

VoglecolRAIsPEm Resource

From: Joshi, Ravindra
Sent: Wednesday, May 30, 2012 9:29 AM
To: VoglecolRAIsPEm Resource
Subject: REQUEST FOR ADDITIONAL INFORMATION LETTER NO. 01 RELATED TO LICENSE AMENDMENT REQUEST (LAR) 12-003 FOR THE VOGTLE ELECTRIC GENERATING PLANT UNITS 3 AND 4 COMBINED LICENSES
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Subject: REQUEST FOR ADDITIONAL INFORMATION LETTER NO. 01 RELATED TO
LICENSE AMENDMENT REQUEST (LAR) 12-003 FOR THE VOGTLE ELECTRIC GENERATING
PLANT UNITS 3 AND 4 COMBINED LICENSES

Sent Date: 5/30/2012 9:29:08 AM

Received Date: 5/30/2012 9:29:11 AM

From: Joshi, Ravindra

Created By: Ravindra.Joshi@nrc.gov

Recipients:

"VogtlecolRAIsPEm Resource" <VogtlecolRAIsPEm.Resource@nrc.gov>

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May 30, 2012

Mr. B. L. Ivey
Vice President, Regulatory Affairs
Southern Nuclear Operating Company
P.O. Box 1295
Bin B022
Birmingham, AL 35201

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION LETTER NO. 01 RELATED TO
LICENSE AMENDMENT REQUEST (LAR) 12-003 FOR THE VOGTLE
ELECTRIC GENERATING PLANT UNITS 3 AND 4 COMBINED LICENSES

Dear Mr. Ivey:

In accordance with the provisions of 10 CFR 50.90, by letter dated April 6, 2012 and revised by letters dated April 12, 2012 and May 7, 2012, Southern Nuclear Operating Company (SNC), submitted a license amendment request (LAR) 12-003 to the U. S. Nuclear Regulatory Commission (NRC) for its Vogtle Electric Generating Plant (VEGP) Units 3 and 4 Combine licenses (Licenses Nos.NPF-91 and NPF-92, respectively). The NRC staff is performing a detailed review of this LAR to enable the staff to reach a conclusion on the safety of the proposed LAR.

The NRC staff has identified that additional information is needed to continue portions of the review. The staff's request for additional information (RAI) is contained in the enclosure to this letter.

To support the review schedule, you are requested to respond within 30 days of the date of this letter. If changes are needed to the final safety analysis report, the staff requests that the RAI response include the proposed wording changes.

If you have any questions or comments concerning this matter, you may contact me at 301-415-6191 or ravindra.joshi@nrc.gov.

Sincerely,

/RA/

Ravindra G. Joshi, Senior Project Manager
Licensing Branch 4
Division of New Reactor Licensing
Office of New Reactors

Docket Nos. 52-025
52-026
eRAI Tracking No. 6534

Enclosure:
Request for Additional Information

CC: see next page

If you have any questions or comments concerning this matter, you may contact me at 301-415-6191 or ravindra.joshi@nrc.gov.

Sincerely,

/RA/

Ravindra G. Joshi, Senior Project Manager
Licensing Branch 4
Division of New Reactor Licensing
Office of New Reactors

Docket Nos. 52-025
52-026
eRAI Tracking No. 6534

Enclosure:
Request for Additional Information

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OFFICE	DNRL/LB4:BC	LB4-PM	OGC	LB4-PM
NAME	MTonacci for R. Joshi*	RJoshi*	N/A*	RJoshi *
DATE	5/24/12	5/24/12	5/25/12	5/25/12

*Approval captured electronically in the electronic RAI system.

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VEGP 3 and 4 LAR 12-003 Basemat Thickness
Southern Nuclear Operating Co.
Docket No. 52-025 and 52-026
SRP Section: 01 - Introduction and Interfaces
Application Section: Technical Evaluation

QUESTIONS from Licensing Branch 4 (LB4)

01-1

a. SNC states (in Technical Evaluation Section) in part that the change of the tolerance for the basemat thickness identified in the DCD to +4 inches was chosen as a standard value to be sufficient for Vogtle Units 3 and 4, and future AP1000 applications.

This statement implies that SNC is requesting review of this tolerance for Vogtle Units 3 and 4 as well as for future AP1000 application. It is our understanding is that the subject LAR is for Vogtle Units 3 and 4 and not for other future applications. The scope of NRC review for this LAR is limited to Vogtle Units 3 and 4. Please revise the submittal to delete the reference to future AP1000 applications.

b. SNC states (in Technical Evaluation) in part that the tolerance of +1 inch in the DCD Revision 19 was determined to be a misapplication of the standard tolerance for cast in place floors during the review of the as-built mudmat elevation.

Please explain the term 'misapplication?' Is this misinterpretation for Vogtle site using ACI 117 specifications for cast in place floor provision? Please clarify and make appropriate changes to the submittal accordingly.

c. Technical Evaluation Section of the LAR (page 5 of 9 of Enclosure 1) refers to FSAR Subsection 2.5.4.10.2. The staff could not find the referenced section in the currently docketed Vogtle FSAR and therefore could not verify the validity of the statement, "Differential settlement of a few inches across the width of the nuclear island would not have an adverse effect on the safety-related functions of structures, systems, and components." Please provide the correct reference to the FSAR section and correct your submittal.