

214 East 23rd Street Cheyenne, WY 82001 307-634-2273 www.crmcwy.org RECEIVED APR 1 6 2012 DNMS

6 April 2012

Michelle Simmons, Health Physicist Nuclear Regulatory Commission, Region IV 612 East Lamar Blvd., Suite 400 Arlington, Texas 76011-4125

Subj: AMENDMENT OF LICENSE NO. 49-01380-01

Dear Ms Simmons;

I am writing this letter to request the following changes to our license:

- To remove from our Materials License the area of use 2301 House Avenue, suite 301 Cheyenne Wyoming. A closeout survey was performed by Pacific Health Physics Inc. This survey is enclosed for your review.
- To remove from our Materials License the Treadmill rooms 6109 and 6107 at 214 east 23rd Street Cheyenne Wyoming. A closeout survey was performed by Radiology staff. This survey is also enclosed for your review.

If you have any questions I can be reached at 307.633.7838. Or you can e-mail me at todd.christensen@crmcwy.org Thank you . Sincercly,

Jo A. Christon

(Todd A. Christensen MS DABR Radiation Safety Officer Cheyenne Regional Medical Center 214 East 23rd Street Cheyenne WY 82001



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Jo A. Christon

(Fodd A. Christensen MS DABR Radiation Safety Officer Cheyenne Regional Medical Center 214 East 23rd Street Cheyenne WY 82001

PACIFIC HEALTH PHYSICS, Inc. 14603 SE 173rd Street, Renton, WA 98058

14603 SE 173rd Street, Renton, WA 98058 (425) 228-2932 - Fax (425) 271-6698 αβγ

March 14, 2012

Todd Christensen, MS Radiation Safety Officer Cheyenne Regional Medical Center Cheyenne, WY 82001

RECEIVED APR 1 6 2012

DNMS

Dear Mr. Christensen:

Enclosed please find the Closeout Survey Report and related attachments for your nuclear cardiology clinic. In summary, we did not identify any fixed or removable radioactivity in the locations described in the report. Therefore, the nuclear cardiology imaging room, treadmill rooms, and radiopharmaceutical administration area (Patient Dressing Room) are suitable for release per NRC regulatory requirements.

Please call me if you have any questions concerning this report or the survey methods used during the performance of our work at your facility.

Sincerely yours,

Minihail C. Sminons

Health Physicist President

Nuclear Cardiology Closeout Survey Report

On behalf of: Cheyenne Cardiology Associates dba Cheyenne Regional Medical Center Cheyenne, WY 82001



Section A – Closeout Survey Narrative

Cheyenne Cardiology Associates dba Cheyenne Regional Medical Center Cheyenne, WY 82001

Survey Certification

Cheyenne Cardiology Associates dba Cheyenne Regional Medical Center Cheyenne, WY 82001

The survey sample assay results contained herein certify that radioactive materials use and storage locations comprised of the nuclear cardiology imaging room, treadmill rooms, hot lab area, and the radiopharmaceutical administration room at the licensee's above location of use were determined to be free of removable and fixed radiological contamination at the conclusion of the closeout surveys described in this report.

Minhail C. Aminons

Date: March 12, 2012

Michael C. Simmons Health Physicist Pacific Health Physics, Inc.

Cheyenne Cardiology Associates dba Cheyenne Regional Medical Center Cheyenne, WY 82001

Closeout Survey Narrative Survey Date: March 12, 2012

1. List of isotopes, quantities, and dates of last use for radioactive materials used: Radioactive materials use included the radioactive materials listed below

1. Radionuclide	2. Monthly Quantity (curies/month)	3. Physical Form	4. Last Date of Use
А. 99mTc	A. 2.0	A. Liquid	A. 21 Feb 2012
B. I-131	B. 0.04	B. Solid	B. 15 April 2011
С.	C.	С.	C.
D.	D.	D.	D.
E.	E.	E.	Е.

