UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 15, 2012

Mr. Mano Nazar
Executive Vice President and
Chief Nuclear Officer
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

SUBJECT: TURKEY POINT, UNITS 3 AND 4 - ISSUANCE OF AMENDMENTS REGARDING HIGH RANGE-NOBLE GAS EFFLUENT MONITORS, MAIN STEAM LINES ACCIDENT MONITORING INSTRUMENTATION (TAC NOS. ME6891 AND ME6892)

Dear Mr. Nazar:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 250 to Renewed Facility Operating License No. DPR-31 and Amendment No. 246 to Renewed Facility Operating License No. DPR-41 for the Turkey Point Plant, Units Nos. 3 and 4, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated August 17, 2011, as supplemented by letters dated October 14, and December 1, 2011. The amendments revise items in TS 3.3.3.3, Table 3.3-5, Accident Monitoring Instrumentation, High Range-Noble Gas Effluent Monitors, Main Steam Lines, Instrument 19d, and TS 4.3.3.3, Table 4.3-4 related to the need to have High Range-Noble Gas Effluent Monitors for the Main Steam Lines. The proposed changes would relocate the TSs and surveillance requirements for this instrument to the Updated Final Safety Analysis Report and related procedures.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

Jason C. Paige, Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-250 and 50-251

Enclosures:
1. Amendment No. 250 to DPR-31
2. Amendment No. 246 to DPR-41
3. Safety Evaluation

cc w/enclosures: Distribution via Listserv

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT PLANT, UNIT NO. 3

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 250
Renewed License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power and Light Company (the licensee) dated August 17, 2011, as supplemented by letters dated October 14, and December 1, 2011, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Renewed Facility Operating License No. DPR-31 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 250 are hereby incorporated into this renewed license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA by Eva Brown for/

Douglas A. Broaddus, Chief
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Operating License
and Technical Specifications

Date of Issuance: June 15, 2012

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT PLANT UNIT NO. 4

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 246
Renewed License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power and Light Company (the licensee) dated August 17, 2011, as supplemented by letters dated October 14, and December 1, 2011, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Renewed Facility Operating License No. DPR-41 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 246 are hereby incorporated into this renewed license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA by Eva Brown for/

Douglas A. Broaddus, Chief
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Operating License
and Technical Specifications

Date of Issuance: June 15, 2012

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 250 RENEWED FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO. 246 RENEWED FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NOS. 50-250 AND 50-251

Replace Page 3 of Renewed Operating License DPR-31 with the attached Page 3.

Replace Page 3 of Renewed Operating License DPR-41 with the attached Page 3.

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

Remove pages	Insert pages
3/4 3-43	3/4 3-43
3/4 3-46	3/4 3-46

E. Pursuant to the Act and 10 CFR Parts 40 and 70 to receive, possess, and use at any time 100 milligrams each of any source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactively contaminated apparatus;

F. Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of Turkey Point Units Nos. 3 and 4.

3. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect, and is subject to the additional conditions specified below:

A. Maximum Power Level

The applicant is authorized to operate the facility at reactor core power levels not in excess of 2300 megawatts (thermal).

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 250 are hereby incorporated into this renewed license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

C. Final Safety Analysis Report

The licensee's Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on November 1, 2001, describes certain future inspection activities to be completed before the period of extended operation. The licensee shall complete these activities no later than July 19, 2012.

The Final Safety Analysis Report supplement as revised on November 1, 2001, described above, shall be included in the next scheduled update to the Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following the issuance of this renewed license. Until that update is complete, the licensee may make changes to the programs described in such supplement without prior Commission approval, provided that the licensee evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

E. Pursuant to the Act and 10 CFR Parts 40 and 70 to receive, possess, and use at any time 100 milligrams each of any source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactively contaminated apparatus;

F. Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of Turkey Point Units Nos. 3 and 4.

3. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect, and is subject to the additional conditions specified below:

A. Maximum Power Level

The applicant is authorized to operate the facility at reactor core power levels not in excess of 2300 megawatts (thermal).

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 246 are hereby incorporated into this renewed license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

C. Final Safety Analysis Report

The licensee's Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on November 1, 2001, describes certain future inspection activities to be completed before the period of extended operation. The licensee shall complete these activities no later than April 10, 2013.

