



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

March 29, 2012

Mr. John T. Conway
Senior Vice President – Energy Supply
and Chief Nuclear Officer
Pacific Gas and Electric Company
Diablo Canyon Power Plant
77 Beale Street, Mail Code B32
San Francisco, CA 94105

**SUBJECT: DIABLO CANYON POWER PLANT, UNIT NOS. 1 AND 2 - ISSUANCE OF
AMENDMENTS RE: REVISION TO TECHNICAL SPECIFICATION 3.8.1, "AC
SOURCES – OPERATING," TO INCORPORATE TSTF-163, REVISION 2 (TAC
NOS. ME5939 AND ME5940)**

Dear Mr. Conway:

The U.S. Nuclear Regulatory Commission (NRC, the Commission) has issued the enclosed Amendment No. 211 to Facility Operating License No. DPR-80 and Amendment No. 213 to Facility Operating License No. DPR-82 for the Diablo Canyon Power Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated March 28, 2011, as supplemented by letter dated February 5, 2012.

The amendments revise TS 3.8.1, "AC [Alternating Current] Sources - Operating," to incorporate Technical Specification Task Force (TSTF) traveler TSTF-163, Revision 2, "Minimum vs. Steady State Voltage and Frequency," dated April 22, 1998. The amendments also revise the Final Safety Analysis Report Update (FSAR Update) to identify an exception to Revision 0 of NRC Regulatory Guide (RG) 1.9, "Application and Testing of Safety-Related Diesel Generators in Nuclear Power Plants" (issued as NRC Safety Guide 9, "Selection of Diesel Generator Set Capacity for Standby Power Supplies," dated March 10, 1971).

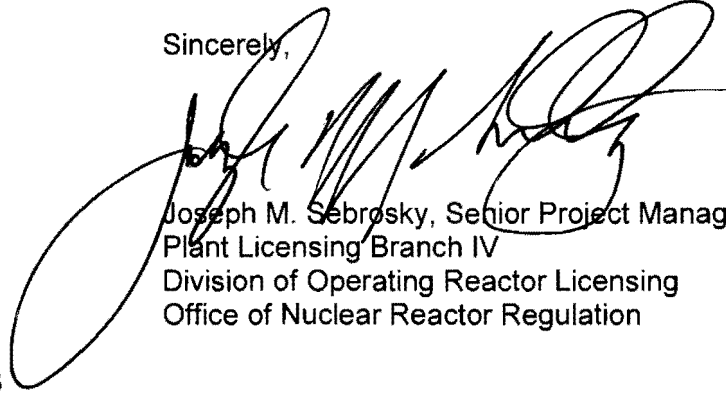
The amendments revise TS 3.8.1 surveillance requirements per TSTF-163, Revision 2, to verify minimum frequency and voltage, and steady state frequency and voltage within limits following diesel generator (DG) start. The FSAR Update is revised to specify an exception to RG 1.9, Revision 0, Regulatory Position C.4, for frequency recovery for the Auxiliary Feedwater pump loading for DGs 1-1, 1-3, 2-2, and 2-3.

J. Conway

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A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,

A large, stylized handwritten signature in black ink, likely belonging to Joseph M. Sebrosky, is written over the typed name and title.

Joseph M. Sebrosky, Senior Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-275 and 50-323

Enclosures:

1. Amendment No. 211 to DPR-80
2. Amendment No. 213 to DPR-82
3. Safety Evaluation

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**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

PACIFIC GAS AND ELECTRIC COMPANY

DOCKET NO. 50-275

DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 211
License No. DPR-80

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas and Electric Company (the licensee), dated March 28, 2011, as supplemented by letter dated February 5, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. DPR-80 is hereby amended to read as follows:

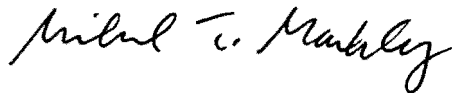
(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 211, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

In addition, the license is amended to authorize changes to the Final Safety Analysis Report Update (FSARU) Sections 8.1.4.3, "Regulatory Guides," and 8.3.1.1.13.1, "Diesel Generator Unit Description," to identify an exception to Revision 0 of Regulatory Guide 1.9, "Application and Testing of Safety-Related Diesel Generators in Nuclear Power Plants." The changes to the FSARU shall be as set forth in the enclosure of the licensee's application dated March 28, 2011

3. This license amendment is effective as of its date of issuance and shall be implemented within 120 days from the date of issuance. The licensee shall submit the changes authorized by this amendment with the next update of the FSARU in accordance with 10 CFR 50.71(e).

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Facility
Operating License No. DPR-80
and Technical Specifications

Date of Issuance: March 29, 2012



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PACIFIC GAS AND ELECTRIC COMPANY

DOCKET NO. 50-323

DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 213
License No. DPR-82

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas and Electric Company (the licensee), dated March 28, 2011, as supplemented by letter dated February 5, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. DPR-82 is hereby amended to read as follows:

(2) Technical Specifications (SSER 32, Section 8)* and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 213, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

In addition, the license is amended to authorize changes to the Final Safety Analysis Report Update (FSARU) Sections 8.1.4.3, "Regulatory Guides," and 8.3.1.1.13.1, "Diesel Generator Unit Description," to identify an exception to Revision 0 of Regulatory Guide 1.9, "Application and Testing of Safety-Related Diesel Generators in Nuclear Power Plants." The changes to the FSARU shall be as set forth in the enclosure of the licensee's application dated March 28, 2011

3. This license amendment is effective as of its date of issuance and shall be implemented within 120 days from the date of issuance. The licensee shall submit the changes authorized by this amendment with the next update of the FSARU in accordance with 10 CFR 50.71(e).