2. The physical form for the isotopes was primarily liquid as radiopharmaceuticals used for nuclear cardiology studies, and solids as sodium iodide I-131 capsules <33 mCi per dosage for outpatient treatment of graves disease. Encapsulated materials were never opened or breached.

3. History of major spills or possible spread of contamination to inaccessible areas: No major spills or radiologic contamination.

4. Sealed source information: All qualifying sealed sources at this location were leak tested at six-month intervals. None of the sealed sources were ever found to be leaking.

5. The closeout survey results are as follows:

a. Dose rate surveys were performed at each numbered location on the attached survey maps. The survey results were obtained by holding the detector one meter above work surfaces, allowing the meter to stabilize, and noting the exposure rate. Representative background readings were obtained from a non-radioactive materials use area, (hallway outside the imaging room). Dose rate survey results did not locate any radiation above background readings, (14-16 uR/hour).

b. Scanning surveys were performed over 100% of affected work surfaces and fixtures/furnishings by moving the detector slowly back and forth at a speed of about one detector width per second and at a nominal distance of 1 centimeter. Representative background readings were obtained from a non-radioactive materials use area, (hallway outside the imaging room). Scanning survey results did not locate any radioactive contamination above background readings, (35 counts per minute).

c. Wipe surveys were taken from each numbered location on the attached survey maps. Removable activity sampling was performed using thin soft absorbent paper squares,

approximately 2 x 2 cm in size. Moderate pressure was applied to the smear with two or three fingers during surface wipe sampling. Wipe sampling occurred as a repeated "S" pattern over the entire sampling area. The smears were then placed in individual counting vials and identified with the numbered location or other pertinent information. One smear sample for removable contamination was obtained from each measurement location. The samples were assayed for gamma content using an Atomlab Model 950 multichannel analyzer interfaced with two inch drilled sodium iodide crystal housed in a shielded counting chamber. The counting windows were set to include the isotope energies for the radioactive materials listed in paragraph 1 above, (119-162, and for samples taken from the I-131 dose storage and dose administration areas 316-416 keV). Wipe survey results did not locate any radioactive contamination above background.

Survey results were compared with guideline values appended to this report as Section D.

6. Miscellaneous Surveys – wipe, scanning, and dose rate surveys were taken from the imaging room work table and file cabinet, and department light switches, door handles, and telephones. The survey results did not identify any fixed or removable radioactivity above background levels.

7. Survey instrument calibration data is appended to this report. The wipe samples were assayed in a two inch drilled well counter to evaluate the presence of gamma radiation. The well counter used to assay the wipe samples for gamma content has a proven efficiency of 16.9 % for 137 Cs.

The system MDA for gamma analysis was calculated using the Sorensen formula. The system MDA is 3*sqrt backround cpm / system efficiency = $3*(30)^{(1/2)} / 0.169 = 56$ dpm. The system MDC is 56 dpm / 2.22 e6 dpm / uCi = 2.5 e-5 uCi.

8. Decontamination of the nuclear cardiology imaging room, treadmill room, radiopharmaceutical administration area, and hot lab area is unnecessary since the closeout survey did not identify any fixed or removable radioactive contamination.

9. Radioactive materials use was confined to the use of solids and non-volatile solutions that did not require the use of a fume hood or consideration for airborne radioactive contamination.

Sample #	Sample CPM	Sample DPM		I-131 CPM	I-131 DPM
1	25		148		
2	17		101		
3	25		148		
4	27		160		
5	19		112		
6	22		130		
7	35		207		
8	26		154		
9	28		166		
10	27		160		
11	30		178		
12	30		178		
13	35		207		
14	33		195		
15	27		160		
16	30		178		
17	16		95		
18	19		112		
19	34		201		
20	27		160		
21	28		166		
22	20		118		
23	26		154		
24	24		142		
25	24		142		
26	34		201		
27	21		124		
28	28		166	24	4 142
29	24		142	23	3 136
30	21		124	32	2 189
31	34		201		
32	29		172		
33	29		172		
34	34		201		
35	21		124		
36	31		183		
37	28		166		
38	27		160		
39	24		142		
40	28		166		
41	29		172		
42	25		148		

