



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

January 11, 2012

Mr. George H. Gellrich, Vice President
Calvert Cliffs Nuclear Power Plant, LLC
Calvert Cliffs Nuclear Power Plant
1650 Calvert Cliffs Parkway
Lusby, MD 20657-4702

SUBJECT: CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2 -
AMENDMENT RE: TECHNICAL SPECIFICATION TASK FORCE TRAVELER
487-A, REVISION 1, RELOCATE DNB PARAMETERS TO THE COLR (TAC
NOS. ME7086 AND ME7087)

Dear Mr. Gellrich:

The Commission has issued the enclosed Amendment No. 301 to Renewed Facility Operating License No. DPR-53 and Amendment No. 278 to Renewed Facility Operating License No. DPR-69 for the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated August 31, 2011.

The amendments revise TS 3.4.1, "RCS [reactor coolant system] Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits," the bases for TS 3.4.1 and TS 5.6.5, "Core Operating Limits Report," by replacing the DNB numeric limits with references to the CORL. The changes are consistent with TS Task Force change traveler TSTF-487-A, Revision 1, "Relocate DNB Parameters to the COLR." The amendments also remove outdated notes in TS 3.4.1 that were associated with the Unit No. 2 steam generator replacement in 2003.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink that reads "Douglas V. Pickett".

Douglas V. Pickett, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-317 and 50-318

Enclosures:

1. Amendment No. 301 to DPR-53
2. Amendment No. 278 to DPR-69
3. Safety Evaluation

cc w/encls: Distribution via Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CALVERT CLIFFS NUCLEAR POWER PLANT, LLC

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 301
Renewed License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Calvert Cliffs Nuclear Power Plant, LLC (the licensee) dated August 31, 2011, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2. of Renewed Facility Operating License No. DPR-53 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 301, are hereby incorporated into the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Nancy L. Salgado, Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the License and Technical
Specifications

Date of Issuance: January 11, 2012



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CALVERT CLIFFS NUCLEAR POWER PLANT, LLC

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 278
Renewed License No. DPR-69

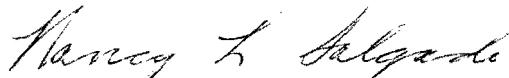
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 - A. The application for amendment by Calvert Cliffs Nuclear Power Plant, LLC (the licensee) dated August 31, 2011, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2. of Renewed Facility Operating License No. DPR-69 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 278, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Nancy L. Salgado, Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the License and Technical
Specifications

Date of Issuance: January 11, 2012

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 301 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-53

AMENDMENT NO. 278 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-69

DOCKET NOS. 50-317 AND 50-318

Replace the following page of the Facility Operating License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove Page

3

Insert Page

3

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages

3.4.1-1

3.4.1-2

5.6-5

5.6-6

Insert Pages

3.4.1-1

3.4.1-2

5.6-5

5.6-6

rules, regulations, and orders of the Commission, now or hereafter applicable; and is subject to the additional conditions specified and incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady-state reactor core power levels not in excess of 2737 megawatts-thermal in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 301, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications.

- (a) For Surveillance Requirements (SRs) that are new, in Amendment 227 to Facility Operating License No. DPR-53, the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment 227. For SRs that existed prior to Amendment 227, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the Surveillance was last performed prior to implementation of Amendment 227.

(3) Additional Conditions

The Additional Conditions contained in Appendix C as revised through Amendment No. 297 are hereby incorporated into this license. Calvert Cliffs Nuclear Power Plant, LLC shall operate the facility in accordance with the Additional Conditions.

(4) Secondary Water Chemistry Monitoring Program

The Calvert Cliffs Nuclear Power Plant, LLC; shall implement a secondary water chemistry monitoring program to inhibit steam generator tube degradation. This program shall include:

- a. Identification of a sampling schedule for the critical parameters and control points for these parameters;
- b. Identification of the procedures used to quantify parameters that are critical to control points;

C. This license is deemed to contain and is subject to the conditions set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act, and the rules, regulations, and orders of the Commission, now and hereafter applicable; and is subject to the additional conditions specified and incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor steady-state core power levels not in excess of 2737 megawatts-thermal in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 278 are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications.

(a) For Surveillance Requirements (SRs) that are new, in Amendment 201 to Facility Operating License No. DPR-69, the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment 201. For SRs that existed prior to Amendment 201, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the Surveillance was last performed prior to implementation of Amendment 201.

(3) Less Than Four Pump Operation

The licensee shall not operate the reactor at power levels in excess of five (5) percent of rated thermal power with less than four (4) reactor coolant pumps in operation. This condition shall remain in effect until the licensee has submitted safety analyses for less than four pump operation, and approval for such operation has been granted by the Commission by amendment of this license.

