UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 29, 2012

Mr. Rafael Flores
Senior Vice President and
Chief Nuclear Officer
Attention: Regulatory Affairs
Luminant Generation Company LLC
P.O. Box 1002
Glen Rose, TX 76043

SUBJECT: COMANCHE PEAK NUCLEAR POWER PLANT, UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: ADOPTION OF TSTF-491, REVISION 2, "REMOVAL OF THE MAIN STEAM AND MAIN FEEDWATER VALVE ISOLATION TIME FROM TECHNICAL SPECIFICATIONS USING CONSOLIDATED LINE ITEM IMPROVEMENT PROCESS" (TAC NOS. ME7703 AND ME7704)

Dear Mr. Flores:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 157 to Facility Operating License No. NPF-87 and Amendment No. 157 to Facility Operating License No. NPF-89 for Comanche Peak Nuclear Power Plant, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated December 13, 2011.

The amendments modify TS 3.7.2, "Main Steam Isolation Valves (MSIVs)," and TS 3.7.3, "Feedwater Isolation Valves (FIVs) and Feedwater Control Valves (FCVs) and Associated Bypass Valves," in accordance with previously approved Technical Specification Task Force (TSTF) Change Traveler TSTF-491, Revision 2, by relocating the closure times for MSIVs, FIVs, FCVs, and associated bypass valves to the Technical Requirements Manual as part of the Consolidated Line Item Improvement Process (CLIIP).

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

Balwant K. Singal, Senior Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosures:
1. Amendment No. 157 to NPF-87
2. Amendment No. 157 to NPF-89
3. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

LUMINANT GENERATION COMPANY LLC

COMANCHE PEAK NUCLEAR POWER PLANT, UNIT NO. 1

DOCKET NO. 50-445

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 157
License No. NPF-87

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Luminant Generation Company LLC dated December 13, 2011, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-87 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A as revised through Amendment No. 157 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. Luminant Generation Company LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan as indicated in the attachment to this license amendment.

3. The license amendment is effective as of its date of issuance and shall be implemented within 120 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael T. Markley, Chief
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Facility Operating
License No. NPF-87 and
Technical Specifications

Date of Issuance: June 29, 2012

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

LUMINANT GENERATION COMPANY LLC

COMANCHE PEAK NUCLEAR POWER PLANT, UNIT NO. 2

DOCKET NO. 50-446

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 157
License No. NPF-89

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Luminant Generation Company LLC dated December 13, 2011, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-89 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A as revised through Amendment No. 157 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. Luminant Generation Company LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 120 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael T. Markley, Chief
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Facility Operating
License No. NPF-89 and
Technical Specifications

Date of Issuance: June 29, 2012

ATTACHMENT TO LICENSE AMENDMENT NO. 157

TO FACILITY OPERATING LICENSE NO. NPF-87

AND AMENDMENT NO. 157

TO FACILITY OPERATING LICENSE NO. NPF-89

DOCKET NOS. 50-445 AND 50-446

Replace the following pages of the Facility Operating License Nos. NPF-87 and NPF-89, and Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Operating License No. NPF-87

REMOVE	INSERT
3	3

Facility Operating License No. NPF-89

REMOVE	INSERT
3	3

Technical Specifications

REMOVE	INSERT
3.7-7	3.7-7
3.7-9	3.7-9

(3) Luminant Generation Company LLC, pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time, special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, and described in the Final Safety Analysis Report, as supplemented and amended;

(4) Luminant Generation Company LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use, at any time, any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(5) Luminant Generation Company LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required, any byproduct, source, and special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(6) Luminant Generation Company LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Luminant Generation Company LLC is authorized to operate the facility at reactor core power levels not in excess of 3458 megawatts thermal through Cycle 13 and 3612 megawatts thermal starting with Cycle 14 in accordance with the conditions specified herein.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No. 157 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. Luminant Generation Company LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Luminant Generation Company LLC, pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time, special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, and described in the Final Safety Analysis Report, as supplemented and amended;

(4) Luminant Generation Company LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use, at any time, any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(5) Luminant Generation Company LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required, any byproduct, source, and special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(6) Luminant Generation Company LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Luminant Generation Company LLC is authorized to operate the facility at reactor core power levels not in excess of 3458 megawatts thermal through Cycle 11 and 3612 megawatts thermal starting with Cycle 12 in accordance with the conditions specified herein.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No. 157 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. Luminant Generation Company LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Antitrust Conditions

DELETED

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.7.2.1	----------NOTE---------- Only required to be performed in MODES 1 and 2. ---------- Verify the isolation time of each MSIV is within limits.	In accordance with the Inservice Testing Program
SR 3.7.2.2	----------NOTE---------- Only required to be performed in MODES 1 and 2. ---------- Verify each MSIV actuates to the isolation position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program.

