



Christopher L. Burton  
Vice President  
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APR 2 2012

Serial: HNP-12-050

10 CFR 50.90

ATTN: Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555-0001

Shearon Harris Nuclear Power Plant, Unit No. 1  
Docket No. 50-400 / Renewed License No. NPF-63

Subject: License Amendment Request for Revision to Technical Specification Core  
Operating Limits Report for Realistic Large Break LOCA Analysis  
Revised Technical Specification Page

Reference: Letter from K. Holbrook to the U.S. NRC, "License Amendment Request for  
Revision to Technical Specification Core Operating Limits Report," Serial  
HNP-11-067 dated August 22, 2011, ADAMS Accession No. ML11238A077

Ladies and Gentlemen:

By letter dated August 22, 2011, Carolina Power & Light Company (CP&L), doing business as Progress Energy Carolinas, Inc., requested approval from the U.S. Nuclear Regulatory Commission (NRC) of a license amendment request (LAR). The proposed amendment would revise Technical Specification (TS) 6.9.1.6, "Core Operating Limits Report," to add a plant-specific methodology that implements AREVA's NRC-approved topical report EMF-2103(P)(A), "Realistic Large Break LOCA Methodology for Pressurized Water Reactors," Revision 0, and also EMF-2103(P)(A), "Realistic Large Break LOCA Methodology for Pressurized Water Reactors," Revision 2 or higher upon approval of the specific revision by the NRC.

CP&L rescinds our request for approval of EMF-2103(P)(A), Revision 2 or higher. The attached retyped Technical Specification page 6-24a has been revised accordingly.

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A001  
A002  
NR

This document contains no new Regulatory Commitment.

Please direct any questions regarding this submittal to Dave Corlett at (919) 362-3137.

Sincerely,

A handwritten signature in cursive script, reading "Christopher L. Burton".

CLB/jrc

Enclosed: Revised Technical Specification Page

cc: Mr. J. D. Austin, NRC Sr. Resident Inspector, HNP  
Mr. W. L. Cox, III, Section Chief N.C. DENR  
Ms. A. T. Billoch Colon, NRC Project Manager, HNP  
Mr. V. M. McCree, NRC Regional Administrator, Region II

## ADMINISTRATIVE CONTROLS

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### 6.9.1.6 CORE OPERATING LIMITS REPORT (Continued)

- d. XN-75-32(P)(A), "Computational Procedure for Evaluating Fuel Rod Bowing," approved version as specified in the COLR.  
(Methodology for Specification 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).
- e. EMF-84-093(P)(A), "Steam Line Break Methodology for PWRs," approved version as specified in the COLR.  
(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient, 3.1.3.5 - Shutdown Bank Insertion Limits, 3.1.3.6 - Control Bank Insertion Limits, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).
- f. ANP-3011(P), "Harris Nuclear Plant Unit 1 Realistic Large Break LOCA Analysis," Revision 1, as approved by NRC safety evaluation dated  
(Methodology for Specification 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).
- g. XN-NF-78-44(NP)(A), "A Generic Analysis of the Control Rod Ejection Transient for Pressurized Water Reactors," approved version as specified in the COLR.  
(Methodology for Specification 3.1.3.5 - Shutdown Bank Insertion Limits, 3.1.3.6 - Control Bank Insertion Limits, and 3.2.2 - Heat Flux Hot Channel Factor).