(4) Environmental Monitoring Program

If harmful effects or evidence of irreversible damage are detected by the biological monitoring program, hydrological monitoring program, and the radiological monitoring program specified in the Appendix B Technical Specifications, the licensee will provide to the staff a detailed analysis of the problem and a program of remedial action to be taken to eliminate or significantly reduce the detrimental effects or damage.

RCS Pressure, Temperature, and Flow DNB Limits
3.4.1

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.1 RCS Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits

LCO 3.4.1 RCS DNB parameters for pressurizer pressure, cold leg temperature, and RCS total flow rate shall be within the limits specified in the COLR.

APPLICABILITY: MODE 1.

----- NOTE -----
Pressurizer pressure limit does not apply during:

- a. THERMAL POWER ramp > 5% RTP per minute; or
- b. THERMAL POWER step > 10% RTP.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. RCS DNB parameter(s) not within limits.	A.1 Restore parameter(s) to within limit.	2 hours
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 2.	6 hours

RCS Pressure, Temperature, and Flow DNB Limits
3.4.1

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.1.1	Verify pressurizer pressure is within the limits specified in the COLR.	12 hours
SR 3.4.1.2	Verify RCS cold leg temperature is within the limits specified in the COLR.	12 hours
SR 3.4.1.3	Verify RCS total flow rate is greater than or equal to the limits specified in the COLR.	12 hours
SR 3.4.1.4	Verify measured RCS total flow rate is within the limits specified in the COLR.	24 months

5.6 Reporting Requirements

25. CEN-161-(B)-P, Supplement 1-P, "Improvements to Fuel Evaluation Model"
 26. Letter from Mr. S. A. McNeil, Jr. (NRC) to Mr. J. A. Tiernan (BG&E), dated February 4, 1987, Docket Nos. 50-317 and 50-318, "Safety Evaluation of Topical Report CEN-161-(B)-P, Supplement 1-P, Improvements to Fuel Evaluation Model"
 27. CEN-372-P-A, "Fuel Rod Maximum Allowable Gas Pressure"
 28. CENPD-135, Supplement 5-P, "STRIKIN-II, A Cylindrical Geometry Fuel Rod Heat Transfer Program"
 29. CENPD-387-P-A, "ABB Critical Heat Flux Correlations for PWR Fuel"
 30. CENPD-404-P-A, "Implementation of ZIRLO™ Cladding Material in CE Nuclear Power Fuel Assembly Designs"
 31. WCAP-11596-P-A, "Qualification of the PHOENIX-P, ANC Nuclear Design System for Pressurized Water Reactor Cores"
 32. WCAP-10965-P-A, "ANC: A Westinghouse Advanced Nodal Computer Code"
 33. WCAP-10965-P-A Addendum 1, "ANC: A Westinghouse Advanced Nodal Computer Code; Enhancements to ANC Rod Power Recovery"
 34. WCAP-16072-P-A, "Implementation of Zirconium Diboride Burnable Absorber Coatings in CE Nuclear Power Fuel Assembly Designs"
 35. WCAP-16045-P-A, "Qualification of the Two-Dimensional Transport Code PARAGON"
- c. The core operating limits shall be determined assuming operation at RTP such that all applicable limits (e.g., fuel

5.6 Reporting Requirements

thermal mechanical limits, core thermal hydraulic limits, ECCS limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.

- d. The COLR, including any mid cycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.

5.6.6 Not Used

5.6.7 Post-Accident Monitoring Report

When a report is required by Condition B or F of LCO 3.3.10, "Post Accident Monitoring Instrumentation," a report shall be submitted within the following 14 days. The report shall outline the preplanned alternate method of monitoring, the cause of the inoperability, and the plans and schedule for restoring the instrumentation channels of the Function to OPERABLE status.

5.6.8 Tendon Surveillance Report

Any abnormal degradation of the containment structure detected during the tests required by the Pre-Stressed Concrete Containment Tendon Surveillance Program shall be reported to the NRC within 30 days. The report shall include a description of the tendon condition, the condition of the concrete (especially at tendon anchorages), the inspection procedures, the tolerances on cracking, and the corrective action taken.

