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TOKYO, JAPAN

May 31, 2012

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001

Attention: Mr. Jeffrey A. Ciocco

Docket No. 52-021
MHI Ref: UAP-HF-12136

Subject: **MHI's Amended Response to US-APWR DCD RAI No. 373-2826 Revision 1 (SRP 14.03.04)**

Reference: 1) "Request for Additional Information No. 373-2826 Revision 1, SRP Section 14.03.04 – Reactor Systems - Inspections, Tests, Analyses, and Acceptance Criteria - Application Section: 14.3.4", dated October 18, 2011 (ML091410241).
2) "MHI's Response to US-APWR DCD RAI No.373-2826 Revision 0", UAP-HF-09320, dated June 16, 2009 (ML091690071)

With this letter, Mitsubishi Heavy Industries, Ltd. ("MHI") transmits to the U.S. Nuclear Regulatory Commission ("NRC") a document entitled "Amended Response to Request for Additional Information No. 373-2826 Revision 1 " This amended response is submitted to update the response to reflect recent design changes related to the closure of GSI-191.

Enclosed is the amended response to Question 14.03.04-39 contained within Reference 1. This response supersedes the previous response to Question 14.03.04-39 transmitted by Reference 2 in its entirety. The other responses in Reference 2 are not changed by the enclosure to this letter,

Please contact Mr. Joseph Tapia, General Manager of Licensing Department, Mitsubishi Nuclear Energy Systems, Inc. if the NRC has questions concerning any aspect of this submittal. His contact information is provided below.

Sincerely,

Y. Ogata

Yoshiki Ogata,
Director- APWR Promoting Department
Mitsubishi Heavy Industries, LTD.

D081
NRO

Enclosure:

1. Amended Response to Request for Additional Information No. 373-2826 Revision 1

CC: J. A. Ciocco
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Enclosure 1

UAP-HF-12136
Docket No. 52-021

Amended Response to Request for Additional Information
No. 373-2826 Revision 1

May 2012

RESPONSE TO REQUEST FOR ADDITIONAL INFORMATION

05/31/2012

US-APWR Design Certification

Mitsubishi Heavy Industries

Docket No. 52-021

RAI NO.: **NO. 373-2826 REVISION 1**

SRP SECTION: **14.03.04 – REACTOR SYSTEMS - INSPECTIONS, TESTS, ANALYSES, AND ACCEPTANCE CRITERIA**

APPLICATION SECTION: **14.3.4**

DATE OF RAI ISSUE: **5/21/2009**

QUESTION NO.: 14.03.04-39

In Table 2.4.5-5, 8.f. Acceptance Criteria, is the NPSH of 17.9 ft at 3650 gpm determined by the RHR or Containment Spray system need? If RHR, what is the Containment Spray NPSH and flow rate required values? When aligned for Containment Spray what NPSH loss is assumed for degraded sump strainer performance?

ANSWER:

The NPSH is determined by the Containment Spray System need. The maximum NPSH loss is considered for degraded sump strainer performance. Details of the sump strainer performance and NPSH evaluation are described in technical report MUAP-08001.

Impact on DCD

There is no impact on the DCD.

Impact on R-COLA

There are no impacts on the R-COLA.

Impact on S-COLA

There are no impacts on the S-COLA.

Impact on PRA

There is no impact on the PRA.

Impact on Technical / Topical Reports

There is no impact on the Technical / Topical Reports.