

SEQUOYAH JAN 2012 SRO EXAM
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76. Given the following plant conditions:

- Unit 1 has just experienced a Rx Trip and SI due to a failed open PORV.
- The crew has entered E-0, "Reactor Trip or Safety Injection."
- The operating crew is unable to close the associated PORV block valve.
- Pressurizer level indicates off-scale high.
- CCPIT Flow indicates 100 gpm.
- Containment pressure is 0.3 psig
- RCS pressure is 1150 psig and lowering.
- CETs 565°F and stable
- RVLIS Dynamic range 44% and lowering.

Which ONE of the following completes the statement below?

The initiation of an Appendix to place Hydrogen Analyzers in service will first be implemented (1) and the RCPs will be tripped to (2).

(1)

(2)

- | | |
|---------------------|----------------------------------------------------------------|
| A. by a step in E-0 | prevent excessive depletion of RCS inventory through the break |
| B. by a step in E-0 | prevent damage to RCPs operating in a highly voided system |
| C. by a step in E-1 | prevent excessive depletion of RCS inventory through the break |
| D. by a step in E-1 | prevent damage to RCPs operating in a highly voided system |

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77. Given the following:

- Unit 1 is at 100% power.
- RHR Pump 1B-B is tagged for motor bearing maintenance.
- An automatic Reactor Trip and Safety Injection occurs due to a rapid drop in RCS pressure.
- RHR Pump 1A-A trips on instantaneous overcurrent.
- Thirty minutes after the Reactor Trip, the crew transitions from E-1, "Loss of Reactor or Secondary Coolant", to ECA-1.1, "Loss of RHR Sump Recirculation".
- The following conditions are noted:
 - RCS pressure is currently 150 psig
 - RWST level is 54% and dropping
 - The lowest RCS Tcold is 280°F.
 - CNMT pressure is 6.2 psig and slowly lowering.

Which ONE of the following describes the required operator action?

- A. Transition to FR-P.1, "Pressurized Thermal Shock," and then to FR-Z.1, "High Containment Pressure," and ensure at least one Containment Spray pump running.
- B. Transition to FR-P.1, "Pressurized Thermal Shock," make preparations and start an RCP. Entry into FR-Z.1, "High Containment Pressure," is not required.
- C. Remain in ECA-1.1, and establish one train of ECCS flow. Entry into FR-Z.1, "High Containment Pressure," is not required.
- D. Transition to FR-P.1, "Pressurized Thermal Shock," and then transition to ECA-1.1 and concurrently perform actions of FR-Z.1, "High Containment Pressure."

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79. Given the following plant conditions:

- Unit 1 is at 100% power.
- The reactor trips, and subsequently, a loss of offsite power occurs.
- The crew is performing actions of ES-0.2, "Natural Circulation Cooldown."

Which of the following identifies both:

(1) the basis for checking T-hot less than 550°F

and

(2) the procedure transition required if an RCP is started?

- A. (1) To ensure the RCS remains 90°F subcooled when blocking low pressurizer pressure SI.
(2) Return to ES-0.1, "Reactor Trip Response," step 18, "DETERMINE if natural circulation cooldown is required:"
- B. (1) To ensure the RCS remains 90°F subcooled when blocking low pressurizer pressure SI.
(2) GO to 0-GO-7, "Unit Shutdown from Hot Standby to Cold Shutdown."
- C. (1) To ensure the RCS remains 50°F subcooled when blocking low pressurizer pressure SI.
(2) Return to ES-0.1, "Reactor Trip Response," step 18, "DETERMINE if natural circulation cooldown is required."
- D. (1) To ensure the RCS remains 50°F subcooled when blocking low pressurizer pressure SI.
(2) GO to 0-GO-7, "Unit Shutdown from Hot Standby to Cold Shutdown."

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80. Given the following plant conditions:

- Unit 1 is in MODE 5 following a Refuel Outage.
- ABGTS Train B is tagged for heater bank inspection.
- Fuel movement is in progress in the Spent Fuel Pool.
- Annunciator PS-32-104 TRAIN A AUX CONTROL AIR PRESSURE LOW (1-M-15) alarms.
- AUO reports the Train A Aux air is isolated from control air, Train A Aux Air Compressor is running and the Train A header is 67 psig and slowly lowering.

