as a Function of Enrichment

Fuel Type	Cooling Time (Years)	Coefficients		
		A	B	C
1	0	-36.6860	22.4942	-1.4413
2	0	-36.1742	16.6000	-0.8958
3	0	-34.7091	23.1361	-1.6204
4	0	-24.5145	21.3404	-1.2444
	2.5	-26.8311	22.5246	-1.5029
	5	-24.7233	20.9763	-1.3246
	10	-23.6285	19.9541	-1.2505
	15	-23.5458	19.9336	-1.3180
	20	-22.4382	19.2460	-1.2629
5	0	-8.1856	14.5275	-0.0719
	2.5	-11.8506	16.1475	-0.3969
	5	-16.5196	18.5309	-0.7837
	10	-13.6831	16.3475	-0.5844
	15	-12.5819	15.6175	-0.5656
	20	-12.6469	15.4575	-0.5906

NOTES:

- To qualify in a "fuel type," the burnup of a fuel assembly must exceed the minimum burnup "BU" calculated by inserting the "coefficients" for the associated "fuel type" and "cooling time" into the polynomial function:

$$BU = A + B \cdot E + C \cdot E^2, \text{ where:}$$

BU = Minimum Burnup (GWD/MTU)

E = Initial Maximum Planar Average Enrichment (weight percent uranium-235)

A, B, C = Coefficients

- Interpolation between values of cooling time is not permitted.

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Sincerely,

/RA/

Tracy J. Orf, Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-335

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