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1	The Primary and Secondary Review Branches for "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants (NUREG-0800)," shown in the table applies to only to branches in the New Reactors Office (NRO) and for staff reviews associated with Early Site Permit (ESP), a Design Certification (DC), or Combined License (COL) or Standard Design Approval (SDA) or a Manufacturing License (ML) submitted in accordance with 10 CFR Part 52.					
2						
3			Post-Reorganization			
4	SRP Section	Section Title	Primary DIV	Primary Branch	Secondary Review DIV	Secondary Review Branch (Note 4)
5	1.0	Introduction and Interfaces	DNRL/DARR	LB1, LB2, LB3, LB4, E PB1, EPB2, APRB	All review organizations	
6	2.0	Site Characteristics and Site Parameters	DNRL/DARR	LB1, LB2, LB3, LB4, E PB1, EPB2, APRB	All SRP Chapter 2 review organizations	
7	2.1.1	Site Location and Description	DSEA	RHMB		
8	2.1.2	Exclusion Area Authority and Control	DSEA	RPAC/RDAT		
9	2.1.3	Population Distribution	DSEA	RPAC/RDAT	NSIR-DPR	LIB under EP
10	2.2.1-2.2.2	Identification of Potential Hazards in Site Vicinity	DSEA	RPAC/RDAT		
11	2.2.3	Evaluation of Potential Accidents	DSEA	RPAC/RDAT		
12	2.3.1	Regional Climatology	DSEA	RHMB		
13	2.3.2	Local Meteorology	DSEA	RHMB		
14	2.3.3	Onsite Meteorological Measurements Programs	DSEA	RHMB	NSIR/DPR/DDEP	NRLB
15	2.3.4	Short Term Dispersion Estimates for Accidental Atmospheric Releases	DSEA	RHMB		
16	2.3.5	Long-Term Atmospheric Dispersion Estimates for Routine Releases	DSEA	RHMB		
17	2.4.1	Hydrologic Description	DSEA	RHMB		
18	2.4.2	Floods	DSEA	RHMB	DE/DSRA	SEB1/SEB2/SPRA
19	2.4.3	Probable Maximum Flood (PMF) on Streams and Rivers	DSEA	RHMB	DSRA	SPRA
20	2.4.4	Potential Dam Failures	DSEA	RHMB	DSEA	RGS1/ RGS2
21	2.4.5	Probable Maximum Surge and Seiche Flooding	DSEA	RHMB	DSEA	RGS1/ RGS2
22	2.4.6	Probable Maximum Tsunami Flooding	DSEA	RHMB	DSEA	RGS1/ RGS2
23	2.4.7	Ice Effects	DSEA	RHMB		
24	2.4.8	Cooling Water Canals and Reservoirs	DSEA	RHMB		
25	2.4.9	Channel Diversions	DSEA	RHMB		
26	2.4.10	Flooding Protection Requirements	DSEA	RHMB	DE	SEB1/SEB2
27	2.4.11	Low Water Considerations	DSEA	RHMB		
28	2.4.12	Groundwater	DSEA	RHMB	DE	SEB1/SEB2
29	2.4.13	Accidental Releases of Radioactive Liquid Effluents in Ground and Surface Waters	DSEA	RHMB	DSEA	RGS1/RGS2
30	2.4.14	Technical Specifications and Emergency Operation Requirements	DSEA	RHMB	DSEA	RPAC
31	2.5.1	Basic Geologic and Seismic Information	DSEA	RGS1/RGS2		
32	2.5.2	Vibratory Ground Motion	DSEA	RGS1/RGS2		
33	2.5.3	Surface Faulting	DSEA	RGS1/RGS2		

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4						
34	2.5.4	Stability of Subsurface Materials and Foundations	DSEA	RGS1/RGS2		
35	2.5.5	Stability of Slopes	DSEA	RGS1/RGS2		
36	3.2.1	Seismic Classification	DE	EMB	DE	CIB
37	3.2.2	System Quality Group Classification	DE	EMB	DE	CIB
38	3.3.1	Wind Loads	DE	SEB1/SEB2		
39	3.3.2	Tornado Loads	DE	SEB1/SEB2		
40	3.4.1	Flood Protection for Onsite Equipment Failure	DSRA	BPFP/BPTS	DSEA	RHMB
41	3.4.2	Analysis Procedures	DE	SEB1/SEB2		
42	3.5.1.1	Internally Generated Missiles (Outside Containment)	DSRA	BPFP/BPTS		
43	3.5.1.2	Internally Generated Missiles (Inside Containment)	DSRA	BPFP/BPTS		
44	3.5.1.3	Turbine Missiles	DE	CIB	DE	SEB1/SEB2
45	3.5.1.4	Missiles Generated by Tornadoes and Extreme Winds	DSRA	BPFP/BPTS		
46	3.5.1.5	Site Proximity Missiles (Except Aircraft)	DSEA	RPAC	DSRA	BPFP/BPTS
47	3.5.1.6	Aircraft Hazards	DSEA	RPAC		
48	3.5.2	Structures, Systems, and Components To Be Protected From Externally Generated Missiles	DSRA	BPFP/BPTS		
