

**CERTIFICATE OF COMPLIANCE  
FOR RADIOACTIVE MATERIAL PACKAGES**

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2. PREAMBLE

- a. This certificate is issued to certify that the package (packaging and contents) described in Item 5 below meets the applicable safety standards set forth in Title 10, Code of Federal Regulations, Part 71, "Packaging and Transportation of Radioactive Material."  
This certificate does not relieve the consignor from compliance with any requirement of the regulations of the U.S. Department of Transportation or
- b. other applicable regulatory agencies, including the government of any country through or into which the package will be transported.

3. THIS CERTIFICATE IS ISSUED ON THE BASIS OF A SAFETY ANALYSIS REPORT OF THE PACKAGE DESIGN OR APPLICATION

- a. ISSUED TO (*Name and Address*)  
EnergySolutions  
Suite 100, Center Point II  
100 Center Point Circle  
Columbia, SC 29210
- b. TITLE AND IDENTIFICATION OF REPORT OR APPLICATION  
EnergySolutions application dated January 24, 2011,  
as supplemented.

4. CONDITIONS

This certificate is conditional upon fulfilling the requirements of 10 CFR Part 71, as applicable, and the conditions specified below.

5.

(a) Packaging

(1) Model No.: 10-160B

(2) Description

A cylindrical carbon steel and lead shielded shipping cask, designed to transport radioactive waste material. The cask is transported in the upright position and is equipped with steel encased, rigid polyurethane foam impact limiters on the top and bottom. The package has approximate dimensions, shielding, and weight as follows:

Cask height	88 inches
Cask outer diameter	78-1/2 inches
Cask cavity height	77 inches
Cask cavity diameter	68 inches
Overall package height, with impact limiters	130 inches
Overall package diameter, with impact limiters	102 inches
Lead shielding thickness	1-7/8 inches
Gross weight	
(packaging and contents)	72,000 lbs
Maximum total weight of contents, shoring, secondary containers, and optional shield insert	14,500 lbs

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5.(a)(2) Description (Continued)

The cask body consists of a 1-1/8-inch thick carbon steel (ASME SA516 or SA537) inner shell, a 1-7/8-inch thick lead gamma shield, and a 2-inch thick carbon steel outer shell (ASME SA516). The inner and outer shells are welded to a 5-1/2-inch thick carbon steel bottom plate. The cask cavity has an optional 11-gage stainless steel liner. A 12-gage stainless steel thermal shield surrounds the cask outer shell in the region between the impact limiters. The impact limiters are secured to each other around the cask by eight ratchet binders.

The cask lid is a 5-1/2-inch thick carbon steel plate, and has a 31-inch diameter opening equipped with a secondary lid. The primary lid is sealed with a double elastomer O-ring and 24 equally spaced 1-3/4-inch diameter bolts. The secondary lid is 46 inches in diameter, is centered within the primary lid, and is sealed to the primary lid by a double elastomer O-ring and 12 equally spaced 1-3/4-inch diameter bolts. The space between the double O-ring seals is provided with a test port for leak testing the primary and secondary lid seals.

The optional cask drain and vent ports are sealed with a plug and an O-ring seal.

The package is equipped with four tie-down lugs welded to the cask outer shell. Two lifting lugs and two redundant lifting lugs are removed during transport. The lid is equipped with three lifting lugs which are covered by the top impact limiter and rain cover during transport.

An optional carbon steel shield insert may be used within the cask cavity for contents as specified in Condition No. 5(b)(1)(i) through (v). For contents specified in Condition No. 5(b)(1)(vi), a source insert shall be used. The source insert design weight is 8,000 lbs; it has side walls consisting of 6.0-inch thick lead, sandwiched between an inner 8 inches nominal SCH 60 steel pipe and an outer 24.0-inch SCH 60 steel pipe. The bottom of the source insert also consists of lead supported by a 0.75-inch thick steel base plate. The lid includes a steel encased lead plug, steel bolting plate and flat silicone rubber gasket.

(3) Drawings

The packaging is constructed and assembled in accordance with EnergySolutions Drawing No. C-110-D-29003-010, sheets 1 through 5, Rev. 14.

An optional shield insert is constructed in accordance with Chem-Nuclear Systems Drawing No. C-119-B-0018, Rev. 2.

The Source Insert Assembly is constructed in accordance with EnergySolutions Drawing No. C-038-145083-004, Rev.0.

The Source Insert Steel Cribbing is constructed in accordance with EnergySolutions Drawing No. C-038-145083-005, Rev. 0.

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5.(b) Contents

(1) Type and form of material

- (i) Byproduct, source, and special nuclear material, non-fissile or fissile-excepted, as special form, or non-special form in the form of process solids or resins, either dewatered, solid, or solidified waste, in secondary containers; or
- (ii) Dewatered, solid or solidified transuranic-containing wastes (TRU), fissile or non-fissile or fissile-excepted, in secondary containers; or
- (iii) Plutonium 239 (Pu-239) as Pu-Be neutron sources meeting the requirements of special form sources; or
- (iv) Neutron activated metals or metal oxides in solid form in secondary containers; or
- (v) Miscellaneous radioactive solid waste materials, including special form materials and powdered solids, in secondary containers.
- (vi) Byproduct material as Co-60 loaded into the source insert.

