



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**

REGION III
2443 WARRENVILLE ROAD, SUITE 210
LISLE, IL 60532-4352

May 8, 2012

Mr. Michael J. Pacilio
Senior Vice President, Exelon Generation Company, LLC
President and Chief Nuclear Officer (CNO), Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: QUAD CITIES NUCLEAR POWER STATION, UNIT 2
NRC POST APPROVAL SITE INSPECTION FOR LICENSE RENEWAL
INSPECTION REPORT 05000265/2012008

Dear Mr. Pacilio:

On March 28, 2012, the U.S. Nuclear Regulatory Commission (NRC) completed Phase I of the Post-Approval Site Inspection for License Renewal at your Quad Cities Nuclear Power Station, Unit 2. The enclosed report documents the inspection activities, which were discussed on March 28, 2012, with Mr. T. Hanley and other members of your staff.

This inspection was an examination of activities conducted under your renewed license as they relate to the completion of commitments made during the renewed license application process and compliance with the Commission's rules and regulations and the conditions of your operating license. Within these areas, the inspection involved examination of selected procedures and representative records, observations of activities, and interviews with personnel. On the basis of the sample selected for review, there were no findings of significance identified during this inspection. The NRC staff did not identify any instances of incomplete commitments with respect to timeliness or adequacy.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS), accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

Ann Marie Stone, Chief
Engineering Branch 2
Division of Reactor Safety

Docket No. 50-265
License No. DPR-30

Enclosure: Inspection Report 05000265/2012008
w/Attachment: Supplemental Information

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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket No: 50-265

License No: DPR-30

Report No: 05000265/2012008

Licensee: Exelon Generation Company, LLC

Facility: Quad Cities Nuclear Power Station, Unit 2

Location: Cordova, IL

Dates: March 19–28, 2012

Inspectors: T. Bilik, Senior Reactor Engineer (Lead)
D. Jones, Reactor Engineer

Approved by: Ann Marie Stone, Chief
Engineering Branch 2
Division of Reactor Safety

Enclosure

SUMMARY OF FINDINGS

IR 05000265/2012008; 03/19/2012–03/28/2012; Quad Cities Nuclear Power Station, Unit 2;
Post-Approval Site Inspection for License Renewal

The inspection was conducted by two regional based inspectors. No instances were noted of incomplete license renewal commitments with respect to timeliness or adequacy. The NRC's program for overseeing the safe operation of commercial nuclear power reactors as described in NUREG-1649, "Reactor Oversight Process," Revision 4, dated December 2006.

A. NRC-Identified and Self-Revealed Findings

Cornerstone: Initiating Events, Mitigating Systems, Barrier Integrity

No findings were identified.

B. Licensee-Identified Violations

No violations of significance were identified.

REPORT DETAILS

Summary of Plant Status

Quad Cities Nuclear Power Station, Unit 2 was in a refueling outage during the period of this inspection.

4. OTHER ACTIVITIES

4OA5 Other Activities

.1 Post-Approval Site Inspection for License Renewal (Phase I) - IP 71003

a. Inspection Scope

(1) Review of Newly Identified Structures Systems and Components (SSC)

The inspectors discussed the identification of new SSCs, under the purview of Title 10 of the Code of Federal Regulations (CFR) 54.37(b), with the licensee's license renewal staff. The licensee personnel indicated that newly identified components had resulted from a change to the Environmental Qualification (EQ) Program. The components were subsequently added to the site EQ program which is also credited as the License Renewal (LR) Aging Management Program (AMP) B1.35. The AMP will manage the effects of aging for those newly identified components.

The License Renewal Updated Final Safety Analysis Report Supplement, Appendix A.1.35, "Environmental Qualification (EQ) of Electrical Components," remains an accurate summary of the AMP, and does not require revision as a result of the newly identified components.

(2) Review of Revised Commitments

As part of reviewing the AMPs associated with the commitments, the inspectors reviewed a sample of commitment revisions related to license renewal. It was identified that of the six commitment revisions provided, five had already been provided during the Phase I inspection of Quad Cities Units 1. As a result, the inspectors reviewed the most recent commitment revision provided, a revision to Commitment No. 44 (AMP B.2.6), as documented in Appendix A of NUREG 1796, NRC Safety Evaluation of the Dresden Nuclear Power Station, Units 2 and 3 and Quad Cities Nuclear Power Station, Units 1 and 2. In Commitment No. 44, Exelon Generation Company, LLC (EGC) made the following commitment:

"An aging management program will be developed and implemented for heat exchangers in the scope of license renewal that are not tested and inspected by the Open-Cycle Cooling water System and Closed-Cycle Cooling Water System aging management programs".