Based on the above conditons, which ONE of the following identifies the ABGTS status and the required Unit 1 Tech Spec entry, if any?

- A. ABGTS Train A remains OPERABLE until the Train A Containment Air Isolation valve automatically closes; Tech Spec 3.0.3 entry is NOT required.
- B. ABGTS Train A remains OPERABLE until the Train A Containment Air Isolation valve automatically closes; Tech Spec 3.0.3 entry is required.
- C. Both Trains of ABGTS are INOPERABLE; Tech Spec 3.0.3 entry is NOT required.
- D. Both Trains of ABGTS are INOPERABLE; Tech Spec 3.0.3 entry is required.

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81. Given the following plant conditions:

- The Unit 1 crew enters ECA-1.2, "LOCA Outside Containment."
- When ECA-1.2 is complete the following conditions exist:
 - CNMT pressure is 0.1 psig and stable.
 - RWST level is 70% and lowering.
 - CETs are indicating 510°F.
 - RCS pressure is 800 psig and lowering.
 - RVLIS is 46% and slowly rising.

Which ONE of the following completes the statements below?

The crew is required to transition to (1) .

The Emergency Plan Classification for this event is a(n) (2)_____ .

REFERENCE PROVIDED

- A. (1) E-1, "Loss of Reactor or Secondary Coolant"
(2) Site Area Emergency
- B. (1) ECA-1.1, "Loss of RHR Sump Recirculation"
(2) Notification of Unusual Event
- C. (1) E-1, "Loss of Reactor or Secondary Coolant"
(2) Notification of Unusual Event
- D. (1) ECA-1.1, "Loss of RHR Sump Recirculation"
(2) Site Area Emergency

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82. Given the following plant conditions:

- Unit 1 is operating at 25% power.
- 1-XS-68-339E, LEVEL CONTROL CHANNEL SELECTOR, is in the LI-68-339 & 335 position.
- AOP-I.11, "Eagle 21 Malfunction," was implemented due to a failure of an LCP in Eagle Rack 5.
- The crew is performing actions using AOP-I.04, "Pressurizer Instrument Control Malfunctions," due to the resulting failure of pressurizer level transmitter 1-LT-68-335.

Which ONE of the following completes the statements below?

In accordance with AOP-I.11, actions to hard trip the 1-LT-68-335 bistable should be (1) an Eagle system reset is attempted.

In accordance with T.S. 3.3.1.1, the maximum time allowed to hard trip the bistable is (2) hours.

- A. (1) completed before
(2) 6
- B. (1) completed before
(2) 8
- C. (1) delayed until after
(2) 6
- D. (1) delayed until after
(2) 8

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83. Given the following:

- 0500 - Conditions required the Unit 2 Main Control Room (MCR) to be abandoned.
- 0501 - The reactor is TRIPPED.
- 0505 - All MCR actions of AOP-C.04, "Shutdown From Auxiliary Control Room," were completed and the crew has established control in the Auxiliary Control Room.
- 0515 - Tech Spec 3.0.3 is entered.
- 0730 - The crew initiates a natural circulation cooldown and the following SG pressure trends are observed:

<u>Time</u>	<u>SG Pressures</u>
0730 -	1040 psig
0750 -	825 psig
0810 -	565 psig
0830 -	385 psig
0850 -	365 psig

Which ONE of the following completes the statements below?

Tech Spec 3.4.9 RCS Pressure/Temperature Limit (1) been violated.

The latest time allowed by Tech Specs to place the unit in Mode 5 is at (2) the next day.

- A. (1) has
(2) 1215
- B. (1) has
(2) 1815
- C. (1) has **NOT**
(2) 1215
- D. (1) has **NOT**
(2) 1815

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84. Given the following:

- Unit 1 is in Mode 3 with the RCS at normal operating pressure and temperature.
- 48 hours ago, RCS Activity was determined to be 0.38 microcuries/gram DOSE EQUIVALENT I-131.
- Chemistry now reports that the RCS Activity has been on a continuous slow increase and is now 0.43 microcuries/gram DOSE EQUIVALENT I-131.
- The crew begins an RCS cooldown.

