Craver, Patti

153

From:

Min, Seung

Sent: To: Wednesday, September 14, 2011 12:29 PM Pham, Bo; Medoff, James; Yee, On; Ng, Ching

Subject:

RE: Questions/Issues related to North Anna Seismic Event

Attachments:

RG 1.167 ML003740093.pdf

Good Afternoon,

RG 1.167, Regulatory Position 1.2 addresses fatigue analysis for ASME Code Class 1 components and systems. I am wondering if activities per this position have been completed.

Thanks, Seung Min

C. REGULATORY POSITION

After a plant has been shut down by an earthquake, the guidelines for inspections and tests of nuclear power plant equipment and structures that are in EPRI NP-6695, depicted in Figure 3-2 and specified in Sections 5.3.2, 5.3.3, and 5.3.4; the documentation specified in Section in 5.3.5 to be submitted to the NRC; and the long-term evaluations that are specified in Section 6.3, with the exceptions specified below, are acceptable to the NRC staff for satisfying the requirements in Paragraph IV(a)(3) of Appendix S to 10 CFR Part 50.

1. EXCEPTIONS TO SECTION 6.3.4.1 OF EPRI NP-6695

1.1 Item (1) should read:

If the calculated stresses from the actual seismic loading conditions are less than the allowables for emergency conditions (e.g., ASME Code Level C Service Limits or equivalent) or original design bases, the item is considered acceptable, provided the results of inspections and tests (Section 5.3.2) show no damage.

1.2 The second dashed statement of Item (3) should read:

—An engineering evaluation of the effects of the calculated stresses on the functionality of the item. This evaluation should address all lo-

0/153

cations where stresses exceed faulted allowables and should include fatigue analysis for ASME Code Class 1 components and systems.

2. LONG-TERM EVALUATIONS

Coincident with the long-term evaluations, the plant should be restored to its current licensing basis. Exceptions to this must be approved by the Director, Office of Nuclear Reactor Regulation.

From: Pham, Bo

Sent: Wednesday, September 14, 2011 12:20 PM

To: Medoff, James; Fu, Bart; Homiack, Matthew; Sakai, Stacie; Brittner, Donald; Kalikian, Roger; Yee, On; Ng, Ching;

Min, Seung

Subject: Fw: Questions/Issues related to North Anna Seismic Event

Importance: High

RARB.

While the event at North Anna may seem to be only affecting the structural aspexts of the facility, please let Jim know today if there are other technical concerns within our purview that should be considered.

NRR is responsible for the decision to let NA restart post-earthquake. We need to do our best to inform NRR of all relevant issues that should be vetted before allowing them to start back up.

Thanks.
Sent from NRC blackberry
Bo Pham

From: Khanna, Meena

To: Mendiola, Anthony; Ulses, Anthony; Casto, Greg; Dennig, Robert; Bailey, Stewart; Taylor, Robert; Lupold, Timothy; Mitchell, Matthew; McMurtray, Anthony; Wilson, George; Murphy, Martin; Mathew, Roy; Tate, Travis; Martin, Robert; Harrison, Donnie; Circle, Jeff; Klein, Alex; Pham, Bo; Auluck, Rajender; Pelton, David; Kulesa, Gloria; Manoly, Kamal; Miranda, Samuel; Mitman, Jeffrey; Karipineni, Nageswara; Rahn, David; McConnell, Matthew; Goel, Vijay; Sahay, Prem; Som, Swagata; Scales, Kerby; Matharu, Gurcharan; Laur, Steven; Snodderly, Michael; Sheikh, Abdul; Bedi, Gurjendra; Lin, Bruce; Franke, Mark; McCoy, Gerald; Boyle, Patrick; Martin, Robert; Kulesa, Gloria

Cc: Hiland, Patrick; Howe, Allen; Galloway, Melanie; Lubinski, John; Wilson, George; Lee, Samson; Evans, Michele;

Giitter, Joseph; Farzam, Farhad **Sent**: Wed Sep 14 12:13:55 2011

Subject: Questions/Issues related to North Anna Seismic Event

All,

Pls note that the licensee has indicated that they plan to submit their restart document to us this Friday. Having said that, we'd like to request that all technical BCs, as applicable, identify any high priority questions, e.g., related to fuel, etc. be submitted to Bob Martin and myself by 2:30 pm today. The plan is for us to share these questions with the licensee by COB, today, and provide them with an opportunity to hold conference calls with us tomorrow to discuss, as needed. We would like to send whatever high priority questions we have today, so that at least they can consider them prior to issuing their document. Pls. note that this is by no means intended to be an all inclusive list.