Further, in NUREG-1796, the original commitment states that "Eddy current testing is to be performed at least once every ten years, and the procedure provides for increasing the inspection frequency based on the results."

The licensee revised the commitment description to read, "Eddy current testing is to be performed at least once every ten years, or no greater than once every fifth refuel outage if on an outage schedule. The eddy current testing procedure provides for increasing the inspection frequency based on the results."

The inspectors noted that the verbiage in the commitment in its current form, the modifier of "no greater than" could be tied to the frequency "once" which would indicate that the licensee could not test more than once during the five refueling outage period. The licensee initiated action request AR 01376172 to address the issue. The licensee plans to clarify the language in the commitment revision to reflect the intent of the revision, that is, to specify that the test interval cannot exceed every fifth refueling outage.

After discussion with licensee staff on this commitment revision, the inspectors find the licensee's commitment revision acceptable once edited.

The inspectors also reviewed the licensee's commitment tracking program to evaluate its effectiveness.

(3) Review of Commitments

The inspectors reviewed supporting documents including completed surveillance records, conducted interviews, observed non-destructive examination (NDE) activities, performed visual inspection of structures and components, including those not accessible during power operation, and observed the activities described below to verify the licensee completed the necessary actions to comply with the license conditions that are a part of the renewed operating license. The inspectors verified the licensee implemented the "outage related" aging management programs included in NUREG 1796, in accordance with 10 CFR Part 54, "Requirements for the Renewal of Operating Licenses for Nuclear Power Plants." The inspectors also verified a selected sample of corrective actions taken as a result of the license renewal inspection.

b. Findings and Observations

Results of Detailed Reviews

The inspectors reviewed portions of the commitments below, which are referenced to Appendix A of NUREG 1796. Activities observed related to these commitments are also listed.

(1) (B.1.1) ASME Code, Section XI Inservice Inspection, Subsections IWB, IWC, and IWD (Commitment 1)

The ASME Section XI Inservice Inspection, Subsections IWB, IWC, and IWD aging management program (AMP) is part of the inservice inspection (ISI) program (an existing program) and provides for condition monitoring of reactor coolant pressure retaining piping and components within the scope of license renewal, except for the reactor pressure vessel. It also provides for condition monitoring of reactor internal components within the scope of license renewal. The program is implemented through procedures that require examinations consistent with ASME Section XI and approved relief request I4R-02.

A Risk-Informed Inservice Inspection (RISI) is implemented in lieu of ASME Section XI requirements for portions of Class 1 and Class 2 systems. The RISI is implemented as an alternative to the 1995 Edition through 1996 Addenda of ASME Section XI requirements for portions of Class 1 and Class 2 systems.

With enhancement, the ASME Section XI Inservice Inspection, Subsections IWB, IWC, and IWD, the AMP is consistent with the ten elements of Aging Management Program XI.M1, "ASME Section XI Inservice Inspection, Subsections IWB, IWC, and IWD," specified in NUREG-1801.

The inspectors conducted interviews and reviewed documentation. The inspectors also observed a number of NDE examinations as part of the RISI Program. These included ultrasonic testing (UT) of containment spray system, elbow-to-pipe welds, 1401-7, 1402-5, and 1402-7 (recorded in report numbers Q2R21-045, 46, and 47 respectively). The inspectors had no concerns with the observed activities.

(2) (B.1.11) Flow Accelerated Corrosion (FAC) (Commitment No. 11)

The Flow Accelerated Corrosion (FAC) AMP is an existing program which will be enhanced to include portions of the main steam and the reactor vessel head vent systems that are within the scope of license renewal. The program's activities predict, detect, and monitor wall thinning in piping and valves due to flow-accelerated corrosion. It also provides for prediction of the amount of wall thinning in pipes and fittings through analytical evaluations and periodic examinations of locations most susceptible to FAC induced loss of material. Specifically, the program includes analyses to determine critical locations, baseline inspections to determine the extent of thinning at these locations, and follow-up inspections to confirm the predictions. Repairs and replacements are performed as necessary.