Cheyenne Cardiology Associates, dba Cheyenne Regional Medical Center Gamma Wipe Sample Assay Results

Cheyenne Cardiology Associates, dba Cheyenne Regional Medical Center
Gamma Wipe Sample Assay Results

43	32	189		
44	30	178		
45	22	130		
46	25	148		
47	17	101		
48	30	178		
49	33	195		
50	26	154		
51	29	172		
52	21	124		
53	36	213		
54	26	154		
55	38	225		
56	29	172		
57	28	166		
58	32	189		
59	31	183		
60	33	195		
61	31	183		
62	22	130		
63	22	130		
64	33	195		
65	26	154		
66	21	124		
67	20	118		
68	24	142	28	166
69	25	148	25	148
70	29	172	33	195
71	28	166	32	189
72	33	195	27	160
73	23	136	30	178
74	29	172	33	195
75	23	136	32	189
	30	178	31	183

Sample #75 is a representative background sample taken from a common hallway located at the opposite end of the clinic from the imaging room.

Blank

Section B – Locator Maps and Grid Coordinates

Cheyenne Cardiology Associates dba Cheyenne Regional Medical Center Cheyenne, WY 82001

Pacific Health Physics, Inc. 14603 SE 173rd Street, Renton, WA 98058 425.228.2932 - Office 425.271.6698 - Fax Pacific Health Physics, Inc. Closeout Survey Facility - Cheyenne Regional Medical Center Location - Cardiology Imaging Room Date of Survey: 3/12/12



Scale = 3/8" = 1 Foot

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Pacific Health Physics, Inc. Closeout Survey Facility - Cheyenne Regional Medical Center Location - Cardiology Imaging, Treadmill 2 Date of Survey: 3/12/12



Pacific Health Physics, Inc. Closeout Survey Facility - Cheyenne Regional Medical Center Location - Cardiology Imaging, Treadmill 1 Date of Survey: 3/12/12



Pacific Health Physics, Inc. Closeout Survey Facility - Cheyenne Regional Medical Center Location - Cardiology Imaging, Dressing Rm. 2 Date of Survey: 3/12/12



Section C - Radiological Survey Instrumentation and Sensitivity

Cheyenne Cardiology Associates dba Cheyenne Regional Medical Center Cheyenne, WY 82001

Pacific Health Physics, Inc. 14603 SE 173rd Street, Renton, WA 98058 425.228.2932 - Office 425.271.6698 - Fax

SECTION C

CLOSEOUT SURVEY AND ANALYTICAL PROCEDURES

SURVEY PROCEDURES

Dose Rate Measurements:

Dose rate surveys were performed at each numbered location on the attached survey maps. The survey results were obtained by holding the detector, (Victoreen 450P ion chamber) one meter above work surfaces, allowing the meter to stabilize, and noting the exposure rate. Representative background readings were obtained from a non-radioactive materials use area, (common hallway at the opposite end of the clinic). Dose rate survey results did not locate any radiation above background readings, (14-16 uR/hour).

Surface Scans

Scanning surveys were performed by passing the detector slowly over surfaces of interest at a speed of 1 detector width per second. The distance between the detector and the surface was maintained at a minimum - nominally about 1 centimeter. A thin window halogen quenched GM pancake detector coupled to a portable survey meter with an audible indicator was used to scan the floors, cabinetry, shelves, equipment, and room fixtures of the surveyed areas. Identification of elevated levels was based on increases in the audible signal from the survey instrument.

Removable Activity Measurements

Removable activity levels were determined using pre-numbered thin soft absorbant paper squares, approximately 2 x 2 cm. Moderate pressure was applied to the wipe with two or three fingers during surface wipe sampling. Wipe sampling occurred as a repeated "S" pattern over the entire work surface or piece of equipment. The wipes were then placed in individual counting vials and identified with the numbered location or other pertinent information. One wipe sample for removable contamination was obtained from each measurement location. Representative background wipe samples were obtained from a non-radioactive materials use area, (common hallway at the opposite end of the clinic). The wipe survey results did not locate any radiation above background readings, (refer to data sheets).