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Facility
Operating License No. DPR-82
and Technical Specifications

Date of Issuance: March 29, 2012

ATTACHMENT TO LICENSE AMENDMENT NOS. 211 AND 213
TO FACILITY OPERATING LICENSE NOS. DPR-80 AND DPR-82
DOCKET NOS. 50-275 AND 50-323

Replace the following pages of the Facility Operating License Nos. DPR-80 and DPR-82, and Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Operating License No. DPR-80

REMOVE

-3-

INSERT

-3-

Facility Operating License No. DPR-82

REMOVE

-3-

INSERT

-3-

Technical Specifications

REMOVE

3.8-4
3.8-5
3.8-7
3.8-8
3.8-10

INSERT

3.8-4
3.8-5
3.8-7
3.8-8
3.8-10

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This License shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The Pacific Gas and Electric Company is authorized to operate the facility at reactor core power levels not in excess of 3411 megawatts thermal (100% rated power) in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 211, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

(3) Initial Test Program

The Pacific Gas and Electric Company shall conduct the post-fuel-loading initial test program (set forth in Section 14 of Pacific Gas and Electric Company's Final Safety Analysis Report, as amended), without making any major modifications of this program unless modifications have been identified and have received prior NRC approval. Major modifications are defined as:

- a. Elimination of any test identified in Section 14 of PG&E's Final Safety Analysis Report as amended as being essential;

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This License shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The Pacific Gas and Electric Company is authorized to operate the facility at reactor core power levels not in excess of 3411 megawatts thermal (100% rated power) in accordance with the conditions specified herein.

(2) Technical Specifications (SSER 32, Section 8)* and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 213, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

(3) Initial Test Program (SSER 31, Section 4.4.1)

Any changes to the Initial Test Program described in Section 14 of the FSAR made in accordance with the provisions of 10 CFR 50.59 shall be reported in accordance with 50.59(b) within one month of such change.

*The parenthetical notation following the title of many license conditions denotes the section of the Safety Evaluation Report and/or its supplements wherein the license condition is discussed.

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.8.1.1	Verify correct breaker alignment and indicated power availability for each required offsite circuit.	In accordance with the Surveillance Frequency Control Program
SR 3.8.1.2	<p>-----NOTES-----</p> <ol style="list-style-type: none"> 1. Performance of SR 3.8.1.7 satisfies this SR. 2. All DG starts may be preceded by an engine prelube period and followed by a warmup period prior to loading. <p>-----</p> <p>Verify each DG starts from standby conditions and achieves steady state voltage ≥ 3785 V and ≤ 4400 V, and frequency ≥ 58.8 Hz and ≤ 61.2 Hz.</p>	In accordance with the Surveillance Frequency Control Program
SR 3.8.1.3	<p>-----NOTES-----</p> <ol style="list-style-type: none"> 1. DG loadings may include gradual loading as recommended by the manufacturer. 2. Momentary transients outside the load range do not invalidate this test. 3. This Surveillance shall be conducted on only one DG at a time. 4. This SR shall be preceded by and immediately follow without shutdown a successful performance of SR 3.8.1.2 or SR 3.8.1.7. <p>-----</p> <p>Verify each DG is synchronized and loaded and operates for ≥ 60 minutes at a load ≥ 2340 kW and ≤ 2600 kW.</p>	In accordance with the Surveillance Frequency Control Program
SR 3.8.1.4	Verify each day tank contains ≥ 250 gal of fuel oil.	In accordance with the Surveillance Frequency Control Program
SR 3.8.1.5	Check for and remove accumulated water from each day tank.	In accordance with the Surveillance Frequency Control Program
SR 3.8.1.6	Verify the fuel oil transfer system operates to transfer fuel oil from storage tanks to the day tank.	In accordance with the Surveillance Frequency Control Program

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.8.1.7 -----NOTE-----</p> <p>All DG starts may be preceded by an engine prelube period.</p> <p>-----</p> <p>Verify each DG starts from standby condition and achieves:</p> <p>a. In ≤ 10 seconds, voltage ≥ 3785 V and frequency ≥ 58.8 Hz; and</p> <p>b. Steady state voltage ≥ 3785 V and ≤ 4400 V, and frequency ≥ 58.8 Hz and ≤ 61.2 Hz.</p>	<p>In accordance with the Surveillance Frequency Control Program</p>
<p>SR 3.8.1.8 -----NOTE-----</p> <p>This Surveillance shall not normally be performed for automatic transfers in MODE 1 or 2. However, this Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced.</p> <p>-----</p> <p>Verify automatic and manual transfer of AC power sources from the normal offsite circuit to the alternate required offsite circuit and manual transfer from the alternate offsite circuit to the delayed access circuit.</p>	<p>In accordance with the Surveillance Frequency Control Program</p>
<p>SR 3.8.1.9 -----NOTES-----</p> <p>1. This Surveillance shall not normally be performed in MODE 1 or 2. However, this Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced.</p> <p>2. If performed with the DG synchronized with offsite power, it shall be performed at a power factor ≤ 0.9.</p> <p>-----</p> <p>Verify each DG rejects a load greater than or equal to its associated single largest post-accident load, and:</p> <p>a. Following load rejection, the frequency is ≤ 63 Hz;</p> <p>b. Within 2.4 seconds following load rejection, the voltage is ≥ 3785 V and ≤ 4400 V; and</p> <p>c. Within 2.4 seconds following load rejection, the frequency is ≥ 58.8 Hz and ≤ 61.2 Hz.</p>	<p>In accordance with the Surveillance Frequency Control Program</p>