5.6.9 Steam Generator Tube Inspection Report

A report shall be submitted within 180 days after the initial entry into MODE 4 following completion of an inspection performed in accordance with the Specification 5.5.9, Steam Generator (SG) Program. The report shall include:



UNITED STATES
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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 301 TO RENEWED

FACILITY OPERATING LICENSE NO. DPR-53

AND AMENDMENT NO. 278 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-69

CALVERT CLIFFS NUCLEAR POWER PLANT, LLC

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-317 AND 50-318

1.0 INTRODUCTION

By application dated August 31, 2011 (Agencywide Documents Accession and Management System (ADAMS) Accession No. ML11244A132), Calvert Cliffs Nuclear Power Plant, LLC, the licensee, requested changes to the Technical Specifications (TSs) for Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2. The proposed changes would revise TS 3.4.1, "RCS [reactor coolant system] Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits," the bases for TS 3.4.1 and TS 5.6.5, "Core Operating Limits Report [COLR]," by replacing the DNB numeric limits with references to the COLR. The changes are consistent with TS Task Force change traveler TSTF-487-A, Revision 1, "Relocate DNB Parameters to the COLR." The proposed changes also include removal of outdated notes in TS 3.4.1.c and Surveillance Requirement (SR) 3.4.1.3 that were temporarily inserted to support replacement of the steam generators in 2002.

These changes would allow the licensee to recalculate the DNB parameter limits using Nuclear Regulatory Commission (NRC)-approved methodologies without the need for a license amendment request (LAR).

The proposed changes include the following:

- Change TS 3.4.1, "RCS Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits," and the associated Surveillance Requirements (SRs) to replace the specific limit values of RCS pressurizer pressure, cold leg temperature, and RCS total flow rate with "the limits specified in the COLR."
- Change the bases for limiting condition for operation (LCO) 3.4.1 to reflect that the DNB limits are specified in the COLR.
- Change TS 5.6.5, "Core Operating Limits Report," to add the methodology requirements used to calculate the DNB limits.

- Remove outdated notes included in TS 3.4.1.c and SR 3.4.1.3 that were associated with the Unit No. 2 steam generator replacement in 2003.

Generic Letter (GL) 88-16, "Removal of Cycle-Specific Parameter Limits from Technical Specifications," is the regulatory guidance for this change.

2.0 REGULATORY EVALUATION

The Commission's regulatory requirements related to the content of TSs are specified in Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.36, "Technical specifications." 10 CFR 50.36(c)(2)(i) defines that LCOs are the lowest functional capability or performance levels of equipment required for safe operation of the facility. For the DNB parameters, 10 CFR 50.36(c)(2)(ii)(B) Criterion 2 applies, which requires that TS LCOs be established for each process variable, design feature, or operating restriction that is an initial condition of a design-basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.

LARs are required for each fuel cycle design that results in changes to parameter limits specified in TS. To meet 10 CFR 50.36(c)(2)(ii) requirements and alleviate the need for LARs to update parameter limits every fuel cycle, the NRC issued GL 88-16 with specific guidance for replacing the limit values for cycle-specific parameters in the TSs with references to an owner-controlled document, namely, the COLR. The guidance in GL 88-16 includes the following three actions:

1. The addition of the definition of a named formal report (i.e., COLR) in TSs that includes the values of cycle-specific parameter limits that have been established using an NRC-approved methodology and consistent with all applicable limits of the safety analyses.
2. The addition of an administrative reporting requirement (in TS 5.6.5) to submit the formal report on cycle-specific parameter limits to the Commission for information.
3. The modification of individual TSs to note that the specific parameters shall be maintained within the limits provided in the defined formal report (COLR).

The proposed change has been evaluated against GL 88-16 and found to be consistent with that regulatory guidance.

3.0 TECHNICAL EVALUATION

3.1 TSTF-487-A, Revision 1

TS LCO 3.4.1 specifies the limit values of the DNB parameters to assure that the pressurizer pressure, the RCS cold leg temperature, and RCS flow rate during operation at rated thermal power (RTP) will be maintained within the limits assumed in the safety analyses in the final safety analysis report (FSAR). The safety analyses of anticipated operational occurrences (AOOs) and accidents assume initial conditions within the envelope of normal steady state operation at the RTP to demonstrate that the applicable acceptance criteria, including the specified acceptable fuel design limits (such as DNB ratio) and RCS pressure boundary design conditions, are met for each event analyzed. The TS limits placed on the DNB-related

parameters ensure that these parameters, when appropriate measurement uncertainties are applied, will be bounded by those assumed in the safety analyses, and thereby provide assurance that the applicable acceptance criteria will not be violated should a transient or accident occur while operating at the RTP.

It is essential to safety that the plant is operated within the DNB parameter limits. This change retains the requirement to maintain the plant within the DNB parameter limits in LCO 3.4.1 along with the SR verification for each of the DNB parameters. As these parameter limits are calculated using NRC-approved methodologies and are consistent with all applicable limits of the plant safety analyses, this change does not affect nuclear safety.