The Final Safety Analysis Report supplement as revised on November 1, 2001, described above, shall be included in the next scheduled update to the Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following the issuance of this renewed license. Until that update is complete, the licensee may make changes to the programs described in such supplement without prior Commission approval, provided that the licensee evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

TABLE 3.3-5 (Continued)

ACCIDENT MONITORING INSTRUMENTATION

INSTRUMENT	TOTAL NO. OF CHANNELS	MINIMUM CHANNELS OPERABLE	APPLI-CABLE MODES	ACTIONS
14. In Core Thermocouples (Core Exit Thermo-couples)	4/core quadrant	2/core quadrant	1, 2, 3	31, 32
15. Containment High Range Area Radiation	2	1	1, 2, 3	34
16. Reactor Vessel Level Monitoring System	2(1)	1(1)	1, 2, 3	37, 38
17. Neutron Flux, Backup NIS (Wide Range)	2	1	1, 2, 3	31, 32
18. DELETED				
19. High Range-Noble Gas Effluent Monitors				
a. Plant Vent Exhaust	1	1	ALL	34
b. Unit 3-Spent Fuel Pit Exhaust	1	1	ALL	34
c. Condenser Air Ejectors	1	1	1, 2, 3	34
20. RWST Water Level	2	1	1, 2, 3	31, 32
21. Steam Generator Water Level (Narrow Range)	2/stm. Gen.	1/stm. Gen.	1, 2, 3	31, 32
22. Containment Isolation Valve Position Indication*	1/valve	1/valve	1, 2, 3	39

TABLE NOTATIONS

1. A channel is eight sensors in a probe. A channel is OPERABLE if a minimum of four sensors are OPERABLE.

2. Inputs to this instrument are from instrument items 3, 4, 5 and 14 of this Table.

* Applicable for containment isolation valve position indication designated as post-accident monitoring instrumentation (containment isolation valves which receive containment isolation Phase A, Phase B, or containment ventilation isolation signals).

TABLE 4.3-4

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

INSTRUMENT		CHANNEL CHECK	CHANNEL CALIBRATION
1.	Containment Pressure (Wide Range)	M	R
2.	Containment Pressure (Narrow Range)	M	R
3.	Reactor Coolant Outlet Temperature - T_{HOT} (Wide Range)	M	R
4.	Reactor Coolant Inlet Temperature - T_{COLD} (Wide Range)	M	R
5.	Reactor Coolant Pressure - Wide Range	M	R
6.	Pressurizer Water Level	M	R
7.	Auxiliary Feedwater Flow Rate	M	R
8.	Reactor Coolant System Subcooling Margin Monitor	M	R
9.	PORV Position Indicator (Primary Detector)	M	R
10.	PORV Block Valve Position Indicator	M	R
11.	Safety Valve Position Indicator (Primary Detector)	M	R
12.	Containment Water Level (Narrow Range)	M	R
13.	Containment Water Level (Wide Range)	M	R
14.	In Core Thermocouples (Core Exit Thermocouples)	M	R
15.	Containment - High Range Area Radiation Monitor	M	R*
16.	Reactor Vessel Level Monitoring System	M	R
17.	Neutron Flux, Backup NIS (Wide Range)	M	R
18.	DELETED		
19.	High Range - Noble Gas Effluent Monitors		
	a. Plant Vent Exhaust	M	R
	b. Unit 3 - Spent Fuel Pit Exhaust	M	R
	c. Condenser Air Ejectors	M	R
20.	RWST Water Level	M	R
21.	Steam Generator Water Level (Narrow Range)	M	R
22.	Containment Isolation Valve Position Indication	M	R

*Acceptable criteria for calibration are provided in Table II.F.1-3 of NUREG-0737.

UNITED STATES
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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 250 TO

RENEWED FACILITY OPERATING LICENSE NO. DPR-31 AND

AMENDMENT NO. 246 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-41

FLORIDA POWER AND LIGHT COMPANY

TURKEY POINT PLANT, UNIT NOS. 3 AND 4

DOCKET NOS. 50-250 AND 50-251

1.0 INTRODUCTION

By application dated August 17, 2011 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML11250A124), as supplemented by letters dated October 14 (ADAMS Accession No. ML11298A179), and December 1, 2011 (ADAMS Accession No. ML11350A234), Florida Power and Light Co. (FPL, the licensee) proposed an amendment to the technical specifications (TSs) for Turkey Point Plant, Units 3 and 4. The requested changes revise items in TS 3.3.3.3, Table 3.3-5, Accident Monitoring Instrumentation, High Range-Noble Gas Effluent Monitors, Main Steam Lines, Instrument 19d, and TS 4.3.3.3, Table 4.3-4 related to the need to have High Range-Noble Gas Effluent Monitors for the Main Steam Lines. The changes would relocate the TSs and surveillance requirements for this instrument to the Updated Final Safety Analysis Report and related procedures.