The inspectors observed a surveillance credited for the FAC AMP. This surveillance, documented in report 2FD01DD, performed an ultrasonic testing thickness measurement of portions of the feedwater tee to detect any material loss as a result of FAC. The inspectors also reviewed UT thickness measurement records for report number 2FR13C. The inspectors had no concerns with the observed activities.

(3) (B.1.23) One-Time Inspection Program (Commitment No. 23 (H, I and J))

The One-Time Inspection AMP is a new program which provides one-time inspections to manage aging effects of identified components within the scope of license renewal. The purpose of the program is to determine if a specified aging effect is occurring. If the aging effect is occurring, the program provides for evaluation of the affected components' ability to perform their intended functions for the period of extended operation, and appropriate corrective action. The program is implemented through station procedures.

The inspectors reviewed the licensee's activities to implement Commitment Item No. 23 which documents the agreement to perform one-time inspection sampling of 11 specific inspection areas.

To verify the program's effectiveness the inspectors interviewed the program owner and reviewed completed WO 00725725, related to AMP B.1.23H, which credits the One-Time Inspection Program to perform a one-time visual examination of the Unit 2

diesel generator filter housing and external surfaces for evidence of general, crevice, or pitting corrosion. In addition, the inspectors reviewed completed WO 00738013, related to AMP B.1.23I, which credits the One-Time Inspection Program to perform a one-time visual examination of the Unit 2 condensate valve, 2-3399-547, also for evidence of general, surface, or pitting corrosion. Lastly, the inspectors reviewed completed WO 001127927, related to AMP B.1.23J, which credits the One-Time Inspection Program to perform a one-time UT examination of a nozzle-to-safe end weld of the capped control rod drive (CRD) return system. The inspectors had no concerns with the reviewed activities.

(4) (B.1.24) Selective Leaching of Materials (Commitment No. 24)

The Selective Leaching of Materials AMP is a new program. The Selective Leaching of Materials AMP consists of numerous one-time inspections to determine if selective leaching of materials is occurring. The scope of the program includes susceptible components within the scope of license renewal that are exposed to chemically treated water, de-mineralized water, raw water and ground water, and moist ventilation and gas environments. Susceptible component materials are gray cast iron, copper alloys with less than 85 percent copper, aluminum bronze, and Muntz metal.

The program is consistent with the ten elements of Aging Program XI.M33, "Selective Leaching of Materials," specified in NUREG-1801 with the following exceptions. NUREG-1801 indicates that the Selective Leaching of Materials AMP includes a one-time hardness measurement of a selected set of components. The Quad Cities program provides for visual examination in lieu of hardness testing.

The commitment documents that a sample of components that are made of susceptible materials will be visually inspected for evidence of selective leaching and that the sample will be expanded if failed conditions are identified.

To verify the program's effectiveness, the inspectors reviewed program documents and completed WO 00692750 results for a diesel generator cooling water drain valve.

While conducting the Phase I inspection of Quad Cities Unit 1 in the spring of 2011, an issue was identified which led the inspectors to question the adequacy of the selective leaching exams being performed. In a letter to staff dated November 20, 2003, the licensee indicated that visual inspections [for the Selective Leaching Program] would be performed in accordance with ASME Code Section XI Visual Testing (VT) -1. The inspectors, with concurrence from NRR, understood this to mean that the examinations would be performed with VT-1 qualified personnel following the requirements of ASME Section XI for VT-1 examinations. It was the licensee's belief that the commitment was limited to the use of VT-1 qualified personnel to perform the examinations but not to the requirements of the ASME Code. This issue was captured in the licensee's corrective action program.

At the time of the Unit 1 inspection, the licensee agreed, based on the verbiage, an ASME Code Section XI VT-1 should have been completed for the selective leaching exams. As such, the licensee intended to submit a commitment revision to clarify their position to NRR for revision and acceptability. However, while addressing the issue, the licensee concluded the scope was broader than originally identified in AMP B.1.24, and resolution was delayed to address the expanded issue. Their extent of condition identified a number of other AMP's with a commitment to perform ASME Code exams

and AR 01220460 was issued to track activities related this action. The revision has yet to be submitted and resolution has been extended at least six times.