49	3.5.3	Barrier Design Procedures	DE	SEB1/SEB2		
50	3.6.1	Plant Design for Protection Against Postulated Piping Failures in Fluid Systems Outside Containment	DSRA	BPFP/BPTS		
51	3.6.2	Determination of Rupture Locations and Dynamic Effects Associated with the Postulated Rupture of Piping	DE	EMB		
52	3.6.3	Leak-Before-Break Evaluation Procedures	DE	CIB	DSRA	BPFP/BPTS
53	3.7.1	Seismic Design Parameters	DE	SEB1/SEB2		
54	3.7.2	Seismic System Analysis	DE	SEB1/SEB2		
55	3.7.3	Seismic Subsystem Analysis	DE	SEB1/SEB2		
56	3.7.4	Seismic Instrumentation	DSEA	RGS1/RGS2		
57	3.8.1	Concrete Containment	DE	SEB1/SEB2		
58	3.8.2	Steel Containment	DE	SEB1/SEB2		
59	3.8.3	Concrete and Steel Internal Structures of Steel or Concrete Containments	DE	SEB1/SEB2		
60	3.8.4	Other Seismic Category I Structures	DE	SEB1/SEB2		
61	3.8.5	Foundations	DE	SEB1/SEB2		
62	3.9.1	Special Topics for Mechanical Components	DE	EMB		
63	3.9.2	Dynamic Testing and Analysis of Systems, Components, and Equipment	DE	EMB	DE	CIB
64	3.9.3	ASME Code Class 1, 2, and 3 Components, Component Supports, and Core Support Structures	DE	EMB		
65	3.9.4	Control Rod Drive Systems	DE	EMB		
66	3.9.5	Reactor Pressure Vessel Internals	DE	EMB		
67	3.9.6	Functional Design, Qualification, and Inservice Testing Programs for Pumps, Valves, and Dynamic Restraints	DE	CIB		
68	3.9.7	Risk-Informed Inservice Testing of Pumps and Valves	DE	CIB	DSRA	SPRA
69	3.9.8	Risk-Informed Inservice Inspection of Piping	DE	CIB	DSRA	SPRA
70	3.10	Seismic and Dynamic Qualification of Mechanical and Electrical Equipment	DE	EMB	DE	ICE
71	3.11	Environmental Qualification of Mechanical and Electrical Equipment	DE	ICE1/ICE2	DE	ICE,CIB
72	3.12	ASME Code Class 1, 2, and 3 Piping Systems and Associated Supports Design [new]	DE	EMB		
73	3.13	Threaded Fasteners - ASME Code Class 1, 2, and 3	DE	CIB		
74	BTP 3-1	Classification of Main Steam Components Other than the Reactor Coolant Pressure Boundary for BWR Plants	DE	EMB	DE	CIB
75	BTP 3-2	Classification of BWR/6 Main Steam and Feedwater Components Other than the Reactor Coolant Pressure Boundary	DE	EMB	DE	CIB
76	BTP 3-3	Protection Against Postulated Piping Failures Influid Systems Outside Containment	DSRA	BPFP/BPTS		
77	BTP 3-4	Postulated Rupture Locations in Fluid System Piping Inside and Outside Containment	DE	EMB		
78	4.2	Fuel System Design	DSRA	SRSB		
79	4.3	Nuclear Design	DSRA	SRSB		
80	4.4	Thermal and Hydraulic Design	DSRA	SRSB		

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4						
81	4.5.1	Control Rod Drive Structural Materials	DE	CIB	DE	CIB
82	4.5.2	Reactor Internal and Core Support Materials	DE	CIB	DE	CIB
83	4.6	Functional Design of Control Rod Drive System	DSRA	SRSB	DSRA	BFPF/BPTS
84	BTP 4.1	Westinghouse Constant Axial Offset Control (CAOC)	DSRA	SRSB		
85	5.2.1.1	Compliance With the Codes and Standards Rule, 10 CFR 50.55a	DE	EMB	DE	CIB
86	5.2.1.2	Applicable Code Cases	DE	EMB	DE	CIB
87	5.2.2	Overpressure Protection	DSRA	SRSB		
88	5.2.3	Reactor Coolant Pressure Boundary Materials	DE	CIB	DE	CIB
89	5.2.4	Reactor Coolant Pressure Boundary Inservice Inspection and Testing	DE	CIB	DE	CIB
90	5.2.5	Reactor Coolant Pressure Boundary Leakage Detection	DSRA	BFPF/BPTS		
91	5.3.1	Reactor Vessel Materials	DE	CIB	DE	CIB1/CIB2
92	5.3.2	Pressure-Temperature Limits, Upper-shelf Energy, and Pressurized Thermal Shock	DE	CIB		
93	5.3.3	Reactor Vessel Integrity	DE	CIB	DE	CIB1/CIB2
94	5.4	Components and Subsystem Design	DSRA	SRSB	Multiple as defined in the SRP	
95	5.4.1.1	Pump Flywheel Integrity (PWR)	DE	CIB		