TS 5.6.5, "Core Operating Limits Report," specifies that the core operating limits shall be determined such that all applicable limits of the safety analyses are met, and that the analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC. These changes modify the list of NRC-approved methodologies in TS 5.6.5 to include those used to calculate the DNB limits on pressurizer pressure, RCS cold leg temperature, and RCS total flow rate. The limit values of these parameters in the COLR will comply with existing operating fuel cycle analysis requirements, and are initial conditions assumed in safety analyses. Replacing of the DNB parameter values with references to the COLR does not lessen the requirement for compliance with all applicable limits.

Any revisions to the safety analyses that require prior NRC approval will be identified by the 10 CFR 50.59 review process. TS 5.6.5 also specifies that the COLR, including any midcycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC. This will allow NRC staff to continue trending the information even though prior NRC approval of the changes to these limits will not be required.

10 CFR 50.36 requires LCOs to contain the lowest functional capability or performance levels of equipment for safe operation of the facility. The NRC staff finds that the proposed change to LCO 3.4.1 referencing the specific values of the DNB parameter limits in TS in the COLR continues to meet the regulatory requirement of 10 CFR 50.36(c)(2)(ii)(B) (Criterion 2), and follows the guidance described in GL 88-16. The NRC staff, therefore, concludes that this change is acceptable.

For safety analyses of transients or accidents, various sections of Chapter 15 of the Standard Review Plan specify that the reactor is initially at the RTP plus uncertainty, and the RCS flow is at nominal design flow including the measurement uncertainty. If one or more DNB parameter limits change, and these changes do not support the RTP, a license amendment would be required to either reduce the RTP or limit the plant operation at a level below the RTP. 10 CFR Part 50 Appendix K requires that the loss-of-coolant accident analysis be performed at 102% of the RTP. Other plant-specific analyses can contain an initial condition to be performed at RTP. To ensure a clear understanding of this requirement, TS 5.6.5.c. has been reworded to add the underlined text:

The core operating limits shall be determined assuming operation at RTP such that all applicable limits (e.g., fuel thermal mechanical limits, core thermal hydraulic limits, ECCS [emergency core cooling system] limits, nuclear limits such as SDM [shutdown margin], transient analysis limits, and accident analysis limits) of the safety analysis are met.

The NRC staff has reviewed this proposed change to replace the values of the DNB parameters in TS with references to the COLR. This change will allow the licensee the flexibility to manage operating and core design margins associated with the DNB parameters without the need for cycle-specific LARs. Any future revisions to safety analyses that require prior NRC approval will be identified by the 10 CFR 50.59 review process. Based on this evaluation, the NRC staff concludes that this change meets the regulatory requirements of 10 CFR 50.36, follows the guidance described in GL 88-16, and is acceptable.

3.2 Removal of Notes to TS 3.4.1

Amendments Nos. 249 and 225 for Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2, respectively, issued on March 1, 2002 (ADAMS Accession No. ML012400449), included temporary notes in TS 3.4.1 that reflected replacement of the steam generators. As discussed in the safety evaluation, TS 3.4.1.c requires a minimum RCS flow rate of 370,000 gpm and SR 3.4.1.3 requires verification every 12 hours. However, due to steam generator tube plugging over the years, this flow rate could no longer be attained with the original steam generators. As a result, RCS flow was limited to 340,000 gpm.

When the Unit No. 1 steam generators were replaced in the spring 2002 refueling outage, the original RCS flow rate was restored. Unit No. 2 steam generator replacement was scheduled for the spring 2003 refueling outage. Therefore, a temporary note was inserted into TS 3.4.1.c and SR 3.4.1.3 stating that the Unit No. 2 RCS flow rate remained at 340,000 gpm. When the Unit No. 2 steam generator replacement was completed in 2003, the notes were no longer applicable.

The licensee has proposed removing these notes as they are no longer applicable. The NRC staff considers this change to be administrative and approves removing the notes.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Maryland State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes Surveillance Requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (76 FR 64390). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principle Contributor: Douglas Pickett, NRR

Date: January 11, 2012

January 11, 2012

Mr. George H. Gellrich, Vice President
Calvert Cliffs Nuclear Power Plant, LLC
Calvert Cliffs Nuclear Power Plant
1650 Calvert Cliffs Parkway
Lusby, MD 20657-4702

SUBJECT: CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2 -
AMENDMENT RE: TECHNICAL SPECIFICATION TASK FORCE TRAVELER
487-A, REVISION 1, RELOCATE DNB PARAMETERS TO THE COLR (TAC
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Dear Mr. Gellrich:

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A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,

/ra/

Douglas V. Pickett, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-317 and 50-318

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