The licensee proposes to revise these additional commitments to limit the visual examination requirements to the use of VT-1/VT-3 qualified personnel, performing limited VT exams. This is the same limitation as proposed for AMP B.2.6. However, while the Selective Leaching AMP language contains some ambiguity, the other AMPs specifically require ASME Code exams. Furthermore, Q2R21 is the last scheduled outage before the period of extended operation and the licensee has elected to perform all one-time and other license renewal inspections in accordance with the proposed revision and not as indicated in the AMP commitments. The inspectors noted the licensee had not submitted the commitment revision for the Office of Nuclear Reactor Regulation (NRR) review; therefore, the acceptability of their completed actions has not been determined. This will be inspected further during the subsequent IP 71003 inspection.

(5) (B.1.26) ASME Code Section XI, Subsection IWE (Commitment No. 26)

The ASME Section XI, Subsection IWE aging management program is an existing program and provides for primary containment inspections for loss of material. The program includes visual examination and limited surface or volumetric examination, when augmented examination is required. It is implemented through station plans and procedures and covers steel containment shells and their integral attachments; containment hatches and airlocks; seals, gaskets and moisture barriers; and pressure retaining bolting. This AMP will be enhanced as follows:

- The program will be based on the latest edition and addenda, which is approved by the NRC 12 months prior to the end of the current 120-month inspection interval.
- The program will be updated in accordance with 10 CFR 50.55(a).
- Requirements will be provided in procedures to ensure sand pocket drains are clear.
- The pressurized testing methodology will be credited for managing the aging of bellows.

The inspectors reviewed completed WO 01333337, which performed a surveillance of water leakage through the Dryer-Separator Pool, Spent Fuel Pool, and Drywell Liners to verify the integrity of these components. The inspectors had no concerns with the reviewed activities.

(6) (B.1.32) Protective Coating Monitoring and Maintenance Program (Commitment No. 32)

The inspectors reviewed the licensee's activities to implement Commitment Item No. 32 which documents the crediting of the existing Protective Coating Monitoring and Maintenance Program and commitments to enhance the program as follows: (1) visual inspection of Service Level I coatings near sumps or screens associated with the emergency core cooling system; (2) pre-inspection review of previous reports so that trends can be identified for the program; and (3) analysis of coating failures to determine reasons for failures.

The Protective Coating Monitoring and Maintenance Program provide for aging management of Service Level I coatings inside primary containment. Service Level I coatings are used in areas where the coating failure could adversely affect the operation of post-accident fluid systems and thereby impair safe shutdown. With the enhancements noted, the program is consistent with the ten elements of Aging Management Program XI.S8, "Protective Coating Monitoring and Maintenance Program," specified in NUREG-1801.

The coatings program uses the guidance provided in Electric Power Research Institute (EPRI) Technical Report TR-109937, "Guidelines on Nuclear Safety-Related Coating," to develop a corporate procedure for performing coating inspections.

The inspectors reviewed program documents and observed coating examinations of the "centipede" per direction of procedure ER-QC-330-1000. The inspection consisted of areas behind the missile shield, the vent headers, and the down-comers in accordance with WO01336189. As part of the same WO, the inspectors observed the coatings examinations of the drywell head, and levels three and four of the drywell. The inspectors had no concerns with the reviewed activities.

(7) (B.1.33) Electrical Cables and Connections Not Subject to 10 CFR 50.49 Environmental Qualification Requirements (Commitment No. 33)

The inspectors reviewed the licensee's activities to implement Commitment Item No. 33 which documents the agreement to develop a program consistent with NUREG-1801 AMP X1.E1 for electrical cables and connections installed in adverse localized environments not subject to 10 CFR 50.49 environmental qualification requirements.

The AMP for electrical cables and connections not subject to 10 CFR 50.49 environmental qualification requirements is a new program and is consistent with the ten elements of aging program XI.E1, "Electrical Cables and Connections Not Subject to 10 CFR 50.49 Environmental Qualification Requirements," specified in NUREG-1801. The program manages cables and connections within the scope of license renewal that are subject to an adverse environment. It also identifies and manages cables and connections subject to an adverse localized environment.

Adverse localized environments include areas with temperature, radiation, and moisture significantly higher than the maximum normal (plant design) ambient conditions, and could appreciably increase the rate of aging of the cables or connections or have an immediate adverse effect on operability.

Cables and connections subject to an adverse environment are managed by inspection of a sample of these components. Selected cables and connections from accessible areas represent, with reasonable assurance, the cables, and connections in adverse environments and are inspected for signs of accelerated age-related degradation. Additional inspections, repairs, or replacements are initiated as appropriate.

