UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
245 PEACHTREE CENTER AVENUE NE, SUITE 1200
ATLANTA, GEORGIA 30303-1257

March 14, 2012

Mr. Dennis R. Madison
Vice President
Southern Nuclear Operating Company, Inc.
Edwin I. Hatch Nuclear Plant
11028 Hatch Parkway North
Baxley, GA 31513

SUBJECT: NOTIFICATION OF EDWIN I. HATCH NUCLEAR PLANT - NRC COMPONENT DESIGN BASES INSPECTION - INSPECTION REPORT 05000321/2012008 AND 05000366/2012008

Dear Mr. Madison:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a component design bases inspection at your Edwin I. Hatch Nuclear Plant during the weeks of June 25 – June 29, July 9 - July 13, and July 23 – July 27, 2012. Patrick Higgins, a Senior Reactor Inspector from the NRC's Region II Office, will lead the inspection team. This inspection will be conducted in accordance with the baseline inspection procedure, Procedure 71111.21, Component Design Bases Inspection, issued December 6, 2010.

The inspection will evaluate the capability of risk significant/low margin components to function as designed and to support proper system operation. The inspection will also include a review of selected operator actions, operating experience, and modifications.

During a telephone conversation on March 12, 2012, Mr. Higgins confirmed with Mr. Tipps of your staff, arrangements for an information- gathering site visit, and the three-week onsite inspection. The schedule is as follows:

- Information-gathering visit: Week of June 4 - 8, 2012
- Onsite weeks: June 25 – June 29, July 9 - July 13, and July 23 – July 27, 2012

The purpose of the information-gathering visit is to meet with members of your staff to identify risk-significant components and operator actions. Information and documentation needed to support the inspection will also be identified. Mr. Rudy Bernhard, a Region II Senior Reactor Analyst, will accompany Mr. Higgins during the information-gathering visit to review probabilistic risk assessment data and identify risk significant components that will be examined during the inspection.

The Enclosure lists documents that will be needed prior to the information-gathering visit. Please provide the referenced information to the Region II office by May 21, 2012. Contact Mr. Higgins with any questions concerning the requested information. The inspectors will try to

minimize your administrative burden by specifically identifying only those documents required for inspection preparation.

Additional documents will be requested during the information-gathering visit. The additional information will need to be available to the team in the Region II office prior to the inspection team's preparation week of June 18, 2012. Mr. Higgins will also discuss the following inspection support administrative details: availability of knowledgeable plant engineering and licensing personnel to serve as points of contact during the inspection; method of tracking inspector requests during the inspection; licensee computer access; working space; arrangements for site access; and other applicable information.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its Enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact Mr. Higgins at 404-997-4615 or me at 404-997-4530.

Sincerely,

/RA/

Rebecca Nease, Chief
Engineering Branch 1
Division of Reactor Safety

Docket Nos. 50-321, 50-366
License Nos. DPR-57, NPF-5

Enclosure:
Information Request for Edwin I. Hatch Nuclear Plant –
Component Design Bases Inspection

cc w/encl:
(See pages 3 and 4)

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Sincerely,

/RA/
Rebecca Nease, Chief
Engineering Branch 1
Division of Reactor Safety

Docket Nos. 50-321, 50-366
License Nos. DPR-57, NPF-5

Enclosure:
Information Request for Edwin I. Hatch Nuclear Plant –
Component Design Bases Inspection

cc w/encl:
(See pages 3 and 4)

Distribution w/encl:
F. Ehrhardt, RII, DRP
M. Miller, RII, DRP
RIDSNRRDIRS
PUBLIC
RidsNrrPMHatch Resource

X PUBLICLY AVAILABLE ☐ NON-PUBLICLY AVAILABLE ☐ SENSITIVE **X** NON-SENSITIVE
ADAMS: **X** Yes ACCESSION NUMBER: ML12074A239 **X** SUNSI REVIEW COMPLETE **X** FORM 665 ATTACHED

OFFICE	RII:DRS	RII:DRS					
SIGNATURE	/RA/	/RA S. Sandal for/					
NAME	P. Higgins	R. Nease					
DATE	3/ 13 /2012	3/ 14 /2012					
E-MAIL COPY	**YES** NO	**YES** NO					

OFFICIAL RECORD COPY DOCUMENT NAME: S:\DRS\ENG BRANCH 1\BRANCH INSPECTION FILES\2011-2012-2013 CYCLE EB1 INSPECTION FOLDERS FOR ALL SITES\HATCH\HATCH CDBI 2012008 NOTIFICATION LETTER (PH).DOCX

cc w/encl:
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Electronic Mail Distribution