ANALYTICAL PROCEDURES

Removable Activity

Gross Gamma Counting

Wipe samples were assayed in a Atomlab Model 950 two inch drilled sodium iodide crystal shielded well counter. The counting system was calibrated with a NIST Cs-137 rod source immediately prior to the sample assays. The samples were individually counted for one minute each. The assay results are detailed in a separate worksheet.

Pacific Health Physics, Inc. Facility: Cheyenne Cardiology Associates dba Cheyenne Regional Medical Center Nuclear Cardiology Closeout Survey

Gamma Counter Sensitivity And Detection Limits

The analytical data presented in this report represent the 99 % confidence level for that data. These data were calculated based on gross sample count levels and the associated background count levels. When the net sample count was less than 3 multiplied by the statistical deviation of the background count $[3^* (background CPM)^{1/2}] / total instrument efficiency], the sample concentration was reported as less than the detection limit of the detection system. Because of variations in background levels and measurement efficiencies, the detection limits may differ from sample to sample and from instrument to instrument.$

Radiation Survey Instrument Calibration And Quality Assurance

Portable survey equipment is calibrated on an annual frequency, see attached calibration documents. The well counter was calibrated on March 12, 2012. Survey instrument calibration documents are appended to this section.

Calibration of field and laboratory instrumentation is based on standards and sources traceable to the National Institute of Standards and Traceability.

PHP quality control procedures include:

- Daily or each time used instrument background and constancy check source/NIST source measurements to confirm that equipment operation is within acceptable limits,
- · Documents review to ensure agreement between survey findings and report data,
- Training and certification of individuals supervising or performing the survey procedures.

Instrument Sensitivity

1. The following calculations are for surveys, including scanning surveys performed in support of the closeout survey for the Cheyenne Cardiology location of use. The survey instrument is a Ludlum Model 14C meter with a Ludlum Model 44-9 pancake detector. The calibration data for the survey instrument are from 30 August 2011.

2. NUREG-1556, Volume 11, Table S.5, has release criteria for the radionuclides commonly used in research. A Historical Site Analysis or appropriate survey results should be used to determine the % contribution for each radionuclide. In this case, a typical % contribution for the two radionuclides is shown in the table below. Since the radiation safety practices for the two radionuclides are equivalent, one method for the closeout surveys is to use results for ^{99m}Tc as a basis to conclude scanning instrument sensitivity.

Radionuclide	Release Limit Maximum (dpm/100 cm ²)	% Contribution
¹³¹ J	3,000	2% **
^{99m} Tc	15,000	98%

**I-131 contribution is negligible due to physical decay; according to the HSA the last date of use was 15 April 2011.

3. The sample calculations below are based on NUREG-1575, "Multi-Agency Radiation Survey and Site Investigation Manual, (MARSSIM)."

4. The survey instrument used for surface scans for fixed radioactivity (either scanning or direct measurement surveys) must be sufficiently sensitive to detect the radionuclides present. The total efficiency for a survey instrument is the sum of the efficiencies for individual radionuclides. The total efficiency was calculated using the manufacturer's calibration data.

Efficiency_{total} = (0.05)(0.25) = 0.012 or about 1% total efficiency.

Where 5% is the 2 II 14 C efficiency and 25 % is the β energy efficiency per ISO 7503.