96	5.4.2.1	Steam Generator Materials	DE	CIB		
97	5.4.2.2	Steam Generator Program	DE	CIB		
98	5.4.6	Reactor Core Isolation Cooling System (BWR)	DSRA	SRSB		
99	5.4.7	Residual Heat Removal (RHR) System	DSRA	SRSB		
100	5.4.8	Reactor Water Cleanup System (BWR)	DE	CIB, BWR only		
101	5.4.11	Pressurizer Relief Tank	DSRA	SRSB		
102	5.4.12	Reactor Coolant System High Point Vents	DSRA	SRSB		
103	5.4.13	Isolation Condenser System (BWR)	DSRA	SRSB		
104	BTP 5-1	Monitoring of Secondary Side Water Chemistry in PWR Steam Generators	DE	CIB		
105	BTP 5-2	Overpressure Protection of Pressurized-Water Reactors While Operating at Low Temperatures	DSRA	SRSB		
106	BTP 5-3	Fracture Toughness Requirements	DE	CIB		
107	BTP 5-4	Design Requirements of the Residual Heat Removal System	DSRA	SRSB		
108	6.1.1	Engineered Safety Features Materials	DE	CIB		
109	6.1.2	Protective Coating Systems (Paints) □ Organic Materials	DE	CIB		
110	6.2.1	Containment Functional Design	DSRA	SCVB		
111	6.2.1.1.A	PWR Dry Containments, Including Subatmospheric Containments	DSRA	SCVB		
112	6.2.1.1.B	Ice Condenser Containments	DSRA	SCVB		
113	6.2.1.1.C	Pressure-Suppression Type BWR Containments	DSRA	SCVB		
114	6.2.1.2	Subcompartment Analysis	DSRA	SCVB		
115	6.2.1.3	Mass and Energy Release Analysis for Postulated Loss of Coolant Accidents	DSRA	SCVB		
116	6.2.1.4	Mass and Energy Release Analysis for Postulated Secondary System Pipe Ruptures	DSRA	SCVB		
117	6.2.1.5	Minimum Containment Pressure Analysis for Emergency Core Cooling System Performance Capability Studies	DSRA	SCVB		
118	6.2.2	Containment Heat Removal Systems	DSRA	SCVB	DE	CIB
119	6.2.3	Secondary Containment Functional Design	DSRA	SCVB		
120	6.2.4	Containment Isolation System	DSRA	SCVB		
121	6.2.5	Combustible Gas Control in Containment	DSRA	SCVB		
122	6.2.6	Containment Leakage Testing	DSRA	SCVB		
123	6.2.7	Fracture Prevention of Containment Pressure Boundary	DE	CIB		

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124	6.3	Emergency Core Cooling System	DSRA	SRSB	misc. as stated in SRP	
125	6.4	Control Room Habitability System	DSRA	SCVB	DSEA, DE	RPAC/RDAT, CIB
126	6.5.1	ESF Atmosphere Cleanup Systems	DSRA	SCVB	DSEA	RPAC/RDAT
127	6.5.2	Containment Spray as a Fission Product Cleanup System	DE	CIB	DSRA	SCVB
128	6.5.3	Fission Product Control Systems and Structures	DSRA	SCVB	DSEA	RPAC
129	6.5.4	Ice Condenser as a Fission Product Cleanup System	DSRA	SCVB		
130	6.5.5	Pressure Suppression Pool as a Fission Product Cleanup System	DSEA	RPAC/RDAT	DSRA, DE	SCVB, CIB
131	6.6	Inservice Inspection of Class 2 and 3 Components	DE	CIB		
132	6.7	Main Steam Isolation Valve Leakage Control System (BWR)	DSRA	SCVB		
133	6.8	Reactor Coolant Depressurization Systems (PWR) [No longer being developed]	N/A	N/A		
134	BTP 6-1	PH for Emergency Coolant Water for Pressurized Water Reactors	DSRA	CIB		
135	BTP 6-2	Minimum Containment Pressure Model for PWR ECCS Performance Evaluation	DSRA	SCVB		
136	BTP 6-3	Determination of Bypass Leakage Paths in Dual Containment Plants	DSRA	SCVB		
137	BTP 6-4	Containment Purging During Normal Plant Operations	DE	SCVB		
138	BTP 6-5	Currently the Responsibility of Reactor Systems Piping from the RWST (or BWST) and Containment Sump(s) to the Safety Injection Pumps	DE	SCVB		
139	7.0	Instrumentation and Controls - Overview of Review Process	DE/NRR or NRO	EEB/ICE, ^{(NOB)(1)}		
140	Appendix 7.0-A	Review Process for Digital Instrumentation and Control Systems	DE/NRR or NRO	EEB/ICE		
141	7.1	Instrumentation and Controls - Introduction	DE/NRR or NRO	EEB/ICE		
