



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

March 2, 2012

Vice President, Operations  
Entergy Nuclear Operations, Inc.  
Palisades Nuclear Plant  
27780 Blue Star Memorial Highway  
Covert, MI 49043-9530

SUBJECT: PALISADES NUCLEAR PLANT – CORRECTION TO AMENDMENT NO. 244  
REGARDING TECHNICAL SPECIFICATION CHANGE TO CALCULATED PEAK  
CONTAINMENT INTERNAL PRESSURE (TAC NO. ME6875)

Dear Sir or Madam:

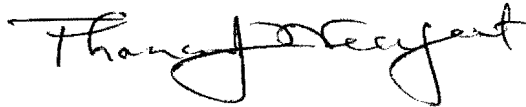
On January 19, 2012, the U.S. Nuclear Regulatory Commission (NRC) issued Amendment No. 244 to Renewed Facility Operating License No. DPR-20 for the Palisades Nuclear Plant. This amendment consisted of a change to the Technical Specifications (TSs) in response to your application dated August 16, 2011, as supplemented by letter dated October 6, 2011. The amendment revised TS Section 5.5.14, "Containment Leak Rate Testing Program" to increase the value of the calculated peak containment internal pressure.

Following issuance of the amendments, the NRC staff was informed by the Palisades licensee that the second line of Section 5.5.14.c of TS page 5.0-18, (i.e., "0.1% of containment air weight per day.") was inadvertently removed from the TS page. The NRC staff found that the error was entirely administrative in nature, was inadvertently introduced, and was not the subject of the amendment or the associated notice to the public. Thus, according to the NRC's policy established by SECY-96-238 (Agencywide Documents Access and Management System Accession No. 9611250030 (Legacy Library)), this error can be corrected by a letter. Enclosed, please find the replacement TS page.

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This correction does not change any of the conclusions in the safety evaluation associated with the amendment. We regret any inconvenience this may have caused. If you have any questions regarding this matter, please call me at (301) 415-4037.

Sincerely,

A handwritten signature in black ink, appearing to read "Thomas J. Wengert". The signature is written in a cursive style with a large, prominent initial "T".

Thomas J. Wengert, Senior Project Manager  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-255

Enclosure: Corrected TS Page

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ENCLOSURE

PALISADES NUCLEAR PLANT

DOCKET NO. 50-255

REPLACEMENT TECHNICAL SPECIFICATION PAGE  
5.0-18 FOR AMENDMENT NO. 244

5.5 Programs and Manuals

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5.5.13 Safety Functions Determination Program (SFDP) (continued)

- c. A required system redundant to support system(s) for the supported systems (a) and (b) above is also inoperable.

The SFDP identifies where a loss of safety function exists. If a loss of safety function is determined to exist by this program, the appropriate Conditions and Required Actions of the LCO in which the loss of safety function exists are required to be entered. When a loss of safety function is caused by the inoperability of a single Technical Specification support system, the appropriate Conditions and Required Actions to enter are those of the support system.

5.5.14 Containment Leak Rate Testing Program

- a. A program shall establish the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines of Regulatory Guide 1.163, "Performance-Based Containment Leakage-Test Program," dated September 1995, except that the next Type A test performed after the May 3, 2001, Type A test shall be performed no later than August 3, 2012, as modified by the following exceptions:

1. Leakage rate testing is not necessary after opening the Emergency Escape Air Lock doors for post-test restoration or post-test adjustment of the air lock door seals. However, a seal contact check shall be performed instead.

Emergency Escape Airlock door opening, solely for the purpose of strongback removal and performance of the seal contact check, does not necessitate additional pressure testing.

2. Leakage rate testing at  $P_a$  is not necessary after adjustment of the Personnel Air Lock door seals. However, a between-the-seals test shall be performed at  $\geq 10$  psig instead.
  3. Leakage rate testing frequency for the Containment 4 inch purge exhaust valves, the 8 inch purge exhaust valves, and the 12 inch air room supply valves may be extended up to 60 months based on component performance.
- b. The calculated peak containment internal pressure for the design basis loss of coolant accident,  $P_a$ , is 54.2 psig. The containment design pressure is 55 psig.
- c. The maximum allowable containment leakage rate,  $L_a$ , at  $P_a$ , shall be 0.1% of containment air weight per day.

This correction does not change any of the conclusions in the safety evaluation associated with the amendment. We regret any inconvenience this may have caused. If you have any questions regarding this matter, please call me at (301) 415-4037.

Sincerely,

**/RA/**

Thomas J. Wengert, Senior Project Manager  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-255

Enclosure: Corrected TS Page

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