Which ONE of the following completes the statements below?

RCS Tav_g will first be less than the required temperature when it is reduced to below (1).

The basis for the RCS Tav_g reduction is to limit doses at the site boundary in the event of a (2).

- A. (1) 500°F
(2) LOCA in conjunction with 0.25La leakage from containment
- B. (1) 500°F
(2) SGTR in conjunction with steady state SG tube leakage of 1 gpm
- C. (1) 350°F
(2) LOCA in conjunction with 0.25La leakage from containment
- D. (1) 350°F
(2) SGTR in conjunction with steady state SG tube leakage of 1 gpm

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85. Given the following plant conditions;

- Unit 1 has been operating for 377 days.
- The condensate storage tank is at 245,000 gals due to a leak.
- A loss of offsite power has occurred.
- Both units are tripped.
- Unit 1 has remained in Hot Standby conditions with T-hot at 550°F for 5 hours awaiting the return of offsite power.
- Power System Ops has just indicated that it will be at least another 8 hrs before offsite power can be restored.

Which ONE of the following identifies the procedural actions, if any, that would be required to comply with the times described in T.S. 3.7.1.3 "CONDENSATE STORAGE TANK" basis?

Note:

ES-0.2 "Natural Circulation Cooldown"

ES-0.3 "Natural Circulation Cooldown With Steam Void in the Vessel (with RVLIS)"

- A. Transition to ES-0.2 and cooldown to RHR entry conditions.
- B. Transition to ES-0.3 and cooldown to RHR entry conditions.
- C. Continue to maintain current plant conditions until offsite power is restored.
- D. Transition to ES-0.2, then transition to ES-0.3 to cooldown to RHR entry conditions.

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86.

Given the following plant conditions:

- Unit 2 is shutdown for maintenance.
- 2A-A RHR pump is in service with RCS temperature at 215°F.
- RCPs #2 and #4 are in service.
- RCPs #1 and #3 are unavailable and tagged.

- S/G Wide Range levels are: #1 #2 #3 #4
 22% 8% 6% 17%

Which ONE of the following identifies

(1) the minimum number of RHR pumps LCO 3.4.1.3 REACTOR COOLANT SYSTEM - SHUTDOWN, currently required to be operable

and

(2) the bases of the requirement?

(1)

(2)

- A. 1 to ensure sufficient circulation of the reactor coolant system to minimize the effects of a boron dilution accident.
- B. 2 to ensure sufficient circulation of the reactor coolant system to minimize the effects of a boron dilution accident.
- C. 1 to ensure sufficient heat removal is provided including single failure criteria.
- D. 2 to ensure sufficient heat removal is provided including single failure criteria.

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87. Given the following plant conditions:

- Unit 1 and 2 are in MODE 1.

Tuesday 2200

- 1A-A CCS pump is removed from service in accordance with 1-SO-70-1, "Component Cooling Water System "A" Train," and tagged for motor replacement.

Wednesday at 1000

- C-S CCS pump motor trips due to a bearing failure.
- AOP-M.03, "Loss of Component Cooling Water," is implemented.
- Unit 1 enters LCO 3.0.3.

Wednesday at 1130

- The 2B-B pump is realigned to supply Train B CCS as directed by AOP-M.03.

Wednesday at 1330

- 1A-A CCS pump is restored to operable status in accordance with 1-SO-70-1.

Which ONE of the following identifies...

(1) why the Train B ECCS pumps are required to be called inoperable on both units from 1000 to 1130 on Wednesday

and

(2) the earliest time on Wednesday that Unit 1 can exit LCO 3.0.3?