5. The MDC_{static} for a survey instrument should be less than 50% of the RLw. The expression for MDC_{static} is below where b is the usual background counting rate (e.g., 35 cpm for the survey instrument):

MDC _{static} = $\frac{3 * 4.65 (b)^{1/2}}{\text{Efficiency total}}$

and 435 dpm / 15,000 dpm = $\leq 3\%$ of the RLw

6. The minimum detectable count rate (MDCR) and scan MDC are calculated as follows.

$$MDCR = d' * \frac{sqrt b_i}{(i * sqrt P)}$$

Where

 $b_i = Background counts in the observations interval (35 cpm X 2 s ÷ 60 s = 1.16 counts)$ d' = Detectability index (2.32 from MARSSIM) P = Surveyor efficiency relative to the ideal observer (0.5 from MARSSIM) i = Survey observation interval of 2 seconds¹

MDCR = (2.32) (sqrt 1.16) / (sqrt 0.5) (2s) = 1.75 cps

(1.75 cps)(60 s / m) = 105 cpm

Scan MDC = MDCR / Efficieny_{total} = dpm / 100 cm^2

Scan MDC = 105 cpm / $0.012 = \frac{8750 \text{ dpm}}{100 \text{ cm}^2}$

7. Conclusion: The survey instrument is adequate to use for surface scans for fixed radioactivity for the use scenario described above. Surface scan results that do not exceed 105 cpm using acceptable methods and techniques are less than the weighted release limits.

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Pacific Health Physics, Inc. Well Counter Calibration and Test Report

Facility:	Cheyenne Regional Medical Center	Calibration Date:	12-Mar-12
	Cheyenne, Wyoming		
Equipment:	Well Counter - #2250510226		

Calibration Information:

NIST Standard:	Cs-137		Standard Bkg. CPM: 60	
NIST Standard Activity:	0.35900uCi		Standard CPM: 135066	
NIST Standard DPM:	796980			
Lower Limit of Detection:	137.12 DPM		Eff: 0.169	
Minimum Detectable Activity:	6.17654E-05	uCi		
Efficiency = standard cpm	/ standard dpm			

Background count rate assumed based on independent measurements.

This well counter meets the lower limit of detection (LLD) for systems used to assay radioactive samples. An acceptable LLD is 2000 dpm based on Nuclear Regulatory Commission NUREG 1556 Volumes 9 and 11.

Report By:

Attiched C. Smines Digitally signed by Michael Simmons Date: 2012.03.13 11:58:02 -07'00'



CERTIFICATE OF CALIBRATION



Calibration Source: 1 Ci of Cs-137; Radiation output 335 mR/hr at 100 cm on December 14, 1982 (± 5%)

J.L. Shepherd, Model 28-6A with Amersham type X.19 source, S/N: 0568GN & Ludium Mini Pulser, S/N: 21053

CALIBRATION NOTES AND COMMENTS

RADIATION LEVELS ARE BASED ON STANDARDS WHOSE CALIBRATION ARE TRACEABLE TO THE N.I.S.T. THE FORMULA FOR % ERROR IS: (Measured reading - calculated reading)/calculated reading/100. ALL READINGS ARE CORRECTED FOR BACKGROUND RADIATION. ANY CORRECTIONS MADE TO THE SURVEY INSTRUMENT (e.g. ENERGY DEPENDENCE) ARE UP TO THE USER TO APPLY. CARE MUST BE USED IN APPLYING ANY SUCH FACTORS. DURING CALIBRATION THE FRONT OF THE PGM PROBE WAS POSITIONED PERPENDICULAR TO THE BEAM AXIS.

CHECK SOURCE MEASURMENT MADE WITHOUT THE BETA SHIELD.

Calibrated by:

Reviewed by:

ANNY SAVORY DEB SMITH ADIATION SAFETY OFFICER California Radioactive Material License #: 5218-36

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CERTIFICATE OF CALIBRATION

This is to certify that the instrument described below was calibrated by measurement in a radiation beam having a calibration which is traceable to the National Institute of Standards and Technology. This calibration has been performed with a Quality Assurance Program which complies with 10 CFR 50, Appendix B and ISO/IEC 17025:2005. Unless stated otherwise, the Calibration Values represent an average of at least two points on each scale. The "best" combined expanded uncertainty with a coverage factor of 2 (95% confidence) of the calibration is 8%. (Best refers to the uncertainty of the calibration of a high quality survey instrument.) Accredited Health Physics Society (HPS) calibration. Procedure: HPS 21

Caution: If a calibration due date is shown, it is shown for the convenience of the user only. The interval is not a recommendation by K + S but is based on either the customer's requirements or the manufacturer's recommendations. Any number of factors, such as time, environment and handling may cause the instrument to drift out of calibration prior to the indicated date.