The Quad Cities AMP provides reasonable assurance the intended functions of the non-EQ electrical cables and connections within the scope of 10 CFR Part 54 exposed to adverse localized environments caused by heat, radiation or moisture will be maintained through the extended period of operation.

The licensee identified a representative sample of accessible electrical cables and connections installed in adverse localized environments and visually inspected for cable and connection jacket surface anomalies, such as discoloration, swelling, cracking, or surface contamination. The inspectors reviewed program documents and accompanied the licensee to observe a visual examination walkdown of cables in an adverse environmental area in the plant per direction of procedure MA-AA-723-500 and in accordance with WO 01415310 and WO 01418897. The inspectors had no concerns with the observed activities.

(8) (B.2.9) Periodic Inspection of Components Subject to Moist Environments (Commitment not included in Appendix A)

There are no commitments listed in Appendix A of NUREG 1796 for this program. The licensee created a new program as a stand-alone aging management program to manage components in moist air environments. This program is listed in UFSAR Appendix B and the intent of the program is noted in NUREG 1796, Section 3.0.3.18.

The program manages the loss of material for stainless steel, carbon steel, cast iron, aluminum, copper, brass, and bronze components in various systems exposed to moist air environments and subject to wetting conditions based on system operation. The program is not based on a Generic Aging Lessons Learned (GALL) report program; therefore, the applicant summarized the program in terms of the ten-element program as described in Branch Technical Position (BTP), Appendix A of the SRP-LR.

The staff determined that the program adequately addresses the ten program element as defined in the BTP, and that the program would provide adequately manage the aging effects for which it is credited.

The inspectors reviewed program documents and accompanied the licensee to observe visual examinations of the reactor core isolation cooling (RCIC) exhaust vacuum breaker valve, the RCIC torus suction check valve, and the high pressure coolant injection (HPCI) Turbine Casing in accordance with WO 00767199, WO 00709561, and WO 00880895. The inspectors had no concerns with the observed activities.

4OA6 Management Meetings

.1 Exit Meeting Summary

On March 28, 2012, the inspectors presented the inspection results to Mr. T. Hanley and other members of the licensee staff. The licensee acknowledged the issues presented.

The inspectors confirmed that none of the potential report input discussed was considered proprietary.

ATTACHMENT: SUPPLEMENTAL INFORMATION

SUPPLEMENTAL INFORMATION

KEY POINTS OF CONTACT

Licensee

T. Hanley, Site Vice President
W. Beck, Regulatory Assurance Manager
S. Darin, Engineering Director
A. Lewis, Regulatory Assurance
A. Misak, Nuclear Oversight Manager
M. Wagner, Regulatory Assurance

LIST OF ITEMS OPENED, CLOSED AND DISCUSSED

Opened, Closed, and Discussed

None

LIST OF DOCUMENTS REVIEWED

The following is a list of documents reviewed during the inspection. Inclusion on this list does not imply that the NRC inspectors reviewed the documents in their entirety, but rather, that selected sections of portions of the documents were evaluated as part of the overall inspection effort. Inclusion of a document on this list does not imply NRC acceptance of the document or any part of it, unless this is stated in the body of the inspection report.

40A5 Other Activities

License Renewal Program Basis Documents

Inspections Observed

WO 01336189-4; Inspect Coating Condition on Dry Well Liner Head; March 17, 2012

WO 01336189-1; Inspect Coating Condition on Levels 3 and 4 of Liner; March 17, 2012

WO 01336189-3; Inspect Coating Condition on the Centipede; March 21, 2012

WO 00767199; (LR) RCIC Turbine Exhaust Header Outboard Vacuum Breaker Disassemble/Inspection (IST); March 17, 2012

WO 01415310; (LR) Perform MA-AA-723-500 Att. 3 on MSIV Room Cables; March 17, 2012

WO 00880895; MM HPCI Turbine Overhaul FPR 10 Year PM Inspection; March 19, 2012

WO 00709561; RCIC Pump Torus Suction Check Valve Disassemble and Inspect (IST); March 17, 2012

WO 01418897; (LR) Perform MA-AA-723-500 on Heater Bay Cables; March 19, 2012

Procedures

ER-AA-335-004; Ultrasonic Measurement of Material Thickness and Interfering Conditions; Revision 5