In going forward, we will discuss a "systematic" approach in issuing the questions to the licensee at the 1:30 pm meeting today.

Thanks so much for all of the support and we'll see you at 1:30.

Meena Khanna, Branch Chief
Mechanical and Civil Engineering Branch
Division of Engineering
Office of Nuclear Reactor Regulation
(301)415-2150
meena.khanna@nrc.gov

REGULATORY GUIDE

OFFICE OF NUCLEAR REGULATORY RESEARCH

REGULATORY GUIDE 1.167

(Draft was DG-1035)

RESTART OF A NUCLEAR POWER PLANT SHUT DOWN BY A SEISMIC EVENT

A. INTRODUCTION

Paragraph IV(a)(3) of Appendix S, "Earthquake Engineering Criteria for Nuclear Power Plants," to 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities," requires shutdown of the nuclear power plant if vibratory ground motion exceeding that of the operating basis earthquake ground motion (OBE) occurs or if significant plant damage occurs. Prior to resuming operations, the licensee must demonstrate to the NRC that no functional damage has occurred to those features necessary for continued operation without undue risk to the health and safety of the public.

This guide provides guidance acceptable to the NRC staff for performing inspections and tests of nuclear power plant equipment and structures prior to restart of a plant that has been shut down by a seismic

The information collections contained in this regulatory guide are covered by the requirements of 10 CFR Part 50, which were approved by the Office of Management and Budget, approval number 3150-0011. The

¹Regulatory Guide 1.166, "Pre-Earthquake Planning and Immediate Nuclear Power Plant Operator Postearthquake Actions," provides criteria for plant shutdown.

NRC may not conduct or sponsor, and a person is not required to respond to, a collection of information unless it displays a currently valid OMB control number.

B. DISCUSSION

Data from seismic instrumentation² and a walkdown of the nuclear power plant are used to make the initial determination of whether the plant must be shut down after an earthquake, if the plant has not already shut down from operational perturbations resulting from the seismic event.

The Electric Power Research Institute has developed guidelines that will enable licensees to quickly identify and assess earthquake effects on nuclear power plants in EPRI NP-6695, "Guidelines for Nuclear Plant Response to an Earthquake," December 1989.3 This regulatory guide addresses sections of EPRI NP-6695 that relate to postshutdown inspection and tests, inspection criteria, inspection personnel, documentation, and long-term evaluations.

USNRC REGULATORY GUIDES

Regulatory Guides are issued to describe and make available to the public such informa-tion as methods acceptable to the NRC staff for implementing specific parts of the Com-mission's regulations, techniques used by the staff in evaluating specific problems or pos-tulated socidents, and data needed by the NRC staff in its review of applications for per-mits and licenses. Regulatory guides are not substitutes for regulations, and compliance with them is not required. Methods and solutions different from those set out in the guides will be acceptable if they provide a basis for the findings requisite to the lesuance or con-tinuance of a permit or license by the Commission.

This guide was issued after consideration of comments received from the public. Comments and suggestions for improvements in these guides are encouraged at all times, and guides will be revised, as appropriate, to accommodate comments and to reflect new in-

Written comments may be submitted to the Rules Review and Directives Branch, DFIPS, ADM, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

The guides are issued in the following ten broad divisions:

- Research and Test Reactors Fuels and Materials Facilities
- Environmental and Siting Materials and Plant Protection
- Transportation
 Occupational He

Single copies of regulatory guides may be obtained free of charge by writing the Office of Administration, Attention: Distribution and Mail Services Section, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; or by fex at (301)415-2280.