142	Appendix 7.1-A	Acceptance Criteria and Guidelines for Instrumentation and Control Systems Important to Safety	DE/NRR or NRO	EEB/ICE		
143	Appendix 7.1-B	Guidance for Evaluation of Conformance to IEEE Std. 279	DE/NRR or NRO	EEB/ICE		
144	Appendix 7.1-C	Guidance for Evaluation of Conformance to IEEE Std. 603	DE/NRR or NRO	EEB/ICE		
145	Appendix 7.1-D	Guidance for Evaluation of the Application of IEEE Std. 7-4.3.2	DE/NRR or NRO	EEB/ICE		
146	Table 7.1	Regulatory Requirements, Acceptance Criteria, and Guidelines For Instrumentation and Control Systems Important to Safety	DE/NRR or NRO	EEB/ICE		
147	7.2	Reactor Trip System	DE/NRR or NRO	EEB/ICE		
148	7.3	Engineered Safety Features Systems	DE/NRR or NRO	EEB/ICE		
149	7.4	Safe Shutdown Systems	DE/NRR or NRO	EEB/ICE		
150	7.5	Information Systems Important to Safety	DE/NRR or NRO	EEB/ICE		
151	7.6	Interlock Systems Important to Safety	DE/NRR or NRO	EEB/ICE		
152	7.7	Control Systems	DE/NRR or NRO	EEB/ICE		
153	7.8	Diverse Instrumentation and Control Systems	DE/NRR or NRO	EEB/ICE		
154	7.9	Data Communication Systems	DE/NRR or NRO	EEB/ICE		
155	Formerly Appendix 7-A	Branch Technical Positions	DE/NRR or NRO	EEB/ICE		
156	Appendix 7-A (formerly Appendix 7-B)	General Agenda, Station Site visits	DE/NRR or NRO	EEB/ICE		
157	Appendix 7-B (formerly Appendix 7-A)	Acronyms, Abbreviations, and Glossary	DE/NRR or NRO	EEB/ICE		
158	BTP 7-1	Guidance on Isolation of Low-Pressure Systems from the High-Pressure Reactor Coolant System	DE/NRR or NRO	EEB/ICE		
159	BTP 7-2	Guidance on Requirements of Motor-Operated Valves in the Emergency Core Cooling System Accumulator Lines	DE/NRR or NRO	EEB/ICE		
160	BTP 7-3	Guidance on Protection System Trip Point Changes for Operation with Reactor Coolant Pumps out of Service	DE/NRR or NRO	EEB/ICE		
161	BTP 7-4	Guidance on Design Criteria for Auxiliary Feedwater Systems	DE/NRR or NRO	EEB/ICE		
162	BTP 7-5	Guidance on Spurious Withdrawals of Single Control Rods in Pressurized Water Reactors	DE/NRR or NRO	EEB/ICE		

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163	BTP 7-6	Guidance on Design of Instrumentation and Controls Provided to Accomplish Changeover from Injection to Recirculation Mode	DE/NRR or NRO	EEB/ICE		
164	BTP 7-8	Guidance for Application of Regulatory Guide 1.22	DE/NRR or NRO	EEB/ICE		
165	BTP 7-9	Guidance on Requirements for Reactor Protection System Anticipatory Trips	DE/NRR or NRO	EEB/ICE		
166	BTP 7-10	Guidance on Application of Regulatory Guide 1.97	DE/NRR or NRO	EEB/ICE		
167	BTP 7-11	Guidance on Application and Qualification of Isolation Devices	DE/NRR or NRO	EEB/ICE		
168	BTP 7-12	Guidance on Establishing and Maintaining Instrument Setpoints	DE/NRR or NRO	EEB/ICE		
169	BTP 7-13	Guidance on Cross-Calibration of Protection System Resistance Temperature Detectors	DE/NRR or NRO	EEB/ICE		
170	BTP 7-14	Guidance on Software Reviews for Digital Computer-Based Instrumentation and Control Systems	DE/NRR or NRO	EEB/ICE		
171	BTP 7-16 (deleted)	Guidance on Level of Detail Required for Design Certification Applications Under 10 CFR Part 52	DE/NRR or NRO	EEB/ICE		
172	BTP 7-17	Guidance on Self-Test and Surveillance Test Provisions	DE/NRR or NRO	EEB/ICE		
173	BTP 7-18	Guidance on the Use of Programmable Logic Controllers in Digital Computer-Based Instrumentation and Control Systems	DE/NRR or NRO	EEB/ICE		
174	BTP 7-19	Guidance for Evaluation of Diversity and Defense-in-Depth and Diversity Computer-Based Instrumentation and Control Systems	DE/NRR or NRO	EEB/ICE	DSRA	SRSB
175	BTP 7-21	Guidance on Digital Computer Real-Time Performance	DE/NRR or NRO	EEB/ICE		