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.8.1.12 -----NOTES-----</p> <ol style="list-style-type: none"> 1. All DG starts may be preceded by an engine prelube period. 2. This Surveillance shall not normally be performed in MODE 1 or 2. However, portions of the Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced. <p>-----</p> <p>Verify on an actual or simulated Safety Injection signal each DG auto-starts from standby condition and:</p> <ol style="list-style-type: none"> a. In ≤ 10 seconds after auto-start and during tests, achieves voltage ≥ 3785 V and frequency ≥ 58.8 Hz; b. Achieves steady state voltage ≥ 3785 V and ≤ 4400 V, and frequency ≥ 58.8 Hz and ≤ 61.2 Hz; c. Operates for ≥ 5 minutes; d. Permanently connected loads are energized from the alternate offsite power source; and e. Emergency loads are auto-connected through the ESF load sequencing timers to the alternate offsite power source. 	<p>In accordance with the Surveillance Frequency Control Program</p>
<p>SR 3.8.1.13</p> <p>Verify each DG's automatic trips are bypassed when the diesel engine trip cutout switch is in the cutout position and the DG is aligned for automatic operation except:</p> <ol style="list-style-type: none"> a. Engine overspeed; b. Generator differential current; and c. Low lube oil pressure; 	<p>In accordance with the Surveillance Frequency Control Program</p>

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.8.1.14 -----NOTES-----</p> <p>1. Momentary transients outside the load and power factor ranges do not invalidate this test.</p> <p>-----</p> <p>Verify each DG operating at a power factor ≤ 0.87 operates for ≥ 24 hours:</p> <p>a. For ≥ 2 hours loaded ≥ 2600 kW and ≤ 2860 kW; and</p> <p>b. For the remaining hours of the test loaded ≥ 2340 kW and ≤ 2600 kW.</p>	<p>In accordance with the Surveillance Frequency Control Program</p>
<p>SR 3.8.1.15 -----NOTES-----</p> <p>1. This Surveillance shall be performed within 5 minutes of shutting down the DG after the DG has operated ≥ 2 hours loaded ≥ 2340 kW and ≤ 2600 kW.</p> <p>Momentary transients outside of load range do not invalidate this test.</p> <p>2. All DG starts may be preceded by an engine prelube period.</p> <p>-----</p> <p>Verify each DG starts and achieves:</p> <p>a. In ≤ 10 seconds, voltage ≥ 3785 V and frequency ≥ 58.8 Hz; and</p> <p>b. Steady state voltage ≥ 3785 V and ≤ 4400 V, and frequency ≥ 58.8 Hz and ≤ 61.2 Hz.</p>	<p>In accordance with the Surveillance Frequency Control Program</p>
<p>SR 3.8.1.16 -----NOTE-----</p> <p>This Surveillance shall not normally be performed in MODE 1, 2, 3, or 4. However, this Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced.</p> <p>-----</p> <p>Verify each DG:</p> <p>a. Synchronizes with offsite power source while loaded with emergency loads upon a simulated restoration of offsite power;</p>	<p>In accordance with the Surveillance Frequency Control Program</p>

(continued)

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.8.1.19 (continued)	<p>Verify on an actual or simulated loss of offsite power signal in conjunction with an actual or simulated Safety Injection signal:</p> <ul style="list-style-type: none"> a. De-energization of emergency buses; b. Load shedding from emergency buses; and c. DG auto-starts from standby condition and: <ul style="list-style-type: none"> 1. energizes permanently connected loads in ≤ 10 seconds, 2. energizes auto-connected emergency loads through load sequencing timers, 3. achieves steady state voltage ≥ 3785 V and ≤ 4400 V, 4. achieves steady state frequency ≥ 58.8 Hz and ≤ 61.2 Hz, and 5. supplies permanently connected and auto-connected emergency loads for ≥ 5 minutes. 	In accordance with the Surveillance Frequency Control Program
SR 3.8.1.20	<p>-----NOTE----- All DG starts may be preceded by an engine prelube period.</p> <hr/> <p>Verify when started simultaneously from standby condition, each DG achieves:</p> <ul style="list-style-type: none"> a. In ≤ 10 seconds, voltage ≥ 3785 V and frequency ≥ 58.8 Hz; and b. Steady state voltage ≥ 3785 V and ≤ 4400 V, and frequency ≥ 58.8 Hz and ≤ 61.2 Hz. 	In accordance with the Surveillance Frequency Control Program



UNITED STATES
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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 211 TO FACILITY OPERATING LICENSE NO. DPR-80
AND AMENDMENT NO. 213 TO FACILITY OPERATING LICENSE NO. DPR-82
PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON POWER PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-275 AND 50-323

1.0 INTRODUCTION

By letter dated March 28, 2011 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML110880202), as supplemented by letter dated February 5, 2012 (ADAMS Accession No. ML12038A040), Pacific Gas & Electric (the licensee) requested an amendment to Facility Operating License Nos. DPR-80 and DPR-82 for Diablo Canyon Power Plant Units 1 and 2 (DCPP). The amendments would revise Technical Specification (TS) 3.8.1, "AC [Alternating Current] Sources - Operating," to incorporate Technical Specification Task Force (TSTF) traveler TSTF-163, Revision 2, "Minimum vs. Steady State Voltage and Frequency," dated April 22, 1998. The amendments also would revise the Final Safety Analysis Report Update (FSAR Update) to identify an exception to Revision 0 of U.S. Nuclear Regulatory Commission (NRC) Regulatory Guide (RG) 1.9, "Application and Testing of Safety-Related Diesel Generators in Nuclear Power Plants" (issued as NRC Safety Guide 9, "Selection of Diesel Generator Set Capacity for Standby Power Supplies," dated March 10, 1971).

