



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

April 2, 2012

Mr. Lawrence J. Weber  
Senior Vice President and  
Chief Nuclear Officer  
Indiana Michigan Power Company  
Nuclear Generation Group  
One Cook Place  
Bridgman, MI 49106

SUBJECT: D. C. COOK NUCLEAR PLANT (DCCNP), UNITS 1 AND 2 – CLOSEOUT OF INFORMATION REQUEST PURSUANT TO 50.54(f) RELATED TO THE ESTIMATED EFFECT ON PEAK CLADDING TEMPERATURE RESULTING FROM THERMAL CONDUCTIVITY DEGRADATION IN THE WESTINGHOUSE-FURNISHED REALISTIC EMERGENCY CORE COOLING SYSTEM EVALUATION (TAC NO. ME7947 AND ME7948)

Dear Mr. Weber:

By letter dated February 16, 2012 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML12041A384), the U.S. Nuclear Regulatory Commission (NRC) issued a letter in accordance with Title 10 of the *Code of Federal Regulations* (10 CFR), Part 50, Section 50.54(f). Pursuant to this regulation, the NRC staff requested that you provide further information regarding the effect of a potentially significant error, as defined in 10 CFR 50.46(a)(3)(i), associated with thermal conductivity degradation, on peak cladding temperature in the Westinghouse-furnished realistic emergency core cooling system evaluation models. The requested information would enable the NRC staff to determine compliance with the DCCNP, Units 1 and 2 licensing basis, which includes the requirements of 10 CFR 50.46(a)(3)(ii), concerning the reporting of errors and changes to the emergency core cooling system evaluation model.

Specifically, the subject letter required that you provide information within 30 days of February 16, 2012, to address the following issues:

- (1) An estimation of the effect of the thermal conductivity degradation error on the peak fuel cladding temperature calculation for the emergency core cooling system evaluations at DCCNP, Units 1 and 2.
- (2) A description of the methodology and assumptions used to determine the estimates. This description shall include consideration of experimental data relevant to thermal conductivity degradation and specific information regarding any computer code model changes which were necessary to address these data.

By letter dated March 19, 2012 (ADAMS Accession No. ML12088A104), you submitted a response to the 10 CFR 50.54(f) information request for DCCNP, Units 1 and 2. The NRC staff has reviewed your submitted information and determined that for DCCNP, Units 1 and 2, your

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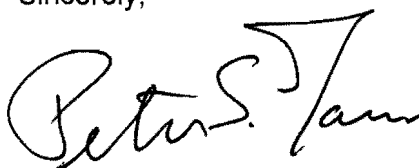
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response provides the information required to be responsive to the issues discussed above. In addition, the NRC staff determined that your response, which included a report pursuant to 10 CFR 50.46(a)(3)(ii), demonstrates that your facility has complied with the applicable reporting requirements. Therefore, the action under 10 CFR 50.54(f) is closed.

After review of the 10 CFR 50.46(a)(3) report, the NRC staff has determined that additional information will be required to verify the adequacy of your emergency core cooling system evaluation model. Therefore, while the information provided pursuant to 10 CFR 50.54(f) enabled the staff to close this action, the NRC staff has initiated separate actions to request for additional information under the auspices of 10 CFR 50.46.

If you have any questions on this matter, please contact me at 301-415-1451.

Sincerely,

A handwritten signature in black ink that reads "Peter S. Tam". The signature is written in a cursive style with a large, stylized initial "P".

Peter S. Tam, Senior Project Manager  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

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response provides the information required to be responsive to the issues discussed above. In addition, the NRC staff determined that your response, which included a report pursuant to 10 CFR 50.46(a)(3)(ii), demonstrates that your facility has complied with the applicable reporting requirements. Therefore, the action under 10 CFR 50.54(f) is closed.

After review of the 10 CFR 50.46(a)(3) report, the NRC staff has determined that additional information will be required to verify the adequacy of your emergency core cooling system evaluation model. Therefore, while the information provided pursuant to 10 CFR 50.54(f) enabled the staff to close this action, the NRC staff has initiated separate actions to request for additional information under the auspices of 10 CFR 50.46.

If you have any questions on this matter, please contact me at 301-415-1451.

Sincerely,

*/RA/*

Peter S. Tam, Senior Project Manager  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

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