Minneapolis, Minnesota 55401	10-29-71	TI-2-71	NO.: 4778
	TR. MEMO: REPORT: OTHER:		
70:	ORIG.: CC: OTHER:		
	1\signed		
L. Manning Muntzing		ONCURRENCE	DATE ANSWERED: BY:
CLASSIF.: POST OFFICE REG. NO:	50-263 50-282 50-306 (ENVIRO FILE)		
DESCRIPTION: (Must Be Unclassified)	REFERRED TO	DATE REC	CEIVED BY DATE
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10-15-71 submittalsre SHOW CAUSE	W/4 cye for ACTION	50-263) nona	IMMI COPCO
Statements for Monticello & Prairie			A M PPROPE
Islandtrans the following:	Long	11-2-71	W.W1111ams F-30
ENCLOSURES:	W/4 cys for ACTION	(50-282/306)	G. Blanc
(1) Ltr notarized 10-15-71 for SHOW CAUSE	DISTRIBUTION:		
Statement of 10-15-71 for Monticello. (2) Ltr notarized 10-15-71 for SHOW CAUSE		docker) E. G.	Case
(S) I'EL DOCULIZED TO-13-11 FOR SHOW CHARACT	AEC PDR(3)(1 ga d	(3/12/23)	phnson
Statement of 10-15-71 for Prairie Island.	Compliance (2)	LUCEI	
(3) Errata Sheet for SHOW CAUSE Statement		DeYou	
for Honticello.	Boyd/Grimes	Dente	
(4) Errata Sheet for SHOW CAUSE Statement REMARKS: for Prairie Island Units 1 & 2.			Laughlin)
(1 cy ea notarized ltr, 50 cys es Errata	Beck/Mann		Buchanan)
Sheets rec'd)	Felton P FACA		Hiles, C-459 GT otter, E-201 GT
Holding (3) cys for ACRS (50-263 &	OGC, Rm P 506A Norris/Schroeder/		
50-282/306) Local PDR(Red Wing, Minn (50-28)	2/306) L. Rosers	1	Nunnq ₇ -A-170 fod
U. S. TATO	OMIC ENERGY COMMISSION MA	LIL CONTROL	FORM FORM AEC-328 (8-60)

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A TOMIC ENERGY

COMMISSION Regulatory

Regulatory

MINNEAPOLIS, MINNESOTA 5540

October 29, 1971

Mr L Manning Muntzing Director of Regulation U S Atomic Energy Commission Washington, DC 20545

Dear Mr Muntzing:

MONTICELLO NUCLEAR CENERATING PLANT E-5979 Operating License DPR-22

PRAIRIE ISLAND NUCLEAR GENERATING PLANT E-6197 Construction Permits CPPR-45 and CPPR-46

Our letters of October 15 for the above projects furnished information pursuant to Section E.3 of Appendix D to 10 CFR Part 50. Inadvertently we failed to have these two letters notarized. We therefore are enclosing substitute letters that have been appropriately notarized.

Unfortunately, several minor typographical errors occurred in the Prairie Island Nuclear Generating Plant "statement" and to correct for this we are enclosing 50 copies of an Errata sheet.

We are also furnishing an errata sheet, in 50 copies, for the Monticello "statement" which includes supplemental information requested by your Mr Victor Benaroya.

Yours very truly,

A V Dienhart Vice President - Engineering

Subscribed and sworn to before me

this $\frac{\cancel{\cancel{2}}\cancel{\cancel{9}}}{\cancel{\cancel{9}}}$

_ day/off

olev, 1971

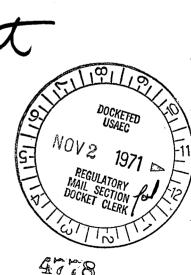
John J Smith

(Notary Public, Hennepin County, Minnesota

My Commission Expires

March 3, 1976

Notarial Seal



Reastred W/Lt: Bated 10-28-7/

Reference: DPR-22 50-263

ERRATA FOR

MONTICELLO NUCLEAR GENERATING PLANT

STATEMENT

TO

U S ATOMIC

ENERGY COMMISSION



Page 3, Line 12 - following word "reactor" change "of" to "at"

Page 3, Line 13 - following word "in" delete "the"

Page 5, Line 14 - change "wise" to "wide"

Page 9, substitute replacement Page 9 and add Page 9a (dated 10/29/71)

Page 15, Line 7 - change "is" to "it"

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2. Operating Experience

For the operating period ending June 30, 1971 the radioactive releases from the plant were a small fraction of 1% of the Technical Specifications-allowed releases. During the period liquid releases amounted to 4.5 x 10⁻⁶ percent. Related gaseous releases were 28 x 10⁻³ percent for noble gases and 12 x 10⁻² percent for iodines and particulates. Solid wastes generated during the period had a total activity of 7.6 curies and all of this material was shipped off-site to an AEC approved disposal facility. Further data can be found in the Six-Month Operating Report No. 1 submitted to the AEC Division of Reactor Licensing on August 30, 1971.

On July 14, 1971 a series of events occurred, initiated by an equipment malfunction, that resulted in an unexpected gaseous release at a rate of about 35,000 micro curies per second (µci/sec) through the main stack and about 20,000 µci/sec through the reactor building vent, which persisted for a period of about 30 minutes. This release was well within the Technical Specification limit and was promptly reported to AEC. Subsequent analysis of this release has shown the maximum integrated off-site dose to be about 0.033 millirem at the site boundary. Such a dose is insignificant compared to a natural background radiation of about 120 millirem per year in the Monticello area.

With the plant now operating for longer periods of time near maximum load, the amount of radioactivity released has increased slightly but still is less than a few percent of the Technical Specification limits. During the review period it is expected that the radwaste releases will be less than 5% of the Technical Specification limits.

The nearest public water supply using Mississippi River water is the Minneapolis-St Paul system about 35 miles downstream from the plant. As a result, liquid releases before discharge to the river must conform to the Technical Specification limits on an instantaneous batch-by-batch basis without