The supplemental letter dated February 5, 2012, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on May 31, 2011 (76 FR 31375).

2.0 REGULATORY EVALUATION

DCPP FSAR Update Section 3.1.5.6, Criterion 24, "Emergency Power for Protection Systems (Category B)," states that, in the event of loss of all offsite power, sufficient alternate sources of power shall be provided to permit the required functioning of the protection systems. The DCPP FSAR Update discussion for Criterion 24 states that, the facility is supplied with normal and standby emergency power to provide for the required functioning of the protection systems. In the event of loss of normal power, emergency alternating current (ac) power is supplied by six

emergency diesel generators, as described in DCPD FSAR Update Chapter 8. Only four diesel generators are required to supply the power requirements with one unit in an accident situation and to bring the other unit to the shutdown condition from full power. The instrumentation and controls portions of the protection systems are supplied initially from the station batteries and subsequently from the emergency diesel generators. A single failure of any one component will not prevent the required functioning of protection systems.

DCPD FSAR Update Section 3.1.8.3, Criterion 39, "Emergency Power for Engineered Safety Features (Category A)," was the original licensing basis of the plant. Revision 20 of the FSAR Update states that Criterion 39, 1967 is no longer part of the DCPD license basis and has been replaced by General Design Criteria (GDC)-17, 1971 "Electric power systems," and GDC-18, 1971, "Inspection and testing of electric power systems."

The following discussion from Section 3.1.8.3.1 of the DCPD FSAR Update Section 3.1.8.3.1 (Revision 20) describes how the DCPD design conforms to GDC 17:

An onsite electric power system and an offsite electric power system shall be provided to permit functioning of structures, systems, and components important to safety. The safety function for each system (assuming the other system is not functioning) shall be to provide sufficient capacity and capability to assure that (1) specified acceptable fuel design limits and design conditions of the reactor coolant pressure boundary are not exceeded as a result of anticipated operational occurrences and (2) the core is cooled and containment integrity and other vital functions are maintained in the event of postulated accidents.

The onsite electric power supplies, including the batteries, and the onsite electric distribution system, shall have sufficient independence, redundancy, and testability to perform their safety functions assuming a single failure.

Electric power from the transmission network to the onsite electric distribution system shall be supplied by two physically independent circuits (not necessarily on separate rights of way) designed and located so as to minimize to the extent practical the likelihood of their simultaneous failure under operating and postulated accident and environmental conditions. A switchyard common to both circuits is acceptable. Each of these circuits shall be designed to be available in sufficient time following a loss of all onsite alternating current power supplies and the other offsite electric power circuit, to assure that specified acceptable fuel design limits and design conditions of the reactor coolant pressure boundary are not exceeded. One of these circuits shall be designed to be available within a few seconds following a loss-of-coolant accident to assure that core cooling, containment integrity, and other vital safety functions are maintained.

The following discussion from DCPD FSAR Update Section 3.1.8.3.2 (Revision 20) describes how the DCPD design conforms to GDC 18.

Electric power systems important to safety shall be designed to permit appropriate periodic inspection and testing of important areas and features, such

as wiring, insulation, connections, and switchboards, to assess the continuity of the systems and the condition of their components. The systems shall be designed with a capability to test periodically (1) the operability and functional performance of the components of the systems, such as onsite power sources, relays, switches, and buses, and (2) the operability of the systems as a whole and, under conditions as close to design as practical, the full operation sequence that brings the systems into operation, including operation of applicable portions of the protection system, and the transfer of power among the nuclear power unit, the offsite power system, and the onsite power system.

In Section 50.36, "Technical specifications," of Title 10 of the *Code of Federal Regulations* (10 CFR), the NRC established its regulatory requirements related to the content of Technical Specifications (TSs). 10 CFR 50.36(c)(3) requires that TSs include Surveillance Requirements (SRs), which are requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the limiting conditions for operation will be met.

In addition to the TS change described above, the licensee determined that an FSAR Update change related to an exception to RG 1.9 conformance required NRC approval under 10 CFR 50.59. PGE's March 28, 2011, license amendment request, submitted in accordance with 10 CFR 50.90, includes this proposed FSAR Update change. Safety Guide 9, "Selection of Diesel Generator Set Capacity for Standby Power Supplies," dated March 10, 1971 (Safety Guide 9 is also known as RG 1.9, Revision 0), describes an acceptable basis for the selection of diesel generator (DG) sets of sufficient capacity and margin to implement GDC 17. FSAR Update Section 8.1.4.3 states that DCPD is committed to RG 1.9, Revision 0. Regulatory Position C.4 of this Safety Guide states the following:

Each diesel generator set should be capable of starting and accelerating to rated speed, in the required sequence, all the needed engineered safety feature and emergency shutdown loads. At no time during the loading sequence should the frequency and voltage decrease to less than 95 percent of nominal and 75 percent of nominal, respectively. During recovery from transients caused by step load increases or resulting from the disconnection of the largest single load, the speed of the diesel generator set should not exceed 75 percent of the difference between nominal speed and the overspeed trip set point or 115 percent of nominal whichever is lower. Voltage should be restored to within 10 percent of nominal and frequency should be restored to within 2 percent of nominal in less than 40 percent of each load sequence time interval.