The supplements dated October 14, and December 1, 2011, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on October 18, 2011 (76 FR 64393).

2.0 REGULATORY EVALUATION

Title 10, Part 50, Section 50.36(c)(2)(i), of the *Code of Federal Regulations* (CFR) states the following:

> Limiting conditions for operation are the lowest functional capability or performance levels of equipment required for safe operation of the facility. When a limiting condition for operation of a nuclear reactor is not met, the licensee shall shut down the reactor or follow any remedial action permitted by the technical specifications until the condition can be met.

Title 10, Part 50, Section 50.36(c)(2)(ii), of the *Code of Federal Regulations* states the following:

> A technical specification limiting condition for operation of a nuclear reactor must be established for each item meeting one or more of the following criteria:
>
> > *(A) Criterion 1.* Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary.
> >
> > *(B) Criterion 2.* A process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
> >
> > *(C) Criterion 3.* A structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
> >
> > *(D) Criterion 4.* A structure, system, or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

Title 10, Part 50, Section 50.36(c)(3), of the *Code of Federal Regulations* states the following:

> Surveillance requirements are requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the limiting conditions for operation will be met.

Title 10, Part 50, Appendix A (General Design Criteria) of the *Code of Federal Regulations* states the following:

> *Criterion 13-Instrumentation and control.* Instrumentation shall be provided to monitor variables and systems over their anticipated ranges for normal operation, for anticipated operational occurrences, and for accident conditions as appropriate to assure adequate safety, including those variables and systems that can affect the fission process, the integrity of the reactor core, the reactor coolant pressure boundary, and the containment and its associated systems. Appropriate controls shall be provided to maintain these variables and systems within prescribed operating ranges.
>
> *Criterion 19-Control room.* A control room shall be provided from which actions can be taken to operate the nuclear power unit safely under normal conditions and to maintain it in a safe condition under accident conditions, including loss-of-coolant accidents. Adequate radiation protection shall be provided to permit

access and occupancy of the control room under accident conditions without personnel receiving radiation exposures in excess of 5 rem whole body, or its equivalent to any part of the body, for the duration of the accident. Equipment at appropriate locations outside the control room shall be provided (1) with a design capability for prompt hot shutdown of the reactor, including necessary instrumentation and controls to maintain the unit in a safe condition during hot shutdown, and (2) with a potential capability for subsequent cold shutdown of the reactor through the use of suitable procedures.

Criterion 64-Monitoring radioactivity releases. Means shall be provided for monitoring the reactor containment atmosphere, spaces containing components for recirculation of loss-of-coolant accident fluids, effluent discharge paths, and the plant environs for radioactivity that may be released from normal operations, including anticipated operational occurrences, and from postulated accidents.

Regulatory Guide (RG) 1.97, Revision 3, provides a method acceptable to the Nuclear Regulatory Commission (NRC, Commission) for complying with Commission regulations pertaining to instrumentation and control to be provided for monitoring plant variables and systems following an accident.

3.0 TECHNICAL EVALUATION

The proposed modifications do not involve any physical alterations of the Turkey Point units. The licensee's proposed amendment would modify two sections of its Turkey Point, Unit 3 & 4 TSs.

TS 3.3.3.3, Table 3.3-5, Item 19d and Technical Specification 4.3.3.3, Table 4.3-4, Item 19d

TS 3.3.3.3, Table 3.3-5, Item 19d currently specifies the minimum number of High Range Noble Gas Effluent Monitors on the Main Steam Lines required to be operable during specified operating modes. TS 4.3.3.3, Table 4.3-4, Item 19d establishes the surveillance requirements (i.e., channel check and channel calibration frequency) applicable to the High Range-Noble Gas Effluent Monitors for the Main Steam Lines. The licensee's proposed amendment would delete Item 19d from each of these tables. These TSs correspond to the instruments identified as RAD6426 in each facility.

By letter dated August 17, 2011, the licensee indicates that the RAD6426 instrument that performs the function of being a High Range-Noble Gas Effluent Monitor for the Main Steam Lines is a Type E, Category 2 variable that does not perform a function that would meet any of the 10 CFR 50.36(c)(2)(ii) criteria for being a limiting condition for operation.

