

DOCKET NO. 50-263...NSPC..MONTICELLO

Revision 1 to License Amendment requested  
dtd 5/1/79 Proposed Changes to Technical  
Specification Appendix A of Operating Lic.  
DPR-22

Rec'd w/ltr 7/23/82...8208020320

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EXHIBIT A

Monticello Nuclear Generating Plant  
Docket No. 50-263

Revision 1 to  
License Amendment Requested dated May 1, 1979  
Proposed Changes to Technical Specification  
Appendix A of Operating  
License No. DPR-22

Pursuant to 10 CFR 50.59 and 50.90, the holders of Operating License DPR-22 hereby propose the following changes to Appendix A, Technical Specifications.

PROPOSED CHANGES

1. Delete the Appendix B, Section 2.4, Interim Radioactive Effluent Technical Specifications in the entirety.
2. Delete the reference to Appendix B Technical Specifications in Section 2.C.2 of the Monticello Operating License.
3. Revise the Appendix A Technical Specifications as shown in Exhibit B.

REASON FOR CHANGES

These changes are being proposed at the request of the NRC Staff. They are needed to implement the requirements of 10 CFR Part 50, Appendix I, and 40 CFR Part 190.

Guidance for the preparation of these proposed changes was included in a letter, dated November 15, 1978, from Mr. Brian Grimes, Assistant Director for Engineering and Projects, Division of Operating Reactors, USNRC. Included with this letter were model radiological effluent Technical Specifications, "Draft Radiological Effluent Technical Specifications for BWR's", NUREG-0473, October, 1978. Additional guidance was provided at a Regional meeting in Chicago on November 28, 1978.

In May, 1979 proposed Technical Specifications, an Offsite Dose Calculation Manual (ODCM), and a Process Control Program (PCP) were submitted for NRC Staff review. Following a meeting with the NRC Staff in 1979, a meeting with the NRC Staff and their contractor, EG&G Idaho, in February, 1982, and a number of followup discussions with our Project Manager in the Division of Licensing we have agreed to a number of changes in the Technical Specifications originally submitted. The ODCM and PCP's have also been revised to address NRC Staff and EG&G comments.

The revised Technical Specification pages, ODCM, and PCP's reflect all of the changes discussed with the NRC Staff and EG&G representatives, including the changes to the NRC Model Technical Specifications contained in a memorandum from C A Willis and F J Congel dated May 19, 1982, "Summary of Draft Contractor Guidance of RETS."

There has been only one significant change from the draft Technical Specifications previously reviewed by the NRC and their contractors. The requirements for monitoring of turbine building air whenever a roof vent is opened have been deleted. A plant design change to be implemented during the 1982 refueling outage (scheduled to begin in September) will reroute all turbine building roof vents to the reactor building ventilation plenum. All turbine building exhaust air will therefore be monitored by the reactor building vent monitoring system. Accordingly, Specification 4.8.B.3.b on page 198a has been deleted.

#### SAFETY EVALUATION

The proposed changes implement the requirements of 10 CFR Part 50, Appendix I, and 40 CFR Part 190. They are being submitted at the request of the NRC Staff and generally conform to the model technical specifications contained in NUREG-0473. When implemented, they will enhance the protection provided to the public by requiring adherence to the design objectives of Appendix I to be periodically verified with calculations using models approved by the NRC Staff.