176	BTP 7-22 - Not Developed	Guidance on Digital Sampling	DE/NRR or NRO	EEB/ICE		
177	BTP 7-23 - Not Developed	Guidance on Digital Operating System Timing	DE/NRR or NRO	EEB/ICE		
178	8.1	Electric Power / Introduction	DE/NRR or NRO	EEB/ICE		
179	8.2	Offsite Power System	DE/NRR or NRO	EEB/ICE		
180	8.3.1	A C Power Systems (Onsite)	DE/NRR or NRO	EEB/ICE		
181	8.3.2	D C Power Systems (Onsite)	DE/NRR or NRO	EEB/ICE		
182	8.4	Station Blackout	DE/NRR or NRO	EEB/ICE		
183	8-A	General Agenda, Station Site Visits	DE/NRR or NRO	EEB/ICE		
184	BTP 8-1	Requirements on Motor-operated Valves in the ECCS Accumulator Lines	DE/NRR or NRO	EEB/ICE		
185	BTP 8-2	Use of Diesel-Generator Sets for Peaking	DE/NRR or NRO	EEB/ICE		
186	BTP 8-3	Stability of Offsite Power Systems,	DE/NRR or NRO	EEB/ICE		
187	BTP 8-4	Application of the Single Failure Criterion to Manually Controlled Electrically Operated Valves	DE/NRR or NRO	EEB/ICE		
188	BTP 8-5	Supplemental Guidance for Bypass and Inoperable Status Indication for Engineered Safety Features Systems	DE/NRR or NRO	EEB/ICE		
189	BTP 8-6	Adequacy of Station Electric Distribution System Voltages	DE/NRR or NRO	EEB/ICE		
190	BTP 8-7	Criteria for Alarms and Indications Associated with Diesel-Generator Unit Bypassed and Inoperable Status	DE/NRR or NRO	EEB/ICE		
191	9.1.1	Criticality Safety of Fresh and Spent Fuel Storage and Handling	DSRA	SRSB	DE	CIB
192	9.1.2	New and Spent Fuel Storage	DSRA	BPPF/BPTS	DE	CIB
193	9.1.3	Spent Fuel Pool Cooling and Cleanup System	DSRA	BPPF/BPTS	DE	CIB
194	9.1.4	Light Load Handling System (Related to Refueling)	DSRA	BPPF/BPTS		
195	9.1.5	Overhead Heavy Load Handling Systems	DSRA	BPPF/BPTS	DE	SEB1/SEB2
196	9.2.1	Station Service Water System	DSRA	BPPF/BPTS		
197	9.2.2	Reactor Auxiliary Cooling Water Systems	DSRA	BPPF/BPTS	DE	CIB
198	9.2.3 - deleted	Demineralized Water Makeup System				
199	9.2.4	Potable and Sanitary Water Systems	DSRA	BPPF/BPTS		
200	9.2.5	Ultimate Heat Sink	DSRA	BPPF/BPTS		
201	9.2.6	Condensate Storage Facilities	DSRA	BPPF/BPTS		
202	9.3.1	Compressed Air System	DSRA	BPPF/BPTS		
203	9.3.2	Process and Post Accident Sampling Systems	DE	CIB	as defined in the SRP	
204	9.3.3	Equipment and Floor Drainage System	DSRA	BPPF/BPTS		
205	9.3.4	Chemical and Volume Control System (PWR) Including Boron Recovery System	DSRA	SRSB	DE	CIB

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206	9.3.5	Standby Liquid Control System (BWR)	DSRA	SRSB		
207	9.4.1	Control Room Area Ventilation System	DSRA	SCVB		
208	9.4.2	Spent Fuel Pool Area Ventilation System	DSRA	SCVB		
209	9.4.3	Auxiliary and Radwaste Area Ventilation System	DSRA	SCVB		
210	9.4.4	Turbine Area Ventilation System	DSRA	SCVB		
211	9.4.5	Engineered Safety Feature Ventilation System	DSRA	SCVB		
212	9.5.1	Fire Protection Program	DSRA	BFPF		
213	9.5.1.2	Risk Informed (RI) and Performance Based (PB) FP Program	DSRA	BFPF		
214	9.5.2	Communications Systems	DE/NRR or NRO	ICE/EEB		
215	9.5.3	Lighting Systems	DE/NRR or NRO	ICE/EEB		
216	9.5.4	Emergency Diesel Engine Fuel Oil Storage and Transfer System	DSRA	BFPF/BPTS	DE	CIB
217	9.5.5	Emergency Diesel Engine Cooling Water System	DSRA	BFPF/BPTS		
218	9.5.6	Emergency Diesel Engine Starting System	DSRA	BFPF/BPTS		
219	9.5.7	Emergency Diesel Engine Lubrication System	DSRA	BFPF/BPTS		
220	9.5.8	Emergency Diesel Engine Combustion Air Intake and Exhaust System	DSRA	BFPF/BPTS		
221	10.2	Turbine Generator	DSRA	BFPF/BPTS		
222	10.2.3	Turbine Rotor Integrity	DE	CIB		
223	10.3	Main Steam Supply System	DSRA	SBPA/SBPB		
224	10.3.6	Steam and Feedwater System Materials	DE	CIB1/CIB2	DE	CIB