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. One or more FIV or FCV bypass valves inoperable.	C.1 Close or isolate bypass valve. AND C.2 Verify bypass valve is closed or isolated.	72 hours Once per 7 days
D. Two valves in the same flowpath inoperable	D.1 Isolate affected flow path.	8 hours
E. Required Action and associated Completion Time not met.	E.1 Be in MODE 3. AND E.2 Be in MODE 4.	6 hours 12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.7.3.1	Verify the isolation time of each FIV, FCV, and associated bypass valves is within limits.	In accordance with the Inservice Testing Program
SR 3.7.3.2	Verify each FIV, FCV, and associated bypass valves actuates to the isolation position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 157 TO

FACILITY OPERATING LICENSE NO. NPF-87

AND AMENDMENT NO. 157 TO

FACILITY OPERATING LICENSE NO. NPF-89

LUMINANT GENERATION COMPANY LLC

COMANCHE PEAK NUCLEAR POWER PLANT, UNITS 1 AND 2

DOCKET NOS. 50-445 AND 50-446

1.0 INTRODUCTION

By application dated December 13, 2011 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML11356A219), Luminant Generation Company LLC (the licensee) requested changes to the Technical Specifications (TSs) for Comanche Peak Nuclear Power Plant, Units 1 and 2 (CPNPP). The proposed changes would modify TS 3.7.2, "Main Steam Isolation Valves (MSIVs)," and TS 3.7.3, "Feedwater Isolation Valves (FIVs) and Feedwater Control Valves (FCVs) and Associated Bypass Valves," in accordance with previously approved Technical Specification Task Force (TSTF) Change Traveler TSTF-491, Revision 2, by relocating the closure times for MSIVs, FIVs, FCVs, and associated bypass valves to the Technical Requirements Manual (TRM). The availability of TSTF-491, Revision 2, was announced in the *Federal Register* on December 29, 2006 (71 FR 78472), as part of the Consolidated Line Item Improvement Process (CLIIP) of the U.S. Nuclear Regulatory Commission (NRC).

2.0 REGULATORY EVALUATION

Section 182a of the Atomic Energy Act (the "Act") requires applicants for nuclear power plant operating licenses to include TSs as part of the license. The TSs ensure the operational capability of structures, systems, and components that are required to protect the health and safety of the public. The Commission's regulatory requirements related to the content of the TSs are contained in Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.36. The regulations in 10 CFR 50.36(c) require that TSs include items in the following specific categories: (1) safety limits, limiting safety systems settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements (SRs); (4) design features; and (5) administrative controls.

In general, there are two classes of changes to TSs: (1) changes needed to reflect modifications to the design basis (TSs are derived from the design basis), and (2) voluntary changes to take advantage of the evolution in policy and guidance as to the required content and preferred format of TSs over time. This amendment deals with the second class of changes.

In determining the acceptability of revising CPNPP TSs, the NRC staff used the accumulation of generically approved guidance in NUREG-1431, Vol. 1, Revision 3, "Standard Technical Specifications, Westinghouse Plants," dated June 2004 (ADAMS Accession No. ML041830612). Licensees may revise the TSs to adopt current improved Standard Technical Specification (STS) format and content provided that the plant-specific review supports a finding of continued adequate safety because: (1) the change is editorial, administrative, or provides clarification (i.e., no requirements are materially altered), (2) the change is more restrictive than the licensee's current requirement, or (3) the change is less restrictive than the licensee's current requirement, but nonetheless still affords adequate assurance of safety when judged against current regulatory standards. The detailed application of this general framework, and additional specialized guidance, are discussed in Section 3.0 in the context of specific proposed changes.

3.0 TECHNICAL EVALUATION

3.1 Proposed TS Changes

In its application dated December 13, 2011, the licensee requested the following changes to TS 3.7.2, "Main Steam Isolation Valves (MSIVs)," and TS 3.7.3, "Feedwater Isolation Valves (FIVs) and Feedwater Control Valves (FCVs) and Associated Bypass Valves."

Current SR 3.7.2.1 states:

> Verify the isolation time of each MSIV is $\leq$ 5 seconds.

Revised SR 3.7.2.1 would state:

> Verify the isolation time of each MSIV is within limits.

Current SR 3.7.3.1 states:

> Verify the isolation time of each FIV, FCV, and associated bypass values is $\leq$ 5 seconds.

Revised SR 3.7.3.1 would state:

> Verify the isolation time of each FIV, FCV, and associated bypass values is within limits.

3.2 NRC Staff Evaluation

The NRC staff reviewed the justification for TSTF-491 as described in the industry submittal dated September 13, 2005 (ADAMS Accession No. ML052590343). The licensee stated in its application that the information presented in the TSTF-491 proposal and the model safety

evaluation, prepared by the NRC staff, are applicable to CPNPP for the incorporation of the changes to CPNPP's TSs. The detailed evaluation below supports the conclusion that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

3.2.1 Main Steam Isolation Valves (MSIVs)

The MSIVs isolate steam flow from the secondary side of the steam generators (SGs) following a high energy line break (HELB). One MSIV is located in each main steam line outside of the containment. Closing the MSIVs isolates each SG from the others and isolates the turbine, steam dump system, and other auxiliary steam supplies from the SG. By isolating the steam flow from the secondary side of the SG, the MSIVs prevent overcooling the reactor core following a HELB. By preventing core overcooling, the MSIVs protect the reactor core from being damaged.

