UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

April 18, 2012

Vice President, Operations
Entergy Nuclear Operations, Inc.
Indian Point Energy Center
450 Broadway, GSB
P.O. Box 249
Buchanan, NY 10511-0249

SUBJECT: ACCEPTANCE REVIEW FOR PROPOSED LICENSE AMENDMENT TO ALLOW USE OF THE BACKUP SPENT FUEL POOL COOLING SYSTEM WHILE THE SPENT FUEL POOL COOLING SYSTEM IS OUT OF SERVICE - INDIAN POINT NUCLEAR GENERATING UNIT NO. 3 (TAC NO. ME8097)

Dear Sir or Madam:

By letter dated February 6, 2012, Entergy Nuclear Operations, Inc., the licensee, submitted a license amendment for Indian Point Nuclear Generating Unit No. 3. The proposed amendment would revise the Updated Final Safety Analysis Report (UFSAR) to allow use of the backup spent fuel pool cooling system (BSFPCS) when the spent fuel pool cooling system (SFPCS) is out of service.

The purpose of this letter is to provide the results of the U.S. Nuclear Regulatory Commission (NRC) staff's acceptance review of this amendment request. The acceptance review was performed to determine if there is sufficient technical information in scope and depth to allow the NRC staff to complete its detailed technical review. The acceptance review is also intended to identify whether the application has any readily apparent information insufficiencies in its characterization of the regulatory requirements or the licensing basis of the plant.

Consistent with Section 50.90 of Title 10 of the *Code of Federal Regulations* (10 CFR), an amendment to the license (including the technical specifications) must fully describe the changes requested, and following as far as applicable, the form prescribed for original applications. Section 50.34 of 10 CFR addresses the content of technical information required. This section stipulates that the submittal address the design and operating characteristics, unusual or novel design features, and principal safety considerations.

The NRC staff has reviewed your application and concluded that the information delineated in the enclosure to this letter is necessary to enable the NRC staff to make an independent assessment regarding the acceptability of the proposed amendment request in terms of regulatory requirements and the protection of public health and safety and the environment.

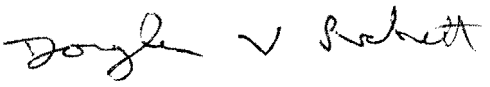
In order to make the application complete, the NRC staff requests that the licensee supplement the application to address the information requested in the enclosure by May 3, 2012. This will enable the NRC staff to complete its detailed technical review. If the information responsive to the NRC staff's request is not received by the above date, the application will not be accepted for review pursuant to 10 CFR 2.101, and the NRC staff will cease its review activities associated with the application. If the application is subsequently accepted for review, you will

be advised of any further information needed to support the NRC staff's detailed technical review by separate correspondence.

The information requested and associated time frame in this letter were discussed with your staff on April 16, 2012.

If you have any questions, please contact me at 301-415-1364 or Douglas.Pickett@nrc.gov.

Sincerely,



Douglas V. Pickett, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosure: As stated

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LIC-109 ACCEPTANCE REVIEW

PROPOSED LICENSE AMENDMENT TO ALLOW USE OF THE BACKUP

SPENT FUEL POOL COOLING SYSTEM WHEN THE SPENT FUEL

COOLING SYSTEM IS OUT OF SERVICE

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

OFFICE OF NUCLEAR REACTOR REGULATION

A. The application does not specifically address the findings of the 2001 Special Inspection of the BSFPCS and their resolution. This includes:

1. Reliability of the BSFPCS versus reliability of the SFPCS.

2. Use of the SFP High Temperature Alarm during periods of elevated SFP temperature.

3. Risk management actions to be put in place during use of the BSFPCS with SFPCS out of service.

B. In order to preserve existing margin, the heat removal capacity of the BSFPCS should not be considered when calculating the delay time prior to spent fuel offload from the reactor. The application does not clarify whether the BSFPCS heat removal capacity will be credited together with the SFPCS heat removal capacity when calculating this delay time. (As described in UFSAR Section 9.3.3).

Enclosure

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The information requested and associated time frame in this letter were discussed with your staff on April 16, 2012.

If you have any questions, please contact me at 301-415-1364 or Douglas.Pickett@nrc.gov.

Sincerely,

/ra/

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