225	10.4.1	Main Condensers	DSRA	BFPF/BPTS		
226	10.4.2	Main Condenser Evacuation System	DSRA	BFPF/BPTS		
227	10.4.3	Turbine Gland Sealing System	DSRA	BFPF/BPTS		
228	10.4.4	Turbine Bypass System	DSRA	BFPF/BPTS		
229	10.4.5	Circulating Water System	DSRA	BFPF/BPTS		
230	10.4.6	Condensate Cleanup System	DE	CIB		
231	10.4.7	Condensate and Feedwater System	DSRA	BFPF/BPTS		
232	10.4.8	Steam Generator Blowdown System (PWR)	DE	CIB	DE	ICE/ICB
233	10.4.9	Auxiliary Feedwater System (PWR)	DSRA	BFPF		
234	BTP 10-1	Design Guidelines for Auxiliary Feedwater System Pump Drive and Power Supply Diversity for Pressurized Water Reactor Plants	DSRA	BFPF/BPTS		
235	BTP 10-2	Design Guidelines for Avoiding Water Hammers in Steam Generators	DSRA	BFPF/BPTS		
236	11.1	Source Terms	DSEA	RPAC/RDAT		
237	11.2	Liquid Waste Management Systems	DSEA	RPAC	DSRA	BFPF/BPTS
238	11.3	Gaseous Waste Management Systems	DSEA	RPAC	DSRA	SCVB, BFPF/BPTS
239	11.4	Solid Waste Management Systems	DSEA	RPAC	DSRA	BFPF/BPTS
240	11.5	Process and Effluent Radiological Monitoring Instrumentation and Sampling Systems	DSEA	RPAC	DE	ICE/ICB
241	BTP 11-3	Design Guidance for Solid Radioactive Waste Management Systems Installed in Light-Water -Cooled Nuclear Power Reactor Plants	DSEA	RPAC	DSRA	BFPF/BPTS
242	BTP 11-5	Postulated Radioactive Releases Due to a Waste Gas System Leak or Failure	DSEA	RPAC		
243	BTP 11-6	Postulated Radioactive Releases Due to Liquid-Containing Tank Failures	DSEA	RPAC		
244	12.1	Assuring that Occupational Radiation Exposures Are As Low As Is Reasonably Achievable	DSEA	RPAC		
245	12.2	Radiation Sources	DSEA	RPAC		
246	12.3 - 12.4	Radiation Protection Design Features	DSEA	RPAC		
247	12.5	Operational Radiation Protection Program	DSEA	RPAC		
248	13.1.1	Management and Technical Support Organization	DCIP	COLP		
249	13.1.2 - 13.1.3	Operating Organization	DCIP	COLP		
250	13.2.1	Reactor Operator Requalification Program; Reactor Operator Training	DCIP	COLP		

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251	13.2.2	Non- Licensed Plant Staff Training	DCIP	COLP		
252	13.3	Emergency Planning	NSIR-DPR	NRLB		
253	13.4	Operational Programs	DNRL/DARR	LB3, LB4, EPB1, EP	Multi as defined by SRP	
254	13.5.1.1	Administrative Procedures - General	DCIP	COLP		
255	13.5.1.2(Deleted)	Administrative Procedures - Initial Test Program (Draft guidance, review scope incorporated into 14.2)	DCIP	COLP		
256	13.5.2.1	Operating and Emergency Operating Procedures	DCIP	COLP		
257	13.5.2.2(Deleted)	Maintenance and Other Operating Procedures (draft guidance, review scope possibly in 17.6)	DCIP	COLP		
258	13.6	Physical Security	NSIR-DSP	RSLB		
259	13.6.1	Physical Security - Combined License and Operating Reactors	NSIR-DSP	RSLB		
260	13.6.2	Physical Security - Design Certification	NSIR-DSP	RSLB		
261	13.6.3	Physical Security - Early Site Permit	NSIR-DSP	RSLB		
262	13.6.4	Access Authorization	NSIR-DSP	SPSB		
263	13.6.6	Cyber Security Program	NSIR-DSP	CSIRB		
264	13.7	Fitness for Duty - Intro	NSIR-DSP	SPSB		
265	13.7.1	Fitness for Duty - Operational	NSIR-DSP	SPSB		
266	13.7.2	Fitness for Duty - Construction	NSIR-DSP	SPSB		
267	14.2	Initial Plant Test Program - Design Certification and New License Applicants	DCIP	CQAB	DSRA	SRSB, BFPF/BPTS
268	14.2.1	Generic Guidelines for Extended Power Uprate Testing Programs				
269	14.3	Inspections, Tests, Analyses, and Acceptance Criteria	DNRL	B2, LB3, LB4, EPB1,	DCIP, DSRA	CITB, SPRA
270	14.3.1(Reserved))	Site Parameters - Inspections, Tests, Analyses, and Acceptance Criteria (guidance moved to Section 2.0)			DCIP	CITB
