

Acronyms and Abbreviations

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|---------|--|--------|---|
| AC | Alternating current | EPRI | Electric Power Research Institute |
| ACRS | Advisory Committee on Reactor Safeguards | ESF | Engineered safeguards feature |
| ADS | Automatic depressurization system | ESW | Emergency service water |
| ADV | Atmospheric dump valve | ESWGR | Emergency switchgear |
| AEOD | Office for Analysis and Evaluation of Operational Data | ET | Event tree |
| AFW | Auxiliary feedwater | FCI | Fuel-coolant interaction |
| AOP | Abnormal Operating Procedure | FIVE | Fire-Induced Vulnerability Evaluation |
| AOT | Allowed outage time | FMEA | Failure modes and effects analysis |
| AOV | Air-operated valve | FSAR | Final Safety Analysis Report |
| APB | Accident progression bin | FT | Fault tree |
| APET | Accident progression event tree | F-V | Fussell-Veseley (importance) |
| ASEP | Accident Sequence Evaluation Program | FW | Feedwater |
| ASP | Accident Sequence Precursor | GE | General Electric |
| ATHEANA | A Technique for Human Event Analysis | GL | Generic Letter |
| ATWS | Anticipated transient without scram | GSI | Generic Safety Issue |
| BC | Boundary condition | HCLPF | High confidence, low probability of failure |
| BNL | Brookhaven National Laboratory | HCR | Human Cognitive Reliability |
| BTP | Branch Technical Position | HEP | Human error probability |
| BWR | Boiling water reactor | HHSI | High-head safety injection |
| BWROG | BWR Owners' Group | HLW | High-level waste |
| BWST | Borated water storage tank | HPCI | High-pressure coolant injection |
| CCDF | Complementary cumulative distribution function | HPCS | High-pressure core spray |
| CCDP | Conditional core damage probability | HPI | High-pressure injection |
| CCF | Common-cause failure | HPR | High-Pressure re-circulation |
| CCI | Core-concrete interaction | HPSI | High-pressure safety injection |
| CCW | Component Cooling Water | HRA | Human reliability analysis |
| CDF | Core damage frequency | HVAC | Heating, ventilation, and air conditioning |
| CDF | Cumulative Density Function | HTGR | High-Temperature Gas Reactor |
| CDFM | Conservative Deterministic Failure Margin | HX | Heat exchanger |
| CDP | Core damage probability | ICDP | Incremental core damage probability |
| CE | Combustion Engineering | ICCDP | Incremental conditional core damage probability |
| CEOG | Combustion Engineering Owners' Group | ILERP | Incremental large early release probability |
| CFR | Code of Federal Regulations | ICLERP | Incremental conditional large early release probability |
| CLB | Current licensing basis | IE | Initiating event |
| CRD | Control rod drive | INL | Idaho National Laboratory |
| CSIP | Charging/safety injection pump | INPO | Institute for Nuclear Plant Operations |
| CST | Condensate storage tank | IPE | Individual Plant Examination |
| CW | Circulating water | IPEEE | Individual Plant Examination for External Events |
| DBA | Design basis accident | IREP | Interim Reliability Evaluation Program |
| DC | Direct current | ISA | Integrated Safety Analysis |
| DCH | Direct containment heating | ISI | In-service inspection |
| DF | Decontamination factor | ISLOCA | Interfacing system loss-of-coolant accident |
| DFSD | Dominant functional sequence diagram | IST | In-service testing |
| DHR | Decay heat removal | JCO | Justification for Continued Operation |
| ECCS | Emergency core-cooling system | LB | Licensing basis |
| EDG | Emergency diesel generator | LCO | Limiting Condition for Operation |
| EOOS | Equipment Out of Service System | LER | Licensee Event Report |
| EOP | Emergency Operating Procedure | LERF | Large early release frequency |
| EPA | Environmental Protection Agency | LERP | Large early release probability |
| EPIX | Equipment performance and information exchange system | LLNL | Lawrence Livermore National Laboratory |
| LOCA | Loss-of-coolant accident | LLW | Low-level waste |
| LOOP | Loss of offsite power | LPCI | Low-pressure coolant injection |
| LOSP | Loss of offsite power | LPCS | Low-pressure core spray |
| LP/SD | Low power and shutdown | LPI | Low-pressure injection |
| | | LPR | Low-pressure re-circulation |