This laboratory is accredited by the American Association for Laboratory Accreditation (A2LA) and the results shown in this report have been determined in accordance with the laboratory's terms of accreditation unless stated otherwise in this report.

MULTIPLY the reading of the instrument by the listed Calibration Value to obtain an exposure or exposure rate value.

Customer #:	3664		
Name	CHEYENNE REGIONAL MEDICAL CENT	ER Source 1	Cs-137
Address	RADIATION ONCOLOGY	Source 2	
Address	214 EAST 23RD STREET	Source 3	
Address	CHEYENNE, WY 82001	Probe Type:	ION CHAMBER
Туре	DIGITAL	Probe Mfgr:	
Mfgr	VICTOREEN	Probe Model:	
Model	450P	Probe Serial:	
Serial	2590	Probe Window:	
		Probe Orientation:	BOTTOM TOWARD SOURCE

Temp		Pressure	RH%	Check Source	Conditio	ons Rea	ading	Battery Cr	IK. ON
21 deg	C	749 mmHg	50	NO					
Status	HPS	Beam	Scale mR/h	Ref. Rate mR/h	Reading mR/h	Ref. Rate mR/h		Reading mR/h	Calibration Value
F	Y		500 uR/h	0.100 *	98 uR/h	0.40		407 uR/h	1.00
F	Y		5	2.5	2.40				
L.	Y		5	2.5	2.51	1.00		1.05	
L	Y	-	5	4.0	3.98				0.98
F	Y		50	10.0	10.2	40		39.5	1.00
F	Y		500	250	241				
L	Y		500	250	251	100		102	
L	Y	-	500	400	394			N.7.7	1.00
F	N		5 R/h	2500	2.49 R/h				
F	Y		5 R/h	1000	0.98 R/h				1.01
STA	TUS: F	= AS FOUND	: L = AS LEFT.	HPS: Accredited Proc	cedure = "Y" for y	/es, "N" for no.		Label	Cal by: REH

CAUTION: 1. Application of the factors to an individual measurement is the responsibility of the user.

The factors shown are not necessarily valid over the entire scale.

Exercise caution in the interpolation or extrapolation of the factors.

NOTE: Please report any problems with the service provided by this HPS accredited laboratory to the HPS Secretariat.

Comments * With "true" bkg exposure rate of 6.8 uR/h, instrument reading was 14 uR/h.

F	Replaced 2 (9V) #EN22 battery.		$\int \infty $		
Calibrated by:	Buluno Hucan	Reviewed by:	Hame Jon	Log: M-34	Page 194
Title:	Richard Hardison	Title:	Paul Sappenfield	Log: N-17	Page 211
	Calibration Technician		Laboratory Supervisor	Log:	Page

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Page 1 of 1

SECTION D

WAC 246-232-140 SCHEDULE D

ACCEPTABLE SURFACE CONTAMINATION LEVELS FOR UNCONTROLLED RELEASE OF FACILITIES AND EQUIPMENT *

Nuclide [A]	Average [B,C]	Maximum [B,D]	Removable [B,E]
U-nat, U-235, U-238	5,000 dpm	15,000 dpm	1,000 dpm
and associated decay products	$/100 \text{ cm}^2$	$/100 \text{ cm}^2$	$/100 \text{ cm}^2$
Transuranics, Ra-226	100 dpm	300 dpm	20 dpm
Ra-228, Th-230, Th-228, Pa-231, Ac-227, 1-125,1-129	$/100 \text{ cm}^2$	$/100 \text{ cm}^2$	$/100 \text{ cm}^2$
Th-nat, Th-232, Sr-90,	1,000 dpm	3,000 dpm	200 dpm
Ra-223, Ra-224, U-232, 1-126,1-131,1-133	/100 cm ²	$/100 \text{ cm}^2$	$/100 \text{ cm}^2$
Beta-gamma emitters	5,000 dpm	15,000 dpm	1,000 dpm
(nuclides with decay modes by other than alpha emission or spontaneous fission) except Sr-90 and others noted above.	/100 cm ²	/100 cm ²	/100 cm ²

*Also Regulatory Guide 8.23 and 1.86.