271	14.3.2	Structural and Systems Engineering - Inspections, Tests, Analyses, and Acceptance Criteria	DE	SEB1/SEB2	RA, DCIP, DSEA, N	BPTS, CITB, RPAC, DS
272	14.3.3	Piping Systems and Components - Inspections, Tests, Analyses, and Acceptance Criteria	DE	EMB	DCIP,DE	CITB,CIB
273	14.3.4	Reactor Systems - Inspections, Tests, Analyses, and Acceptance Criteria	DSRA	SRSB	DCIP	CITB
274	14.3.5	Instrumentation and Controls - Inspections, Tests, Analyses, and Acceptance Criteria	DE	ICE/ICB	DCIP	CITB
275	14.3.6	Electrical Systems - Inspections, Tests, Analyses, and Acceptance Criteria	DE	ICE/ICB	DCIP	CITB
276	14.3.7	Plant Systems - Inspections, Tests, Analyses, and Acceptance Criteria	DSRA	BFPF/BPTS, SCVB	DCIP	CITB
277	14.3.8	Radiation Protection - Inspections, Tests, Analyses, and Acceptance Criteria	DSEA	RPAC	DCIP	CITB
278	14.3.9	Human Factors Engineering - Inspections, Tests, Analyses, and Acceptance Criteria	DCIP	COLP	DCIP	CITB
279	14.3.10	Emergency Preparedness (formerly Initial Test Program and D-RAP) - Inspections, Tests, Analyses, and Acceptance Criteria	NSIR-DPR	NRLB	DCIP	CITB
280	14.3.11	Containment Systems - Inspections, Tests, Analyses, and Acceptance Criteria	DSRA	SCVB	DCIP	CITB
281	14.3.12	Physical Security Hardware - Inspections, Tests, Analyses, and Acceptance Criteria	NSIR-DSP	RSLB	DCIP	CITB
282	15.0	Introduction—Transient and Accident Analyses	DSRA	SRSB		
283	15.0.1	Radiological Consequence Analyses Using Alternate Source Terms	DSEA	RPAC		
284	15.0.2	Review of Transient and Accident Analysis Methods	DSRA	SRSB		
285	15.0.3	Design Basis Accident Radiological Consequence Analyses for Advanced Light Water Reactors	DSEA	RPAC	DSEA	RPAC

	A	B	C	D	E	F
	SRP Section	Section Title	Primary DIV	Primary Branch	Secondary Review DIV	Secondary Review Branch (Note 4)
4						
286	15.1.1 - 15.1.4	Decrease in Feedwater Temperature, Increase in Feedwater Flow, Increase in Steam Flow, and Inadvertent Opening of a Steam Generator Relief or Safety Valve	DSRA	SRSB		
287	15.1.5	Steam System Piping Failures Inside and Outside of Containment (PWR)	DSRA	SRSB		
288	15.1.5.A	Radiological Consequences of Main Steam Line Failures Outside Containment of a PWR	DSEA	RPAC		
289	15.2.1 - 15.2.5	Loss of External Load; Turbine Trip; Loss of Condenser Vacuum; Closure of Main Steam Isolation Valve (BWR); and Steam Pressure Regulator Failure (Closed)	DSRA	SRSB		
290	15.2.6	Loss of Nonemergency AC Power to the Station Auxiliaries	DSRA	SRSB		
291	15.2.7	Loss of Normal Feedwater Flow	DSRA	SRSB		
292	15.2.8	Feedwater System Pipe Breaks Inside and Outside Containment (PWR)	DSRA	SRSB		
293	15.3.1 - 15.3.2	Loss of Forced Reactor Coolant Flow Including Trip of Pump Motor and Flow Controller Malfunctions	DSRA	SRSB		
294	15.3.3 - 15.3.4	Reactor Coolant Pump Rotor Seizure and Reactor Coolant Pump Shaft Break	DSRA	SRSB		
295	15.4.1	Uncontrolled Control Rod Assembly Withdrawal from a Subcritical or Low Power Startup Condition	DSRA	SRSB		
296	15.4.2	Uncontrolled Control Rod Assembly Withdrawal at Power	DSRA	SRSB		
297	15.4.3	Control Rod Misoperation (System Malfunction or Operator Error)	DSRA	SRSB		
298	15.4.4 - 15.4.5	Startup of an Inactive Loop or Recirculation Loop at an Incorrect Temperature, and Flow Controller Malfunction Causing an Increase in BWR Core Flow Rate	DSRA	SRSB		
299	15.4.6	Inadvertent Decrease in Boron Concentration in the Reactor Coolant System (PWR)	DSRA	SRSB		
300	15.4.7	Inadvertent Loading and Operation of a Fuel Assembly in an Improper Position	DSRA	SRSB		
301	15.4.8	Spectrum of Rod Ejection Accidents (PWR)	DSRA	SRSB		