3.0 TECHNICAL EVALUATION

The proposed license amendment includes changes to TS SRs 3.8.1.2, 3.8.1.7, 3.8.1.12, 3.8.1.15, and 3.8.1.20 for the DGs, specifically, DGs 1-1, 1-2, 1-3, 2-1, 2-2, and 2-3. By letter dated February 5, 2012, the licensee provided supplemental information that provided detailed discussions on the impact of proposed changes on the capability of the DGs to perform their intended design functions.

The licensee stated that the proposed changes are consistent with NRC-approved TSTF-163, Revision 2, "Minimum vs. Steady State Voltage and Frequency." TSTF-163, Revision 2 (ADAMS Accession No. ML040500733), was submitted to the NRC on April 22, 1998.

The proposed changes would also revise the FSAR Update to identify an exception to RG 1.9, Revision 0, Regulatory Position C.4.

According to the FSAR Update, the DCPD electrical power distribution system ac sources consist of offsite power sources (230 kiloVolt (kV) and 500 kV switchyards), and the onsite standby power sources (three DGs for each unit). The design of the ac electrical power system provides independence and redundancy to ensure an available source of power to the engineered safety features (ESFs).

In its letter dated March 28, 2011, the licensee stated, in part, the following:

The onsite Class 1E AC Distribution System for each unit is divided into three load groups so that the loss of any one group does not prevent the minimum safety functions from being performed. Each load group has connections to two offsite power sources and a single onsite standby DG.

The onsite standby power source for each 4.16 kV ESF bus is a dedicated DG. For Unit 1, DGs 1-1, 1-2, and 1-3 are dedicated to ESF buses H, G, and F, respectively. For Unit 2, DGs 2-1, 2-2, and 2-3 are dedicated to ESF buses G, H, and F, respectively. A DG starts automatically on a safety injection (SI) signal (e.g., low pressurizer pressure or high containment pressure signals), undervoltage on the offsite standby startup source, or on an ESF bus degraded voltage or undervoltage signal. After the DG has started, it will automatically tie to its respective bus after offsite power is tripped as a consequence of ESF bus undervoltage or degraded voltage, independent of or coincident with an SI signal. The DGs will also start and operate in the standby mode without tying to the ESF bus on an SI signal alone. Following the trip of offsite power, an undervoltage signal strips nonpermanent loads from the ESF bus. When the DG is tied to the ESF bus, loads are then sequentially connected to their respective ESF bus by the load-sequencing timers (ESF timers). The sequencing logic controls the permissive and starting signals to motor breakers to prevent overloading the DG. Each ESF component is provided with its own load-sequencing timer.

In the event of a loss of the preferred 230 kV offsite power source concurrent with a SI signal, the ESF electrical loads are automatically connected to the DGs in sufficient time to provide for safe reactor shutdown and to mitigate the consequences of a design-basis accident (DBA) such as a loss-of-coolant accident (LOCA).

Certain large required ESF loads are returned to service in a predetermined sequence in order to prevent overloading the DG in the process. All loads needed to recover the unit or maintain it in a safe condition are returned to service via the ESF timers (ESF timer sequence and intervals are described in DCPD FSAR Update Section 8.3 "Onsite Power Systems"). Each individual timer connects a single ESF component.

The DGs have a net continuous electrical output rating of 2600 kilowatt (kW) at 0.8 power factor (PF) (continuous rating), and are rated for 2750 kW at 0.8 PF for up to 2000 hours of operation (2000-hour rating). Short-term ratings of the DGs are 2860 kW at 0.8 PF (2 hours per 24-hour rating), 3000 kW at 0.8 PF (2 hours per year rating), and 3250 kW at 0.8 PF (30-minute per 24-hour rating). During the starting sequence for the safeguard loads, these machines can also carry short-time overloads.

Each DG unit consists of a self-contained diesel engine directly connected to an alternating current generator, and the separate accessories needed for proper operation, all mounted on a common structural steel skid-type base. Mechanical power is provided by an 18 cylinder, four-cycle, 3630 horsepower at 900 rpm [revolutions per minute], turbo-charged and after-cooled, heavy-duty, stationary-type diesel engine.

The generator is rated at 3250 kV amperes, 0.8 PF, 4160 Volts (V), and 60 hertz (Hz). The exciter is a static series, boost-type exciter controlled by a static solid-state voltage regulator.

3.1 Request to Revise SR 3.8.1.2

Current SR 3.8.1.2 states, in part:

Verify each DG starts from standby conditions and achieves speed ≥ 900 rpm, steady state voltage ≥ 3785 V and ≤ 4400 V, and frequency ≥ 58.8 Hz and ≤ 61.2 Hz.

Revised SR 3.8.1.2 would state, in part:

Verify each DG starts from standby conditions and achieves steady state voltage ≥ 3785 V and ≤ 4400 V, and frequency ≥ 58.8 Hz and ≤ 61.2 Hz.

The licensee proposed revising the current SR 3.8.1.2 to remove the requirement to achieve a speed of 900 rpm. The NRC staff concludes that it is not necessary to include both a minimum speed requirement and a minimum frequency requirement in SR 3.8.1.2 since the minimum frequency requirement of 58.8 Hz in SR 3.8.1.2 ensures the DG is operating at a minimum of 882 rpm, which is within 2 percent of the DG nominal design speed of 900 rpm. This is consistent with the 2 percent tolerance of the 60 Hz nominal frequency which is derived from the guideline given in RG 1.9, Revision 0, Regulatory Position C.4. Based on this information, the staff concludes that the requirements of 10 CFR 50.36(c)(3) will continue to be met and, therefore, the proposed change to SR 3.8.1.2 is acceptable.