- A. (1) Because the pumps cannot fulfill the design function for sump recirculation.
(2) 1130
- B. (1) Because the pumps cannot fulfill the design function for sump recirculation.
(2) 1330
- C. (1) Because the pumps cannot be operated without CCS cooling water to their mechanical seal HXs.
(2) 1130
- D. (1) Because the pumps cannot be operated without CCS cooling water to their mechanical seal HXs.
(2) 1330

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88. Given the following plant conditions:

- Unit 1 is performing a shutdown **required** by Tech Specs.
- Currently the plant is in Mode 3 with a cooldown in progress to Mode 5.
- The OAC reports that indication on 1-PI-68-322, RCS PZR Pressure, has failed High.
- The crew is performing actions of AOP-I.04, "Pressurizer Instrument and Control Malfunctions,"

Which ONE of the following identifies...

(1) the PORV whose hand switch will be taken to CLOSE

and

(2) in accordance with Tech Specs, how this condition will affect the entry in to Mode 4?

<u>PORV</u>	<u>Mode 4 Entry</u>
A. PCV-334	is Allowed
B. PCV-340	is Allowed
C. PCV-340	is NOT Allowed
D. PCV-334	is NOT Allowed

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89. Given the following plant conditions:

- Time 1235: Unit 2 was operating at 100% power with CNTMT Spray Pump 2B-B tagged out of service.
Time 1236: A LOCA results in a Reactor Trip and Safety Injection.
Time 1315: The actions of ES-1.3, "Transfer to RHR Containment Sump," have been completed.
Time 1316: The crew observes the amps fluctuating on CNTMT Spray Pump 2A-A.
Time 1317: The crew transitions to ECA-1.3, "Containment Sump Blockage," and stops the pump.
Time 1318: The STA confirms Containment Pressure is 12.5 psig.

Which ONE of the following identifies both:

- (1) the required procedure transition, if any,
and
(2) the method of controlling containment pressure by procedure?
- A. (1) Implement FR-Z.1, "High Containment Pressure"
(2) restore normal containment cooling using EA-30-4, "Restoring Containment Coolers," after a TSC evaluation.
- B. (1) Implement FR-Z.1, "High Containment Pressure"
(2) restore containment spray by aligning one RHR pump to containment spray header.
- C. (1) Remain in ECA-1.3,
(2) restore containment spray using Appendix E, "Throttling Containment Spray Flow," after a TSC evaluation.
- D. (1) Remain in ECA-1.3,
(2) restore containment spray by aligning one RHR pump to containment spray header.

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90. Concerning Unit 1 Tech Spec 3.8.1.2, A.C. SOURCES - SHUTDOWN, which ONE of the following identifies both:

(1) a basis in Tech Specs for electrical distribution during shutdown and refueling conditions,
and

(2) the **MINIMUM** electrical power source combination that will satisfy Tech Spec?

- A. (1) Ensures the facility can be maintained in the shutdown or refueling condition for extended periods of time.
(2) 1A-A and 1B-B diesel generator sets and one offsite transmission network to a shutdown board.
- B. (1) Ensures at least one redundant set of onsite AC power sources remain Operable during accident conditions assuming a loss of offsite power and a single failure of the other onsite AC source.
(2) 1A-A and 2A-A diesel generator sets and one offsite transmission network to a shutdown board.
- C. (1) Ensures at least one redundant set of onsite AC power sources remain Operable during accident conditions assuming a loss of offsite power and a single failure of the other onsite AC source.
(2) 1A-A and 1B-B diesel generator sets and one offsite transmission network to a shutdown board.
- D. (1) Ensures the facility can be maintained in the shutdown or refueling condition for extended periods of time.
(2) 1A-A and 2A-A diesel generator sets and one offsite transmission network to a shutdown board.

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91. Given the following:

- Unit 1 is at 100% power.

Which ONE of the following completes the statements below?

The maximum number of igniters that can be inoperable without an action being required in accordance with LCO 3.6.4.3, Hydrogen Mitigation System, is (1).

In accordance with LCO 3.6.4.3, the **minimum** temperature the igniters must attain to be declared OPERABLE is (2).