302	15.4.8.A	Radiological Consequences of a Control Rod Ejection Accident (PWR)	DSEA	RPAC		
303	15.4.9	Spectrum of Rod Drop Accidents (BWR)	DSRA	SRSB		
304	15.4.9.A	Radiological Consequences of Control Rod Drop Accident (BWR)	DSEA	RPAC		
305	15.5.1 - 15.5.2	Inadvertent Operation of ECCS and Chemical and Volume Control System Malfunction that Increases Reactor Coolant Inventory	DSRA	SRSB		
306	15.6.1	Inadvertent Opening of a PWR Pressurizer Pressure Relief Valve or a BWR Pressure Relief Valve	DSRA	SRSB		
307	15.6.2	Radiological Consequences of the Failure of Small Lines Carrying Primary Coolant Outside Containment	DSEA	RPAC		
308	15.6.3	Radiological Consequences of Steam Generator Tube Failure (PWR)	DSEA	RPAC		
309	15.6.4	Radiological Consequences of Main Steam Line Failure Outside Containment (BWR)	DSEA	RPAC		
310	15.6.5	Loss of Coolant Accidents Resulting From Spectrum of Postulated Piping Breaks Within the Reactor Coolant Pressure Boundary	DSRA	SRSB		
311	15.6.5.A	Radiological Consequences of a Design Basis Loss-of-Coolant Accident Including Containment Leakage Contribution	DSEA	RPAC	DE	CIB
312	15.6.5.B	Radiological Consequences of a Design Basis Loss-of-Coolant Accident: Leakage From Engineered Safety Feature Components Outside Containment	DSEA	RPAC		
313	15.6.5.D	Radiological Consequences of a Design Basis Loss-of-Coolant Accident: Leakage From Main Steam Isolation Valve Leakage Control System (BWR)	DSEA	RPAC		
314	15.7.3	Postulated Radioactive Releases Due to Liquid-Containing Tank Failures	DCIP	RPAC		
315	15.7.4	Radiological Consequences of Fuel Handling Accidents	DSEA	RPAC		
316	15.7.5	Spent Fuel Cask Drop Accidents	DSEA	RPAC		
317	15.8	Anticipated Transients Without Scram	DSRA	SRSB		
318	15.9	Boiling Water Reactor Stability	DSRA	SRSB		
319	16.0	Technical Specifications	DSRA	BPTS	Multiple	RSB, BPFP/BPTS, SP
320	16.1	Risk-Informed Decision Making: Technical Specifications	DCIP, DSRA	BPTS, CITB		
321	17.1	Quality Assurance During the Design and Construction Phases	DCIP	CQAB		
322	17.1.1 - [reference from RS - 002]	Early Site Permit Quality Assurance Measures	DCIP	CQAB		
323	17.2	Quality Assurance During the Operations Phase	DCIP	CQAB		
324	17.3	Quality Assurance Program Description	DCIP	CQAB		
325	17.4	Reliability Assurance Program	DSRA	SPRA		

	A	B	C	D	E	F
	SRP Section	Section Title	Primary DIV	Primary Branch	Secondary Review DIV	Secondary Review Branch (Note 4)
4						
326	17.5	Quality Assurance new section	DCIP	CQAB		
327	17.6	Maintenance Rule	DSRA	SPRA		
328	18.0	Human Factors Engineering Introduction	DCIP	COLP	DE	ICE/ICB
329	19.0	Probabilistic Risk Assessment and Severe Accident Evaluation	DSRA	SPRA	Multiple	Multiple
330	19.1	Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities	DSRA	SPRA		
331	19.2 (new)	Review of Risk Information Used to Support Permanent Plant-Specific Changes to the Licensing Basis: General Guidance	DSRA	SPRA	DCIP, DSRA	CTSB, BPTS, SRSB
332						
333		Notes:				
334		(1) The responsibility for electrical branch review has been moved to EEB/NRR				
335		(2) This table was formally commented by all review Divisions/Branches within NRO by YT-2012-0066 and comments suitably incorporated				
336		(3) The table also shows primary/Secondary Review branches pre and post reorganization within the New Reactor's Office				
337		(4) Where secondary review branch(es) are involved, the primary review branch should provide coordination and receive the inputs so that a single, integrated, SER input is provided to Projects.				
338						