3.2 Request to Adopt TSTF-163, Revision 2

Consistent with the approved TSTF-163, Revision 2, the licensee proposed to revise the acceptance criteria of SRs 3.8.1.7, 3.8.1.12, 3.8.1.15, and 3.8.1.20. The Standard TS changes in TSTF-163, Revision 2, were made to provide better agreement with the performance of a diesel generator that is started without subsequent connection to an emergency bus (unloaded diesel generator start). The acceptance criteria for the emergency DG start tests will be modified by specifying the minimum voltage and frequency to be achieved during startup, and assigning the

currently listed voltage and frequency ranges as values to be achieved once "steady state" is reached.

Current SR 3.8.1.7.a states, in part:

in ≤ 10 seconds, speed ≥ 900 rpm; and

Revised SR 3.8.1.7.a would state, in part:

In ≤ 10 seconds, voltage ≥ 3785 V and frequency ≥ 58.8 Hz; and

Current SR 3.8.1.7.b states, in part:

in ≤ 13 seconds, voltage ≥ 3785 V and ≤ 4400 V, and frequency ≥ 58.8 Hz and ≤ 61.2 Hz.

Revised SR 3.8.1.7.b would state, in part:

Steady state voltage ≥ 3785 V and ≤ 4400 V, and frequency ≥ 58.8 Hz and ≤ 61.2 Hz.

Current SR 3.8.1.12.a states, in part:

In ≤ 13 seconds after auto-start and during tests, achieves voltage ≥ 3785 V and ≤ 4400 V;

Revised SR 3.8.1.12.a would state, in part:

In ≤ 10 seconds after auto-start and during tests, achieves voltage ≥ 3785 V and frequency ≥ 58.8 Hz;

Current SR 3.8.1.12.b states, in part:

In ≤ 13 seconds after auto-start and during tests, achieves frequency ≥ 58.8 Hz and ≤ 61.2 Hz;

Revised SR 3.8.1.12.b would state, in part:

Achieves steady state voltage ≥ 3785 V and ≤ 4400 V, and frequency ≥ 58.8 Hz and ≤ 61.2 Hz;

Current SR 3.8.1.15.a states, in part:

in ≤ 10 seconds, speed ≥ 900 rpm; and

Revised SR 3.8.1.15.a would state, in part:

In ≤ 10 seconds, voltage ≥ 3785 V and frequency ≥ 58.8 Hz; and

Current SR 3.8.1.15.b states, in part:

In ≤ 13 seconds, voltage ≥ 3785 V, and ≤ 4400 V, and frequency ≥ 58.8 Hz and ≤ 61.2 Hz.

Revised SR 3.8.1.15.b would state, in part:

Steady state voltage ≥ 3785 V and ≤ 4400 V, and frequency ≥ 58.8 Hz and ≤ 61.2 Hz.

Current SR 3.8.1.20.a states, in part:

in ≤ 10 seconds, speed ≥ 900 rpm; and

Revised SR 3.8.1.20.a would state, in part:

In ≤ 10 seconds, voltage ≥ 3785 V and frequency ≥ 58.8 Hz; and

Current SR 3.8.1.20.b states, in part:

In ≤ 13 seconds, voltage ≥ 3785 V, and ≤ 4400 V, and frequency ≥ 58.8 Hz and ≤ 61.2 Hz.

Revised SR 3.8.1.20.b would state, in part:

Steady state voltage ≥ 3785 V and ≤ 4400 V, and frequency ≥ 58.8 Hz and ≤ 61.2 Hz.

The "Notes" associated with the above SRs are not changed.

The changes that the licensee has proposed maintain the original voltage and frequency range for steady state conditions while in the transient region, only minimum values are proposed in the allowable range. SRs 3.8.1.7, 3.8.1.12, 3.8.1.15, and 3.8.1.20 are unloaded DG start tests that do not result in connecting the DG to the emergency bus. It is understood that for unloaded DG starts, a momentary voltage or frequency overshoot (and/or a subsequent undershoot) can occur because no loads are being connected to the DG. A loaded DG start tends to minimize the overshoot, compared to an unloaded DG start where the overshoot might momentarily exceed the specified limits within the first 10 seconds of startup. This is a condition inherent in the design and physical capabilities of the DG governor as it seeks to control DG speed following fast starts when the DG is not connected to an emergency bus. Applying loads to a DG tends to minimize the overshoot, whereas the unloaded overshoot might momentarily exceed the specified limits within the first 10 or 13 seconds of startup. According to the licensee, the voltage and frequency excursions do not affect the permissive function for closure of the emergency DG output breaker, since the permissive function is primarily dependent on minimum conditions being achieved regardless of any overshoot or subsequent momentary undershoot.

The NRC staff concludes that the voltage and frequency range currently specified in the subject surveillances are more appropriate for steady state limits than transient limits, and that the

revised TS SRs, as proposed, will continue to verify the capability of the emergency DGs to provide power at a voltage and frequency adequate to start and operate the safety loads.

Furthermore, the licensee stated in its letter dated March 28, 2011, that it will revise the TS Bases for SRs 3.8.1.2 and 3.8.1.7 to add the following statement:

In addition to the SR requirements, the time for the DG to reach steady state operation is periodically monitored and the trend evaluated to identify degradation of governor and voltage regulator performance.