- | | <u>(1)</u> | <u>(2)</u> |
|----|------------|------------|
| A. | 1 | 1700°F |
| B. | 1 | 1760°F |
| C. | 2 | 1700°F |
| D. | 2 | 1760°F |

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92. Given the following plant conditions:

- Unit 1 is at 100% power
- The Unit Supervisor enters AOP-R.01, "Steam Generator Tube Leak," due to an identified primary to secondary leakage of 5 gpd.
- Appendix A, "Radiation Monitor Trending Data," is being implemented with a trend established on ICS and values recorded every 15 minutes.
- Chemistry has provided the following data in the turnover package:

75 gpd = 354 cpm

50 gpd = 250 cpm

30 gpd = 166 cpm

5 gpd = 61 cpm

- The OATC reports that the data taken for rad monitor trending has increased to 300 cpm and appears to be stable.

Which ONE of the following identifies:

- (1) the maximum amount of time allowed to shutdown the unit per AOP-R.01
and
(2) the radiation monitor used for leak rate monitoring.

Note:

1-RM-90-119 Condenser Vacuum Pump Exhaust Radiation Monitor

1-RM-90-255 Unit 1 Condenser Vacuum Exhaust

0-GO-5, "Normal Power Operation"

0-GO-6, "Power Reduction From 30% Reactor Power To Hot Standby"

AOP-C.03, "Emergency Shutdown"

(1)

(2)

- | | |
|-----------------------------------------------|-------------|
| A. within 24 hrs. using 0-GO-5
and 0-GO-6. | 1-RM-90-255 |
| B. within 3 hrs. using AOP-C.03 | 1-RM-90-119 |
| C. within 24 hrs. using 0-GO-5
and 0-GO-6 | 1-RM-90-119 |
| D. within 3 hrs. using AOP-C.03 | 1-RM-90-255 |

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93. Given the following plant conditions:

- Unit 1 in Mode 3 with the RCS at normal operating temperature and pressure preparing for reactor startup.
- Alarm "PS-32-104 TRAIN A AUX CONTROL AIR PRESS LOW" (M15-B) is actuated.
- Control Air pressure indications are:
 - PI-32-200 Control Air Header pressure is 65 psig and lowering.
 - PI-32-104 Aux Bldg Control air header A pressure is 65 psig and lowering.
 - PI-32-105 Aux Bldg Control air header B pressure is 82 psig and rising.

Which ONE of the following identifies both...

1) the direction given the AUO

and

2) When reviewing Tech Specs in accordance with direction in AOP-M.02, "Loss of Control Air," which train(s) of AFW will be INOPERABLE on Unit 1?

Note:

0-FCV-32-82, Aux. Compsr. A-A Aux. Bldg Isol.

0-FCV-32-85, Aux. Compsr. B-B Aux. Bldg Isol.

1-FCV-32-80, Unit 1 Train A Rx Bldg Isol.

1-FCV-32-102, Unit 1 Train B Rx Bldg Isol.

Direction	<u>U-1 AFW Trains Inoperable</u>
A. Ensure 0-FCV-32-82, and 0-FCV-32-85 are closed.	Motor Driven Train A only
B. Ensure 1-FCV-32-80 and 1-FCV-32-102 are closed.	Motor Driven Train A and Turbine Driven Train
C. Ensure 0-FCV-32-82, and 0-FCV-32-85 are closed.	Motor Driven Train A and Turbine Driven Train
D. Ensure 1-FCV-32-80 and 1-FCV-32-102 are closed.	Motor Driven Train A only

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94. Given the following plant conditions:

- Unit 1 is operating at 100% power.
- I & C going to perform 1-SI-ICC-063-051.2, "Channel Calibration of RWST Level Channel II Rack 7 Loop L-63-51."
- As part of the surveillance the following 1-M-6E annunciator window inputs from 1-LT-63-51 will be disabled.

LS-63-50A RWST LVL LO
LS-63-50B RWST LVL LO-LO

- All other inputs to these annunciators will remain in service.
- The surveillance is expected to be completed by the end of shift.

In accordance with OPDP-4, "Annunciator Disablement," which ONE of the following completes the statement below as related to the SRO's responsibility for a 50.59 review and a Technical Evaluation?