(2) Maximum quantity of material per package

- (i) The maximum quantity of radioactive materials must be the lesser of the quantity determined by the methodology described in Attachment 1 to Chapter No. 7 of the application or 3000 A<sub>2</sub>, except for contents specified in Condition No. 5(b)(1)(vi) for which the limit is 10,000 Ci of Co-60.
- (ii) Fissile contents must be limited to the fissile gram equivalent of 325 grams of Pu-239, as determined using the conversion factors in Table 9.1.3, in Chapter No. 4, Appendix 4.10.2, of the application. Plutonium content exceeding 0.74 TBq (20 Ci) must be in solid form.
- (iii) TRU exceeding the fissile limits of 10 CFR 71.15 must not be machine-compacted and must have no more than 1% by weight of special reflectors and no more than 25% by volume of hydrogenous material.
- (iv) Neutron sources as described in 5(b)(1)(iii) are limited to a maximum emission rate of 1.1E+8 n/sec.
- (v) Maximum decay heat: 200 watts.
- (vi) Maximum weight of contents: 14,500 pounds including shoring, secondary containers, and either optional shield insert or mandatory source insert. The contents of the source insert have a maximum weight of 500 pounds.

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- (vii) Explosives, corrosives, non-radioactive pyrophorics, and compressed gases are prohibited. Pyrophoric radionuclides may be present only in residual amounts less than 1 weight percent.
- (viii) The total amount of potentially volatile organic compounds present in the headspace of a secondary container is restricted to 500 parts per million.
- (ix) Powdered solid radioactive materials shall not include radioactive forms of combustible metal hydrides or combustible elemental metals, i.e., magnesium, titanium, sodium, potassium, lithium, zirconium, hafnium, calcium, zinc, plutonium, uranium, and thorium, or combustible non-metals, i.e., phosphorus.
- (x) Powdered solids contents with neutron emitters are not permitted.

5.(c) Criticality Safety Index 0.0

6. In addition to the requirements of Subpart G of 10 CFR Part 71:

- (a) The package must be prepared for shipment and operated in accordance with the Operating Procedures of Chapter No. 7 of the application, except Section No. 7.4;
- (b) Each packaging must meet the Acceptance Tests and Maintenance Program of Chapter No. 8 of the application;
- (c) The maximum speed of the truck during a Type B shipment shall be limited to 55 mph;
- (d) Each Type B shipment must be accompanied by a clearly recognizable escort vehicle that provides enhanced emergency response notification, maintains continuous visual and radio communication with the tractor-trailer performing the Type B shipment and separates the shipment from other hazardous material shipments; and
- (e) All Type B shipments shall be treated as if the package contains RAM QC quantities of radioactive material, regardless of quantity of radioactive material in the package, to provide:
  - 1. Increased inspections of equipment to prevent mechanical issues,
  - 2. Increased communications requirements,
  - 3. Mandatory installation of Qualcomm GPS Hardware Equipment,
  - 4. Automatic Ping/Email to Logistics Staff after Accident Conditions, and
  - 5. Route selection planning and enhanced driver training.

7. Transport by air of fissile material is not authorized.

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8. Flammable gas (hydrogen) concentration is limited to less than 5% in volume. For contents other than TRU waste, inerting is not allowed to limit the concentration of flammable gases. For TRU waste, compliance with the 5% hydrogen concentration limit is determined by the methods discussed in Appendix 4.10.2 of the application. For contents with a radioactivity concentration not exceeding that for Low Specific Activity material, the hydrogen concentration can be assumed to be less than 5% provided the package is shipped within 10 days of preparation, or within 10 days after venting of drums or other secondary containers.
9. Payload containers authorized for shipment of TRU waste are the 30-gallon and the 55-gallon drums. TRU waste characteristics are determined and limited in accordance with Appendix 4.10.2 of the application.
10. The non-homogeneity of the package contents may lead to elevated levels of radiation on the package surfaces. Radiation surveys must be performed to obtain measurements from all surfaces of the package, and from the outer surfaces of the vehicle enclosure, unless process knowledge or survey history indicates that elevated radiation levels are not likely to be encountered.
11. Appropriate devices or measures must secure contents in the secondary container, if necessary.
12. The package authorized by this certificate is hereby approved for use under the general license provisions of 10 CFR 71.17, for 90 days from the date of issuance of this certificate.
13. Revision No. 17 of this certificate may be used until December 31, 2012, provided the compensatory measures in Condition 6(c) through (e) of Revision No. 19 of this certificate are followed. The authorization to use Revision No. 17 of this certificate expires 90 days from the date of issuance of Revision No. 18 of this certificate.
15. Expiration date: October 31, 2015.

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REFERENCES

EnergySolutions application dated January 24, 2011.

Supplements dated April 6, September 9, and October 28, 2011; and May 15, and 17, 2012.

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

**/RA/**

Michael D. Waters, Chief  
Licensing Branch  
Division of Spent Fuel Storage and Transportation  
Office of Nuclear Material Safety  
and Safeguards

Date: May 25, 2012