The NRC staff concludes that periodically monitoring, trending, and evaluating the time for the DG to reach steady state operation provides reasonable assurance that degradation of the governor and voltage regulatory performance will be identified and the appropriate corrective actions taken when necessary. In accordance with FSAR Update Section 16.1, "Technical Specifications and Equipment Control Guidelines," the TS Bases are included by reference in Section 16.1 of the FSAR Update and are part of the Diablo Canyon "licensing bases." In addition, to revise or remove this TS Bases statement after implementation of this license amendment PG&E would have to process the change per TS 5.5.14 and determine if the change requires NRC approval pursuant to 10 CFR 50.59. The licensee's March 28, 2011, letter (submitted under penalty of perjury that the information is true and correct) states that the TS Bases will be updated to include a discussion to periodically monitor, trend, and evaluate the time for the DG to reach steady state operation. The NRC staff concludes the licensee's statement that changes will be made to the TS Bases in accordance with the licensee's letter dated March 28, 2011, and the regulatory controls associated with the TS Bases after the information is incorporated into the TS Bases, are acceptable.

The NRC staff evaluated the licensee's request to revise TS SRs 3.8.1.7, 3.8.1.12, 3.8.1.15, and 3.8.1.20 for the DG start time for DCCP. In summary, the staff concludes that the proposed changes are consistent with TSTF-163, Revision 2, for eliminating the upper end of allowable voltage range during transient conditions experienced during emergency DG starts. The staff also concludes that the provisions of the SRs, as amended, will continue to ensure that proper transient and steady state voltage and frequency are attained consistent with the recommendations of RG 1.9, Revision 3. The staff's conclusion is based on the following:

- the minimum frequency requirement of 58.8 Hz ensures the DG is operating at a minimum of 882 rpm, which is within 2 percent of the DG nominal design speed of 900 rpm (the 2 percent tolerance of the 60 Hz nominal frequency is derived from the guideline given in RG 1.9, Revision 0, Regulatory Position C.4),
- the elimination of upper end of allowable voltage for transient conditions during an unloaded emergency DG start does not degrade safety margins,
- the steady state voltage requirements are not impacted and compliance with the SRs will continue to demonstrate the performance capabilities of each emergency DG to maintain bus voltages within acceptable limits for achieving safe shutdown, and

- the licensee will periodically monitor, trend, and evaluate the time for the DG to reach steady state operation to identify degradation of governor and voltage regulator performance.

Based on the above, the NRC staff concludes that the proposed TS SR changes are consistent with TSTF-163, Revision 2, and will not impact the licensee's ability to continue to comply with the requirements of 10 CFR 50.36(c)(3) and, therefore, are acceptable. These changes are also consistent with and have been incorporated into NUREG 1431, Standard Technical Specifications Westinghouse Plants, Revision 3.0.

3.3 Request for Exception to Regulatory Guide 1.9, Revision 0

The licensee has requested an exception to RG 1.9, Revision 0, Regulatory Position C.4, in DCPD FSAR Update Section 8.3.1.1.13.1 for the motor-driven auxiliary feedwater (MDAFW) pump loading for DGs 1-1, 1-3, 2-2, and 2-3, and to remove reference to RG 1.9, Revision 2. The exception would permit restoration of frequency for DGs 1-1, 1-3, 2-2, and 2-3, to within 2 percent of nominal in less than 60 percent of the load sequence time interval (2.4 seconds for the DCPD 4-second nominal load sequence time interval) for MDAFW pump loading. RG 1.9, Revision 0, Regulatory Position C.4 specifies that during the DG loading sequence the frequency should be restored to within 2 percent of nominal in less than 40 percent of each load sequence time interval (1.6 seconds for the DCPD 4-second nominal load sequence time interval). Permitting restoration of frequency to within 2 percent of nominal in less than 60 percent of the load sequence time interval is consistent with the guidance in RG 1.9, Revision 2, dated December 1979, RG 1.9, Revision 3 dated July 1993, and RG 1.9, Revision 4, dated March 2007.

The licensee also requested to revise FSAR Update Section 8.1.4.3 for RG 1.9, Revision 0, to remove reference to RG 1.9, Revision 2, and RG 1.9, Revision 3, and to add that DCPD is committed to Safety Guide 9 (RG 1.9, Revision 0) with the exception of Regulatory Position C.4.

The RG 1.9, Revision 0, Regulatory Position C.4 guideline ensures that the diesel is capable of maintaining the 4 kV system conditions (voltage and frequency) necessary for starting and accelerating, to rated speed, all the needed ESF loads. This is accomplished by specifying the DGs be capable of restoring frequency and voltage within 40 percent of the timer interval to the subsequent load.

As stated above, the DCPD DGs were designed to Safety Guide 9, "Selection of Diesel Generator Set Capacity for Standby Power Supplies," dated March 10, 1971 (also known as RG 1.9, Revision 0). Vendor facility shop tests were conducted by ALCO Engine Division of White Industrial Power, Inc., for the five original DGs (1-1, 1-2, 1-3, 2-1, and 2-2) to verify the unit design capabilities, their reliability, and their conformance to specification requirements as summarized in FSAR Update Table 8.3-8. Extensive onsite preoperational qualification testing was performed for the original five DGs during DCPD startup as summarized in FSAR Update Table 8.3-9. Separate testing was performed on DG 2-3 to support qualification.

Preoperational testing performed during July 1981 identified an exception to RG 1.9, Revision 0, Regulatory Position C.4. The results of the July 1981 testing determined that the frequency dropped to as low as 56.3 Hz during the last load block. This was less than the guideline value

of 57 Hz (95 percent of the nominal frequency of 60 Hz) specified by RG 1.9, Revision 0, Regulatory Position C.4 that at no time during the load sequence should the frequency decrease to less than 95 percent of nominal. The licensee submitted a letter to the NRC on July 31, 1981, (ADAMS Legacy Accession No. 8108040456) summarizing the results of the testing. The NRC responded to the licensee's letter dated July 31, 1981, in a letter dated September 14, 1981 (ADAMS Legacy Accession No. 8109240609). The letter approved a frequency exception to RG 1.9, Revision 0, Regulatory Position C.4, and stated, in part:

We have reviewed your submittal dated July 31, 1981 in regard to diesel generator load tests and find that the tests demonstrate compliance with the guidelines delineated in Regulatory Guide 1.9 except for one minor frequency variation. The frequency variation was slightly below the 95% frequency lower recommended limit for less than one second; the Diesel Generator showed strong recovery capability demonstrated by return to within the frequency limit within less than 0.035 seconds; this meets the objective of Regulatory Guide 1.9, and is, therefore acceptable.