Prior to disabling the annunciator input _____ is required.

- A. only a 50.59 review
- B. only a Technical Evaluation
- C. both a 50.59 review and a Technical Evaluation
- D. neither a 50.59 review nor a Technical Evaluation

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95. Given the following plant conditions:

- Unit 1 is being shutdown for a refueling outage.
- Operating crew is performing 0-GO-7, "Unit Shutdown From Hot Standby To Cold Shutdown."

Which ONE of the following identifies....

(1) when 0-GO-15, "Containment Closure Control", is required to be implemented in accordance with 0-GO-7

and

(2) if an evaluation in accordance with 0-GO-15 Appendix F, "Containment Closure Evaluation Process," indicates the ACT/ECT margin is 12 minutes, what is required?

Note:

ACT/ECT - Allowable Closure Time/Estimated Closure Time

	<u>(1)</u>	<u>(2)</u>
A.	When Unit enters Mode 5	Approval granted to open the penetration without restrictions
B.	When Unit enters Mode 5	Require person to close penetration to be stationed at the penetration
C.	When RHR is placed in service	Approval granted to open the penetration without restrictions
D.	When RHR is placed in service	Require person to close penetration to be stationed at the penetration

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96. Given the following plant conditions:

- An RCS heatup is in progress on Unit 1 in preparation for a Mode change following refueling.
- Tavg is 195°F.
- Lower containment average air temperature is 59°F.

Which ONE of the following identifies...

- (1) the actions relative to the upcoming Mode change
and
(2) the bases for the required minimum containment air temperature?
- A. (1) Mode change is allowed without requiring the provisions of LCO 3.0.4.b being met.
(2) To limit the air mass to prevent exceeding the maximum allowable internal containment pressure during LOCA conditions.
- B. (1) Mode change is allowed without requiring the provisions of LCO 3.0.4.b being met.
(2) To limit the air mass to prevent exceeding the maximum allowable internal containment pressure during a design basis steam line break.
- C. (1) Mode change requires implementing of the provisions of LCO 3.0.4.b.
(2) To limit the air mass to prevent exceeding the maximum allowable internal containment pressure during LOCA conditions.
- D. (1) Mode change requires implementing the provisions of LCO 3.0.4.b.
(2) To limit the air mass to prevent exceeding the maximum allowable internal containment pressure during a design basis steam line break.

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97. Given the following plant conditions:

- A LOCA occurred on Unit 1.
- A Site Area Emergency has been declared.
- The Containment Barrier is intact (i.e. no loss or potential loss of containment).
- The Containment Critical Safety Function Status Tree (FR-Z) is Yellow due to high radiation in containment.
- Containment pressure is 2.1 psig and decreasing.
- There is no release to the environment in progress.
- The "A" train of containment spray is operating normally for plant conditions.
- The "B" containment spray pump tripped, 25 minutes ago, after pump amps were observed to be oscillating.
- Authorization has been given for an emergency responder to receive an emergency exposure of 11 Rem TEDE in order to restore "B" train containment spray.

Which one of the following completes the statement below?

The decision (1) the requirements of EPIP-15, "Emergency Exposure Guidelines," because (2).

- | | <u>(1)</u> | <u>(2)</u> |
|----|------------|-----------------------------------------------------------------------------------------------------------------------------|
| A. | met | emergency exposure is necessary to maintain critical safety functions |
| B. | violated | emergency exposure is not necessary to maintain critical safety functions |
| C. | met | emergency exposure limits apply during accident classifications when at least 2 of 3 fission product barriers are lost. |
| D. | violated | emergency exposure limits <u>only</u> apply during an accident classification when all 3 fission product barriers are lost. |

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98. Given the following plant conditions:

- A diving operation in the Spent Fuel Pit is planned to commence later in the shift.

Which ONE of the following completes the statements below?

The dive (1) the requirements of NPG-SPP-07.3, " Work Activity Risk Management Process," to be classified as a HIGH risk activity

If the rad level in the area of the dive is estimated at 58 Rad/hr, in accordance with RCI-14, "Radiation Work Permit (RWP) Program," the RWP would be required to be approved by (2) .