TSTF-491 relocated the required closure times to a licensee-controlled document (LCD) that is referenced in the STS Bases. The NRC staff approved TSTF-491 on the basis that changes to LCDs are subject to the 10 CFR 50.59 process. This provides adequate assurance that prior staff review and approval will be requested by a licensee for changes to the LCD with the potential to affect the safe operation of the plant. The licensee proposed relocating the required closure times for the MSIVs to the CPNPP TRM, an LCD. CPNPP has a TS Bases Control Program that requires that proposed changes to the Bases are subject to the 10 CFR 50.59 process. The staff reviewed the CPNPP TS Bases Control Program and determined that the CPNPP TS Bases Control Program provides adequate assurance that prior staff review and approval will be requested by the licensee for changes to the TS Bases with the potential to affect the safe operation of the plant. Furthermore, the MSIVs are subject to periodic testing and acceptance criteria in accordance with the Inservice Testing (IST) Program. Compliance with the IST Program is required by TS Section 5.5.8 and 10 CFR 50.55a. The IST Program includes specific reference value baseline operating times for valves that are not subject to arbitrary changes.

The regulations in 10 CFR 50.36 require the inclusion of the periodic testing of the MSIVs in the SRs, not the actual closure time of the valves. The licensee's adoption of TSTF-491 changes maintains the periodic testing requirements for MSIVs in accordance with 10 CFR 50.36.

Based on the requirements of 10 CFR 50.36, 10 CFR 50.59, and the IST Program, the NRC staff concludes that relocating the MSIV closure time to the CPNPP TRM is acceptable.

3.2.2 Feedwater Isolation Valve (FIVs), Feedwater Control Valves (FCVs) and Associated Bypass Valves

In the event of a secondary side pipe rupture inside containment, the FIVs, FCVs, and associated bypass valves limit the quantity of high-energy fluid that enters the containment through the break and provide a pressure boundary for the controlled addition of auxiliary feedwater to the intact loops. By isolating the feedwater flow from the affected SG, the FIVs,

FCVs, and associated bypass valves prevent over cooling the reactor core and over pressurizing of the containment.

TSTF-491 relocated the required closure times to an LCD that is referenced in the STS Bases. The NRC staff approved TSTF-491 on the basis that changes to LCDs are subject to the 10 CFR 50.59 process. This provides adequate assurance that prior NRC staff review and approval will be requested by a licensee for changes to the LCD with the potential to affect the safe operation of the plant. The licensee proposed relocating the required closure times for the valves to the CPNPP TRM. CPNPP has a TS Bases Control Program that requires that proposed changes to the Bases are subject to the 10 CFR 50.59 process. The staff reviewed the CPNPP TS Bases Control Program and determined that the CPNPP TS Bases Control Program provides adequate assurance that prior staff review and approval will be requested by the licensee for changes to the TS Bases with the potential to affect the safe operation of the plant. Furthermore, the valves are subject to periodic testing and acceptance criteria in accordance with the IST Program. Compliance with the IST Program is required by TS Section 5.5.8 and 10 CFR 50.55a. The IST Program includes specific reference value baseline operating times for valves that are not subject to arbitrary changes.

The regulations in 10 CFR 50.36 require the inclusion of the periodic testing of the valves in the SRs, not the actual closure time of the valves. The licensee's adoption of TSTF-491 changes maintains the periodic testing requirements for the valves in accordance with 10 CFR 50.36.

Based on the requirements of 10 CFR 50.36, 10 CFR 50.59, and the IST Program, the NRC staff concludes that relocating the closure times for FIVs, FCVs, and associated bypass valves to the TRM as referenced in the Bases is acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Texas State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding published in the *Federal Register* on January 24, 2012 (77 FR 3511). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Matthew E. Hamm

Date: June 29, 2012

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA/

Balwant K. Singal, Senior Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosures:
1. Amendment No. 157 to NPF-87
2. Amendment No. 157 to NPF-89
3. Safety Evaluation

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ADAMS Accession No. ML12156A068

OFFICE	NRR/LPL4/PM	NRR/LPL4/LA	NRR/DSS/STSB/BC	OGC NLO	NRR/LPL4/BC	NRR/LPL4/PM
NAME	BSingal	JBurkhardt	REIliott (MHamm For)	DRoth	MMarkley	BSingal (MMarkley for)
DATE	6/14/12	6/12/12	6/15/12	6/19/12	6/29/12	6/29/12

OFFICIAL AGENCY RECORD