Issued guides may also be purchased from the National Technical Information Service on a standing order basis. Details on this service may be obtained by writing NTIS, 5285 Port Royal Road, Springfield, VA 22161.

²Regulatory Guide 1.12, Revision 2, "Nuclear Power Plant Instrumentation for Earthquakes," describes seismic instrumentation acceptable

³EPRI reports may be obtained from the Electric Power Research Institute, EPRI Distribution Center, 207 Coggins Dr., P.O. Box 23205, Pleasant Hill, CA 94523.

EPRI NP-6695 has been supplemented to add inspections and tests as a basis for acceptance of stresses in excess of Service Level C and to recommend that engineering evaluations of components with calculated stresses in excess of service Level D focus on areas of high stress and include fatigue analyses.

C. REGULATORY POSITION

After a plant has been shut down by an earthquake, the guidelines for inspections and tests of nuclear power plant equipment and structures that are in EPRI NP-6695, depicted in Figure 3-2 and specified in Sections 5.3.2, 5.3.3, and 5.3.4; the documentation specified in Section in 5.3.5 to be submitted to the NRC; and the long-term evaluations that are specified in Section 6.3, with the exceptions specified below, are acceptable to the NRC staff for satisfying the requirements in Paragraph IV(a)(3) of Appendix S to 10 CFR Part 50.

EXCEPTIONS TO SECTION 6.3.4.1 OF EPRI NP-6695

1.1 Item (1) should read:

If the calculated stresses from the actual seismic loading conditions are less than the allowables for emergency conditions (e.g., ASME Code Level C Service Limits or equivalent) or original design bases, the item is considered acceptable, provided the results of inspections and tests (Section 5.3.2) show no damage.

1.2 The second dashed statement of Item (3) should read:

—An engineering evaluation of the effects of the calculated stresses on the functionality of the item. This evaluation should address all locations where stresses exceed faulted allowables and should include fatigue analysis for ASME Code Class 1 components and systems.

2. LONG-TERM EVALUATIONS

Coincident with the long-term evaluations, the plant should be restored to its current licensing basis. Exceptions to this must be approved by the Director, Office of Nuclear Reactor Regulation.

D. IMPLEMENTATION

The purpose of this section is to provide guidance to applicants and licensees regarding the NRC staff's plans for using this regulatory guide.

Except in those cases in which the applicant proposes an acceptable alternative method for complying with the specified portions of the Commission's regulations, the method described in this guide will be used in the evaluation of applications for construction permits, operating licenses, combined licenses, or design certification submitted after January 10, 1997. This guide will not be used in the evaluation of an application for an operating license submitted after January 10, 1997, if the construction permit was issued prior to that date.

Holders of an operating license or construction permit issued prior to January 10, 1997, may voluntarily implement the methods described in this guide in combination with the methods in Regulatory Guides 1.12, Revision 2, "Nuclear Power Plant Instrumentation for Earthquakes," and 1.166, "Pre-Earthquake Planning and Immediate Nuclear Power Plant Operator Postearthquake Actions." Other implementation strategies, such as voluntary implementation of portions of the cited regulatory guides, will be evaluated by the NRC staff on a case-by-case basis.

REGULATORY ANALYSIS

A separate regulatory analysis was not prepared for this regulatory guide. The regulatory analysis, "Revision of 10 CFR Part 100 and 10 CFR Part 50," was prepared for these amendments, and it provides the regulatory basis for this guide and examines the costs and benefits of the rule as implemented by the guide. A copy of the regulatory analysis is available for inspection and copying for a fee at the NRC Public Document Room, 2120 L Street NW. (Lower Level), Washington, DC, as Attachment 7 to SECY-96-118.



Federal Recycling Program

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, DC 20555-0001

OFFICIAL BUSINESS
PENALTY FOR PRIVATE USE, \$300

FIRST CLASS MAIL
POSTAGE AND FEES PAID
USNRC
PERMIT NO. G-67