- | | <u>(1)</u> | <u>(2)</u> |
|----|----------------------|------------------------------------------------|
| A. | meets | Radiation Protection Manager only |
| B. | meets | Radiation Protection Manager and Plant Manager |
| C. | does NOT meet | Radiation Protection Manager only |
| D. | does NOT meet | Radiation Protection Manager and Plant Manager |

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99. Given the following plant conditions:

- 0200 - Unit 2 is operating at 100% power when implementation of AOP-C.04, "Shutdown from Auxiliary Control Room," was required.
- 0204 - All main control room actions have been completed and the main control room is evacuated.
- 0212 - All transfer switches are placed in the required position in accordance with AOP-C.04, Checklist 2, Unit 2 Auxiliary Control Room.
- 0218 - AUO reports 6.9Kv Shutdown Board equipment is configured in accordance with Checklist 4.

Which ONE of the following completes the statements below?

The conditions require the declaration of a/an (1) in accordance with the Radiological Emergency Plan.

 (2) control is available for Pressurizer Backup Heaters 2A-A.

- A. (1) Alert
 (2) ONLY manual
- B. (1) Alert
 (2) BOTH manual and automatic
- C. (1) Site Area Emergency
 (2) ONLY manual
- D. (1) Site Area Emergency
 (2) BOTH manual and automatic

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100. In accordance with the Emergency Plan Implementing Procedures (EPIPs), which ONE of the following identifies a function of the Site Emergency Director (SED) that can be delegated and who the SED can delegate to perform this function?

<u>Function to be delegated</u>	<u>Can be delegated to...</u>
A. Emergency Doses that exceed occupational dose limits	CECC Director
B. Emergency Doses that exceed occupational dose limits	EP Manager
C. Protective Action Recommendations	CECC Director
D. Protective Action Recommendations	EP Manager

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References for 1201 NRC SRO Exam

1. Steam Tables
2. Mollier Diagram
3. Mechanical Flow Diagram – Essential Raw Cooling Water System, 1, 2-47W845-2
rev 104
4. Pressure Temperature Limits Report – Sequoyah Unit 1 LTOPS Selected Set points
5. EPIP-1 Fission Product Barrier Matrix, rev 43

SRO KEY

ANSWER KEY REPORT
for SQN JAN 2012 POST ATLANTA SRO EXAM Test Form: 0

#	ID	Points	Type	0	Answers
1	008 G2.4.18 76	1.00	MCS	A	
2	011 EA2.14 77	1.00	MCS	D	
3	025 AA2.04 78	1.00	MCS	A	
4	056 AG2.4.18 79	1.00	MCS	D	
5	065 AA2.08 80	1.00	MCS	C	
6	W/E04 G2.4.6 81	1.00	MCS	D	
7	028 AG2.4.11 82	1.00	MCS	C	
8	068 AG2.4.7 83	1.00	MCS	A	
9	076 AG2.2.40 84	1.00	MCS	B	
10	W/E10 EA2.2 85	1.00	MCS	D	
11	005 G2.2.25 86	1.00	MCS	C	
12	008 A2.01 87	1.00	MCS	B	
13	010 G2.1.7 88	1.00	MCS	B	
14	026 A2.07 89	1.00	MCS	C	
15	062 G2.2.25 90	1.00	MCS	D	
16	028 G2.2.40 91	1.00	MCS	C	
17	055 G2.1.19 92	1.00	MCS	C	
18	079 A2.01 93	1.00	MCS	A	
19	G 2.2.14 94	1.00	MCS	D	
20	G 2.1.8 95	1.00	MCS	B	
21	G 2.2.42 96	1.00	MCS	C	
22	G 2.3.4 97	1.00	MCS	B	
23	G 2.3.7 98	1.00	MCS	B	
24	G 2.4.35 99	1.00	MCS	B	
25	G 2.4.38 100	1.00	MCS	C	
SECTION 1 (25 items)		25.00			