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|--------|---|-------|--|
| LPSI | Low-pressure safety injection | RSS | Reactor Safety Study |
| LPZ | Low population zone | RVC | Relief valve re-close |
| LWR | Light water reactor | RWST | Refueling water storage tank |
| MAAP | Modular Accident Analysis Program | S/D | Shutdown |
| MACCS | MELCOR Accident Consequence Code System | SAR | Safety Analysis Report |
| MCS | Minimal cut set | SBO | Station blackout |
| MDP | Motor-driven pump | SDC | Shutdown cooling |
| MGL | Multiple Greek letter | SDP | Significance Determination Process |
| MOV | Motor-operated valve | SER | Safety Evaluation Report (Staff Evaluation Report for IPE/IPEEE) |
| MSIV | Main steam isolation valve | SG | Steam generator |
| MSP | Maintenance and Surveillance Program | SGTR | Steam generator tube rupture |
| MSPI | Mitigating System Performance Index | SHARP | Systematic Human Action Reliability Procedure |
| NCV | Non-cited violation | SI | Safety injection |
| NEI | Nuclear Energy Institute | SIF | Seal injection flow |
| NMSS | Office of Nuclear Materials Safety and Safeguards | SIT | Safety injection tank |
| NOED | Notice of Enforcement Discretion | SLOCA | Small loss-of-coolant accident |
| NPP | Nuclear Power Plant | SNL | Sandia National Laboratory |
| NPRDS | Nuclear Plant Reliability Data System | SPAR | Standardized Plant Analysis Risk |
| NRC | Nuclear Regulatory Commission | SRA | Senior Reactor Analyst |
| NRR | Office Nuclear Reactor Regulation | SRI | Senior Resident Inspector |
| NUMARC | Nuclear Management and Resources Council | SRP | Standard Review Plan |
| OOS | Out of service | SRV | Safety/relief valve |
| ORAM | Outage Risk Assessment and Management | SSC | Systems, structures, and components |
| ORNL | Oak Ridge National Laboratory | SSET | Support state event tree |
| OSHA | Occupational Safety and Health Administration | STG | Source term group |
| P&ID | Piping and instrumentation diagram | SW | Service water |
| PA | Performance assessment | SWGR | Switch gear |
| PCC | PRA Coordinating Committee | TBCCW | Turbine building closed cooling water |
| PCS | Power conversion system | TDP | Turbine-driven pump |
| PDS | Plant damage state | TER | Technical Evaluation Report |
| PM | Preventive maintenance | THERP | Technique for Human Error Rate Prediction |
| PORV | Power-operated relief valve | TRC | Time reliability correlation |
| POS | Plant operating state | USI | Unresolved Safety Issue |
| PRA | Probabilistic risk assessment | VCT | Volume control tank |
| PRT | Plant response tree | WOG | Westinghouse Owners' Group |
| PRV | Pressurizer power-operated relief valves | | |
| PSA | Probabilistic safety assessment | | |
| PSF | Performance shaping factor | | |
| PTFG | PRA Training Focus Group | | |
| PTS | Pressurized thermal shock | | |
| PWR | Pressurized water reactor | | |
| QA | Quality Assurance | | |
| QHO | Quantitative health objective | | |
| QRA | Quantitative risk analysis | | |
| RAW | Risk achievement worth | | |
| RBCCW | Reactor building closed cooling water | | |
| RCIC | Reactor core isolation cooling | | |
| RCP | Reactor coolant pump | | |
| RCS | Reactor coolant system | | |
| RES | Office of Nuclear Regulatory Research | | |
| RG | Regulatory Guide | | |
| RHR | Residual heat removal | | |
| RI | Resident Inspector | | |
| RPS | Reactor protection system | | |
| RPV | Reactor pressure vessel | | |
| RRW | Risk reduction worth | | |