ER-AA-330-008; Exelon Service Level 1, and Safety-Related (Service Level III) Protective Coatings; Revision 8

GEH-UT-601; Procedure for Ultrasonic Thickness Measurements for Flow Accelerated Corrosion (FAC); Revision 6

ER-AA-430-1001; Conduct of Flow Accelerated Corrosion Activities; Revision 7

ER-AA-430-1003; Flow Accelerated Corrosion Program Performance Indicators; Revision 1

ER-QC-330-1000; Primary Containment and Coating Inspections; Revision 2

LS-AA-120; Issues Identification and Screening Process; Revision 14

MA-AA-723-500; Inspection of Non EQ Cables and Connections for Managing Adverse Localized Environments; Revision 4

Completed Work Orders Reviewed

WO 00592895; One-Time Inspection DG Cooling Water Pump Cooler Inlet Valve; August 4, 2011

WO 01405244; License Renewal Selective Leaching Exam Incomplete for 0-4199-112A-D; May 27, 2011

WO 01333337; Dryer Separator Pool, Spent Fuel Pool, Drywell Liner Drain; March 21, 2012

WO 00692750; On-Time Inspect U2 DG Cool Water Drain Valve (License Renewal); September 5, 2008

WO 00725725; DG Engine Inspection; November 3, 2005

WO 00738013; LR – One-Time Inspection Condensate Valve, 2-3399-547; March 23, 2012

WO 01127927; One-Time UT Examination of a Nozzle-to-Safe End Weld of the Capped CRD Return System; March 22, 2010

AR's Generated

AR 01343538; Q2R21 LL LR Coating Inspections Lessons Learned; March 21, 2012

AR 01346172; LR – Commitment Change Regarding Eddy Current Testing; March 27, 2012

AR 01343655; Valve Missing Body to Bonnet Bolt; March 20, 2012

AR's Reviewed

AR 01343713; LR-RCIC 2-1399-103 Inspection; March 21, 2012

AR 01045627; MS FAC Pipe Replacement WO 01218094 Deferred to Q2R21; March 21, 2010

AR 01250164; Q1R22 FAC Support Activities: RX Feed Pump Room and Turbine Building Hallway; August 11, 2011

AR 01220460; License Renewal Requirements Regarding Visual Inspections; May 20, 2011

AR 01250164; Q1R22 FAC Support Activities: RX Feed Pump Room and Turbine Building Hallway; August 11, 2011

AR 01216329; PSU-Q1R21 FAC: 1-1SSH3-5" Unsatisfactory on Steam Seal Header; May 15, 2011

AR 00322402; PSU Coating Deficiencies Inside Drywell and Torus; April 8, 2005

AR 01345139; Water Behind Torus in Reactor Building Basement; March 24, 2012

Miscellaneous

Report 1011838; Recommendations for an Effective Flow-Accelerated Corrosion Program (NSAC-202L-R3); May 2006

LIST OF ACRONYMS USED

ADAMS	Agency wide Document Access Management System
ASME	American Society of Mechanical Engineers
AMP	Aging Management Program
AR	Action Request
BTP	Branch Technical Position
CFR	Code of Federal Regulations
CRD	Control Rod Drive
EGC	Exelon Generating Company, LLC
EPRI	Electric Power Research Institute
EQ	Environmental Qualification
FAC	Flow Accelerated Corrosion
GALL	Generic Aging Lessons Learned
HPCI	High Pressure Coolant Injection
IR	Inspection Report
ISI	Inservice Inspection Program
LR	License Renewal
NDE	Nondestructive Examination
NRC	U.S. Nuclear Regulatory Commission
NRR	Nuclear Reactor Regulation
NUREG	Nuclear Regulatory
PARS	Publicly Available Records
RAI	Request for Additional Information
RISI	Risk Informed Inservice Inspection
RCIC	Reactor Core Isolation Cooling
SSC	Structures, Systems, Components
UT	Ultrasonic Testing
UFSAR	Updated Final Safety Analysis Report
VT	Visual Testing
WO	Work Order

Mr. Michael J. Pacilio
Senior Vice President, Exelon Generation Company, LLC
President and Chief Nuclear Officer (CNO), Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

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Sincerely,
/RA/

Ann Marie Stone, Chief
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Letter to Mr. Michael Pacilio from Ms. Ann Marie Stone dated May 8, 2012.

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