In a letter dated January 26, 1984, the licensee provided its evaluation of their existing accident monitoring instrumentation and control systems against the provisions of RG 1.97, Revision 3. [Reference: Letter from J.W. Williams (Florida Power and Light Company) to Darrell G. Eisenhut, "Turkey Point Units 3 and 4, Docket Nos. 50-250 and 50-251, Supplement 1 to NUREG 0737 – Regulatory Guide 1.97 (Rev. 3), L-84-20, January 26, 1984."] In the parameter listing summary sheets provided with that letter, the High Range-Noble Gas Effluent Monitors for the Main Steam Lines (i.e., RAD6426) was identified as a Type C and Type E, Category 2 variable. According to RG 1.97, Rev. 3, Type C variables are "those variables that provide

information to indicate the potential for being breached or the actual breach of the barriers to fission product release, i.e., fuel cladding, primary coolant pressure boundary, and containment."

Subsequent to its initial RG 1.97 submission, the licensee performed a 10 CFR 50.59 evaluation (October 14, 2011, letter) to reclassify the RAD6426 instrument reading as being only a Type E variable under RG 1.97. The rationale for the change was that the detection range for the RAD6426 instrument was determined to be insufficient to perform radiation monitoring necessary to detect a steam generator tube rupture. Instead, the licensee credited the "detection system associated with the condenser air ejector exhaust (i.e., RAD6417)." The staff reviewed the 50.59 evaluation and the associated UFSAR change rationale documentation and concluded that Turkey Point provided adequate evaluation and rationale for the change.

By letter dated December 1, 2011, the licensee did confirm in its request for additional information (RAI) responses that controls for RAD6417 operability and surveillance are already included in the TSs. RAD6417 is addressed as items 19c in Tables 3.3-5 and 4.3-4 of TS 3.3.3.3. These controls are equivalent to the table entries for RAD6426 that are proposed for removal as part of this amendment request (i.e., items 19d in the same tables). The NRC staff finds that the RAD6417 instrument is already addressed in the Turkey Point TSs consistent with the requirements of 10 CFR 50.36(c) for an instrument that would perform an RG 1.97 Type C function.

The staff also inquired as to whether the RAD6426 instrument was solely credited in any emergency or abnormal procedures that would involve a steam generator tube rupture (i.e., the Type C variable function that it was originally credited with performing). Turkey Point provided supplemental information (December 1, 2011, letter) that detailed exactly where RAD6426 was included in its off-nominal and emergency operating procedures. In each case, the RAD6426 is not the sole indication of a potential release; however, it does provide potential value as an additional source of information to operators. The staff concluded, based on the review of the licensee's RAI response, which included excerpts from selected procedures, that RAD6426's use was consistent with an RG 1.97 Type E variable and did not need to be classified as a Type C variable with associated controls in the plant TSs.

4.0 STATE CONSULTATION

Based upon a letter dated May 2, 2003, from Michael N. Stephens of the Florida Department of Health, Bureau of Radiation Control, to Brenda L. Mozafari, Senior Project Manager, U.S. Nuclear Regulatory Commission, the State of Florida does not desire notification of issuance of license amendments.

5.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such

finding (76 FR 64393). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

Based on the licensee's continued conformance to 10 CFR 50.36(c), 10 CFR 50 Appendix A Criteria 13, 19, and 64, and RG 1.97, the staff determined that it is acceptable to remove Turkey Point TS 3.3.3.3, Table 3.3-5, Item 19d and TS 4.3.3.3, Table 4.3-4, Item 19d. The staff determined that that RAD6426 instrument, which was addressed by TS items being removed, is no longer relied upon to perform an RG 1.97 Type C function. The staff also determined that RAD6417, which now performs the Type C function for which RAD6426 was previously credited, is already included in the TSs. The staff has concluded that an equivalent level of safety is being maintained.

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Timothy S. Mossman

Date: June 15, 2012

June 15, 2012

Mr. Mano Nazar
Executive Vice President and
Chief Nuclear Officer
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

SUBJECT: TURKEY POINT, UNITS 3 AND 4 - ISSUANCE OF AMENDMENTS REGARDING HIGH RANGE-NOBLE GAS EFFLUENT MONITORS, MAIN STEAM LINES ACCIDENT MONITORING INSTRUMENTATION (TAC NOS. ME6891 AND ME6892)

Dear Mr. Nazar:

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A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,
/RA/
Jason C. Paige, Project Manager
Plant Licensing Branch II-2
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