



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

June 22, 2012

LICENSEE: FirstEnergy Nuclear Operating Company  
FACILITY: Davis-Besse Nuclear Power Station  
SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON APRIL 26, 2012, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND FIRSTENERGY NUCLEAR OPERATING COMPANY, CONCERNING REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE DAVIS-BESSE NUCLEAR POWER STATION, LICENSE RENEWAL APPLICATION (TAC. NO. ME4640)

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of FirstEnergy Nuclear Operating Company (FENOC or the applicant) held a telephone conference call on April 26, 2012, to discuss and clarify the staff's requests for additional information (RAIs) concerning the Davis-Besse Nuclear Power Station license renewal application.

Enclosure 1 provides a listing of the participants and Enclosure 2 contains a description of the staff's concerns discussed with the applicant. A brief description on the status of the items is also included.

The applicant had an opportunity to comment on this summary

A handwritten signature in black ink, appearing to read "B. Harris", written over a horizontal line.

Brian K. Harris, Project Manager  
Projects Branch 1  
Division of License Renewal  
Office of Nuclear Reactor Regulation

Docket No. 50-346

Enclosures:

1. List of Participants
2. List of Requests for Additional Information

cc w/encls: Listserv

SUMMARY OF TELEPHONE CONFERENCE CALL  
DAVIS-BESSE NUCLEAR POWER STATION  
LICENSE RENEWAL APPLICATION

LIST OF PARTICIPANTS  
April 26, 2012

PARTICIPANTS

AFFILIATIONS

Samuel Cuadrado de Jesús	U.S. Nuclear Regulatory Commission (NRC)
Allen Hiser	NRC
Christopher Sydnor	NRC
Jeffrey Poehler	NRC
Steve Dort	FirstEnergy Nuclear Operating Company (FENOC)
Larry Hinkle	FENOC
Ashok Nana	FENOC
Mark Rinckel	FENOC
Sarah Davidsaver	FENOC
David Noxon	FENOC

SUMMARY OF TELEPHONE CONFERENCE CALL  
DAVIS-BESSE  
LICENSE RENEWAL APPLICATION  
April 26, 2012

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of FirstEnergy Nuclear Operating Company (FENOC or the applicant) held a telephone conference call on April 26, 2012, to discuss and clarify the following requests for additional information (RAIs) concerning the Davis-Besse license renewal application (LRA).

**RAI 4.2.2-4 Reactor Vessel Upper-Shelf Energy Evaluations**

**Discussion:**

The staff discussed RAI 4.2.2-4 with the applicant. The applicant requested clarification on the options to demonstrate an acceptable upper-shelf energy (USE) for the beltline materials. After some discussion, the applicant indicated that the reactor vessel equivalent margin analysis (EMA) option would be used to demonstrate an acceptable USE for the Linde 80 welds. The applicant stated that the EMA for weld WF-182-1 results were previously provided in LRA Section 4.2.2.3. The applicant also stated that the EMA for weld WF-182-1 bounds the shell welds. For the remaining Linde 80 welds, reactor vessel inlet/outlet nozzle welds, the applicant indicated that an EMA had been performed for the outlet nozzle welds and that this analysis is bounding for the nozzle welds. As part of the RAI response, the applicant agreed to submit the outlet nozzle weld EMA. Also, the applicant agreed to provide a technical justification as to why the EMA for weld WF-182-1 bounds the shell welds and the EMA for the outlet nozzle welds bounds the nozzle welds. These technical justifications are to include addressing geometry differences.

**RAI 4.2.4-1 Pressure-Temperature Limits**

**Discussion:**

For RAI 4.2.4-1, the staff provided clarification that this RAI pertains not only to reactor vessel beltline materials but also to all the other ferritic reactor coolant pressure boundary (RCPB) materials. The applicant stated that the methodology used to develop the pressure temperature (P-T) limit curves, considered all ferritic RCPB materials and is documented in topical report BAW-10046A, "Methods of Compliance with Fracture Toughness and Operational Requirements for 10 CFR 50 Appendix G," Revision 2. This topical report was approved by the NRC in a Safety Evaluation Report dated April 30, 1986.

The staff stated that the information provided by the applicant was understood and that the telephone conference call was productive.

There was no further discussion and the call was concluded.

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**/RA/**

Brian K. Harris, Project Manager  
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Office of Nuclear Reactor Regulation

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ADAMS Accession No.: ML12157a238

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