During DG testing performed in the fifth refueling outage for Units 1 and 2 (1R5 and 2R5) and the seventh refueling outage for Units 1 and 2 (1R7 and 2R7) to support removal of the kilowatt sensing (KWS) relay, the licensee identified that the DG frequency recovery did not meet the RG 1.9, Revision 0, Position C.4 guideline to restore frequency to within 2 percent within 40 percent of the load sequence time interval for the MDAFW pump loads. In addition, in 1997, the licensee noted that its calculations for worst case DG load-sequence timer tolerances showed that the frequency drop and frequency recovery did not meet RG 1.9, Revision 0, guidelines. To address the exceptions to RG 1.9, Revision 0, the licensee revised FSAR Update Section 8.3.1.1.13.1 under 10 CFR 50.59 to identify the exceptions to RG 1.9, Revision 0, based on RG 1.9, Revision 2, that allows frequency recovery within 60 percent of the load block interval and allows "a greater percentage of the time interval may be used if it can be justified by analysis." The licensee later realized that these exceptions to RG 1.9, Revision 0, based on RG 1.9, Revision 2, should have received prior NRC approval and that they were incorrectly included through a change made to the FSAR Update under 10 CFR 50.59. The licensee is seeking the NRC's approval of these exceptions to RG 1.9, Revision 0, in this license amendment request.

As part of its review, the staff requested the licensee to provide a summary of the test data from the 1R5, 2R5, 1R7, and 2R7 outages that demonstrated that there is adequate margin in the 4-second load timer interval to ensure that there is no overlapping of loads, that the DG frequency consistently stabilizes within a nominal 2.2 to 2.3 seconds, and that the DG is capable of starting and accelerating subsequent ESF loads. The licensee responded to the staff's request for additional information in letter dated February 5, 2012. Based on its review of the information provided by the licensee, the staff found that the licensee demonstrated that there is adequate margin in the 4-second timer interval to ensure that there is no overlapping of loads, that the DG frequency consistently stabilizes within a nominal 2.2 to 2.3 seconds, and that the DG is capable of starting and accelerating subsequent ESF loads. The staff also concludes that the licensee's data showed that when MDAFW frequency recovery above 98 percent of 60 Hz was greater than 1.6 seconds (40 percent of the 4-second load timer interval), following breaker closure of the MDAFW pumps, there was no impact on the ability of DG 1-1

and 1-2 to start and accelerate the containment spray pump. For DG 1-3 and 2-3, the MDAFW pump is the last ESF sequenced load.

For the requested exemption to RG 1.9, Revision 0, to allow 2.4 seconds of the frequency recovery for MDAFW pump loading for DGs 1-1, 1-3, 2-2, and 2-3, the licensee has shown that the step load change frequency transient recovery time for the MDAFW pumps does not impact the capability of the already running ESF motors. The NRC staff's conclusion considers that induction motors are designed to operate with a 5 percent frequency variation according to the National Electrical Manufacturers Association Standard MG 1, which corresponds with the RG 1.9, Revision 0, recommended minimum frequency (i.e., at no time during the loading sequence should the frequency decrease to less than 95 percent of nominal). According to the licensee, this includes continuous duty motors (e.g., motor-driven pumps) and motor-operated valves. Induction motor operation at less than nominal frequency can impact the performance capability by increasing the output torque and decreasing the synchronous speed (i.e., rpm).

In its letter dated February 5, 2012, the licensee demonstrated that all large loads are strictly dependant on sequencer permissive signals and that the loading sequence is not affected by process signals (e.g., system pressure, temperature, level, etc.) which could lead to overlapping or simultaneous actuation of loads.

Based on its review, the NRC staff concludes that it is acceptable for MDAFW pump loading on DGs 1-1, 1-3, 2-2, and 2-3 to be restored to within 2 percent of nominal in less than 60 percent of the load sequence time interval (2.4 seconds) since the DGs will still meet the intent of RG 1.9, Revision 0, Regulatory Position C.4 and continue to be capable of performing their intended design functions. In its March 28, 2011, license amendment request, the licensee committed to revise the DCPD FSAR Update to reflect these exceptions upon implementation of this license amendment.

Based on the above, the NRC staff concludes that the proposed revisions to the DCPD TSs and the requested exceptions to RG 1.9, Revision 0, Regulatory Position C.4, provide reasonable assurance of the continued availability of the required power to shut down the reactor and to maintain the reactor in a safe condition after an anticipated operational occurrence or a postulated design-basis accident. The staff also concludes that the proposed TS changes are in accordance with 10 CFR 50.36(c)(3) and consistent with the intent of GDCs 17 and 18. Therefore, the staff concludes the proposed changes are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the California State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The

Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding published in the *Federal Register* on May 31, 2011 (76 FR 31375). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Matthew McConnell

Date: March 29, 2012

J. Conway

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A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,

/RA/

Joseph M. Sebrosky, Senior Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-275 and 50-323

Enclosures:

1. Amendment No. 211 to DPR-80
2. Amendment No. 213 to DPR-82
3. Safety Evaluation

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