- [A] Where surface contamination by both alpha- and beta-gamma-emitting nuclides exists, the limits established for alpha- and beta-gamma- emitting nuclides should apply independently.
- [B] As used in this table, dpm (disintegrations per minute) means the rate of emission by radioactive material as determined by correcting the counts per minute observed by an appropriate detector for background, efficiency, and geometric factors associated with the instrumentation.
- [C] Measurements of average contaminate should not be averaged over more than 1 square meter. For objects of less surface area, the average should be derived for each such object.
- [D] The maximum contamination level applies to an area of not more than 100 cm².
- [E] The amount of removable radioactive material per 100 cm² of surface area should be determined by wiping that area with dry filter or soft absorbent paper, applying moderate pressure, and assessing the amount of radioactive material on the wipe with an appropriate instrument of known efficiency. When removable contamination on objects of less surface area is determined, the pertinent levels should be reduced proportionately and the entire surface should be wiped.



214 EAST 23RD STREET CHEYENNE, WY 82001 307·634·2273 www.crmcwy.org

ROOM 6109 CLOSEOUT

WIPE AND SURVEY RESULTS

AREA LOCATION:

FLOOR AT ENTRANCE: BKG 614 DPM / WIPE 0 DPM BKG 0.04 mr/hr SURVEY 0.04 mr/hr FLOOR BY TREADMILL: WIPE 17 DPM SURVEY 0.04 mr/hr

ROOM 6107 CLOSEOUT

WIPE AND SURVEY RESULTS

AREA LOCATION:

FLOOR AT ENTRANCE: BKG 614 DPM / WIPE 0 DPM BKG 0.04 mr/hr SURVEY 0.04 mr/hr FLOOR BY TREADMILL: WIPE 12 DPM SURVEY 0.04 mr/hr

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hevenne Regional Medical Center 214 Sast 23rd Street Thevenns, VM 62001

> Michelle Simmons, Health Physicist Nuclear Regulatory Commission, Region IV 612 East Lamar Blvd., Suite 400 Arlington, Texas 76011-4125

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DATED: 04/06/2012
view, has been performed.
LICENSE RENEWAL
our initial review.
ewal of the material(s) license identified rdingly, the license will not expire until
our taxpayer identification number.
lections/forms/nrc531.pdf
following number: (301) 415-5387
ee and Accounts Receivable Branch, in ntacted separately if there is a fee issue
IL CONTROL NUMBER. When ntrol number. Your application has t the technical review, which is ation (90 days for all other requests), formation. If you have any questions ct information is listed below:
n

Accounts Receivable/Payable and Program Code: 02230 Program Code: 02230 Status Code: Pending Amendment Fee Category: 70 Eve Category:	BETWEEN:	[FOR ARPB USE]
Accounts reconvoluent system and Regional Licensing Branches Program Code: 02230 Status Code: Program Code: Program Code: Status Code: Account Fin Assur Repd: N Program Code: Status Code: Account Fin Assur Repd: N Program Code: Status C	Appoints Dessivable (Devel-	INFORMATION FROM LTS
Regional Licensing Branches Status Code: Pending Amendment Fee Category: 7C Exp. Date: Fee Comments: CODE 13 Decom Fin Assur Read: N AREGON A.REGON 1. APPLICATION ATTACHED Anendment Applicant/Licensee: MEMORIAL HOSPITAL OF LARAMIE CTV: Previewed Dire: 3003496 Mail Control Number: 3003496 Janunt:	and	Program Code: 02230
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