



FPL

Florida Power & Light Company, 6501 South Ocean Drive, Jensen Beach, FL 34957

May 17, 2012

L-2012-212
10 CFR 50.73

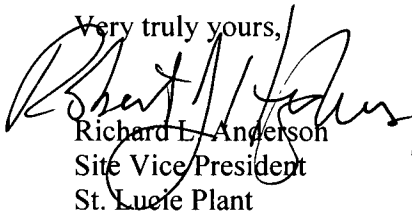
U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Re: St. Lucie Unit 1
Docket No. 50-335
Reportable Event: 2012-002
Date of Event: March 18, 2012

Manual Trip During Physics Testing Due to Unexpected Movement of Control Element Assemblies (CEAs)

The attached Licensee Event Report 2012-002 is being submitted pursuant to the requirements of 10 CFR 50.73 to provide notification of the subject event.

Very truly yours,


Richard L. Anderson
Site Vice President
St. Lucie Plant

RLA/dlc
Attachment

JE22
NRR

NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION (10-2010) LICENSEE EVENT REPORT (LER)	APPROVED BY OMB: NO. 3150-0104 EXPIRES: 10/31/2013 Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA/Privacy Section (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects.resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.
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1. FACILITY NAME St. Lucie Unit 1	2. DOCKET NUMBER 05000335	3. PAGE 1 OF 3
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4. TITLE
Manual Trip During Physics Testing Due to Unexpected Movement of Control Element Assemblies (CEAs)

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIA L NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
3	18	2012	2012	- 002	- 00	05	17	2012	NA	
									FACILITY NAME	DOCKET NUMBER

9. OPERATING MODE 2	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR§: (Check all that apply)									
10. POWER LEVEL < 1%	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)						
	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)						
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)						
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)						
	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)						
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)						
	<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)						
<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER	Specify in Abstract below or in NRC Form 366A						
<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)								

12. LICENSEE CONTACT FOR THIS LER

NAME Don Cecchett - Principal Engineer, Licensing	TELEPHONE NUMBER (Include Area Code) 772-467-7155
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13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU- FACTURE	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU- FACTURE	REPORTABLE TO EPIX
B	AA	ZC	W120	YES					

14. SUPPLEMENTAL REPORT EXPECTED <input checked="" type="checkbox"/> YES (If yes, complete 15. EXPECTED SUBMISSION DATE) <input type="checkbox"/> NO	15. EXPECTED SUBMISSION DATE	MONTH 08	DAY 17	YEAR 2012
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ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On March 18, 2012 at 2336 EDT, while St. Lucie Unit 1 was performing low power testing while in Mode 2 and subcritical at less than 1% power, the reactor was manually tripped due to a control element assembly (CEA) exhibiting unexpected movement. The reactor trip was uncomplicated and all CEAs fully inserted. No automatic safety system actuations were required and none occurred. Reactor coolant system (RCS) heat removal was maintained with auxiliary feedwater and atmospheric dump valves. The Offsite power grid was available and stable.

A root cause evaluation (RCE) for this event is in progress and results will be communicated in a revision to this LER. Corrective actions to prevent recurrence are under development and will be provided in a supplement to this report.

**LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET**

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE
St. Lucie Unit 1	05000335	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	Page 2 of 3
		2012	- 002	- 00	

NARRATIVE

Description of the Event

On March 18, 2012 at 2336 EDT, while St. Lucie Unit 1 was performing low power physics testing while in Mode 2 and subcritical at less than 1% power, the reactor was manually tripped due to a control element assembly (CEA) [ZC] exhibiting unexpected movement. The reactor trip was uncomplicated and all CEAs fully inserted. No automatic safety system actuations were required and none occurred. Reactor coolant system (RCS) heat removal was maintained with auxiliary feedwater and atmospheric dump valves. The Offsite power grid was available and stable.

Cause

The root cause of this event is in progress pending completion of ongoing forensics and evaluation.

Analysis of the Event

The reactor trip was uncomplicated and all CEAs fully inserted. No automatic safety system actuations were required and none occurred. Reactor coolant system (RCS) heat removal was maintained with auxiliary feedwater and atmospheric dump valves. The offsite power grid was available and stable.

The conditional core damage probability (CCDP) and conditional large early release probability (CLERP) values were evaluated and found to be significantly below the thresholds required by RG-1.74 for a non risk-significant event.

A complete analysis of the event will be provided at the conclusion of the ongoing analysis as a supplement to this report.

Analysis of Safety Significance

At the time of the event the plant was in Mode 2. The trip was uncomplicated and all CEAs fully inserted when the reactor was manually tripped while subcritical. No automatic safety system actuations were required and none occurred. Reactor coolant system (RCS) heat removal was maintained with auxiliary feedwater and atmospheric dump valves and the offsite power grid was available and stable.

This licensee event report is being reported in accordance with 10 CFR 50.73(a)(2)(iv)(A) as "an event or condition that resulted in manual or automatic actuation of the Reactor Protection System including reactor scram or reactor trip."

Prompt Corrective Action

Degraded power supplies and ACTM cards have been replaced.

Corrective Actions

**LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET**

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE
St. Lucie Unit 1	05000335	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	Page 3 of 3
		2012	- 002	- 00	

NARRATIVE

The corrective actions will be provided upon completion of the root cause evaluation.

Similar Events

A search of the corrective action database for three years was performed and identified no issues that were related to the faults identified with this Condition Report.

Failed Component (s)

ACTM Input/Output Board - Part No. E-00000-431-015-2

Manufacture

Westinghouse