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SVP & General Counsel-Ops & SNC
Southern Nuclear Operating Company, Inc.
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D. G. Bost
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Senior Resident Inspector
U.S. Nuclear Regulatory Commission
Edwin I. Hatch Nuclear Plant
U.S. NRC
11030 Hatch Parkway N
Baxley, GA 30334

cc w/encl: (con't on page 4)

cc w/encl (con't from page 3)

Mr. Ken Rosanski
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James A. Capp
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Environmental Protection Division
Georgia Department of Natural Resources
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Mr. Steven M. Jackson
Senior Engineer - Power Supply
Municipal Electric Authority of Georgia
Electronic Mail Distribution

INFORMATION REQUEST FOR EDWIN I. HATCH NUCLEAR PLANT
COMPONENT DESIGN BASES INSPECTION

Please provide the information electronically in ".pdf" files, Excel, or other searchable format on CDROM (or FTP site, Sharepoint, etc.) The CDROM (or website) should be indexed and hyperlinked to facilitate ease of use.

1. From your most-recent Probabilistic Safety Analysis (PSA) excluding external events and fires:

    a. Two risk rankings of components from your site-specific PSA

        i) one sorted by Risk Achievement Worth (RAW)
        ii) the other sorted by Birnbaum Importance

    b. A list of the top 500 cutsets

2. From your most-recent Probabilistic Safety Analysis (PSA) including external events and fires:

    a. Two risk rankings of components from your site-specific PSA

        i) one sorted by Risk Achievement Worth (RAW)
        ii) the other sorted by Birnbaum Importance

    b. A list of the top 500 cutsets

3. Risk ranking of operator actions from your site specific PSA sorted by RAW. Provide human reliability worksheets for these items.

4. List of time critical operator actions with a brief description of each action.

5. List of Emergency and Abnormal Operating Procedures revised (significant) since April 1, 2010 with a brief description of each revision.

6. List of components with low design margins (e.g., pumps closest to the design limit for flow or pressure, diesel generator close to design required output, heat exchangers close to rated design heat removal, motor operated valve risk-margin rankings, etc.) and associated evaluations, or calculations.

7. List of station operating experience evaluations/reviews performed and documented in the station's corrective action program for industry events and safety related equipment failures/vulnerabilities [as communicated by NRC generic communications, industry communications, 10 CFR part 21 Notifications, etc.] since April 1, 2010.

8. List and brief description of safety related SSC design modifications implemented since April 1, 2010.

9. List and brief description of common-cause component failures that have occurred since April 1, 2010.

10. List and brief description of operability evaluations completed since April 1, 2010.

11. List of equipment on the site's Station Equipment Reliability Issues List, including a description of the reason(s) why each component is on that list and summaries (if available) of your plans to address the issue(s).

12. List and brief description of equipment currently in degraded or nonconforming status as described in RIS 05-020.

13. List and reason for equipment classified in maintenance rule (a)(1) status since April 1, 2010 to present.

14. Copies of System Descriptions (or the like design basis documents) for Safety-Related Systems.

15. Copy of UFSAR(s).

16. Copy of Technical Specification(s).

17. Copy of Technical Specifications Bases.

18. Copy of Technical Requirements Manual(s).

19. List and brief description of Root Cause Evaluations that have been performed since April 1, 2010.

20. In-service Testing Program Procedure(s).

21. Corrective Action Program Procedure(s).

22. One line diagram of electrical plant (electronic and full size – hard copy week of June 4).

23. Index and legend for electrical plant one-line diagrams.

24. Primary AC calculation(s) for safety-related buses.

25. Primary DC calculation(s) for safety-related buses.

26. PI&D's for ECCS systems (electronic and 1/2 size – hard copy week of June 4).

27. Index and Legend for PI&Ds.

28. Copy of design bases documents for ECCS systems.

29. Copy of Operability determination procedure(s).

30. Copies of condition reports associated with findings from previous CDBI (if applicable).

31. Index (procedure number, titles, and current revision) of station Emergency Operating Procedures (EOPs), Abnormal Operating Procedures (AOPs), and Annunciator Response Procedures (ARPs).

32. Contact information for a person to discuss PRA information prior to the information-gathering trip (name, title, phone number, and e-mail address).

LIST OF ACRONYMS

AC	Alternating Current
AOP	Abnormal Operating Procedure
ARP	Annunciator Response Procedure
CDBI	Component Design Basis Inspection
DC	Direct Current
ECCS	Emergency Core Cooling System
EOP	Emergency Operating Procedure
FTP	File Transfer Protocol
NRC	Nuclear Regulatory Commission
OE	Operating Experience
P&ID	Piping and Instrumentation Diagram
PRA	Probabilistic Risk Assessment
PSA	Probabilistic Safety Assessment
RAW	Risk Achievement Worth
RIS	Regulatory Issue Summary
SSC	Structure, System or Component
UFSAR	Updated Final Safety Analysis Report