Dominion Nuclear Connecticut, Inc.
Millstone Power Station
Rope Ferry Road
Waterford, CT 06385

Dominion

NOV 07 2011

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Serial No.	11-626
MPS Lic/LES	R0
Docket No.	50-423
License No.	NPF-49

DOMINION NUCLEAR CONNECTICUT, INC.
MILLSTONE POWER STATION UNIT 3
LICENSEE EVENT REPORT 2011-001-00
MILLSTONE POWER STATION UNIT 3 FAILURE TO ENTER TECHNICAL SPECIFICATION ACTION STATEMENT

This letter forwards Licensee Event Report (LER) 2011-001-00 documenting a condition prohibited by the plant's Technical Specifications at Millstone Power Station Unit 3 on September 10, 2011. This LER is being submitted pursuant to 10 CFR 50.73(a)(2)(i)(B).

If you have any questions or require additional information, please contact Mr. William D. Bartron at (860) 444-4301.

Sincerely,

A. J. Jordan
Site Vice President – Millstone

Attachments: 1

Commitments made in this letter: None

IE22
NRR

cc: U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406-1415

C. J. Sanders
Project Manager - Millstone Power Station
U.S. Nuclear Regulatory Commission
One White Flint North
11555 Rockville Pike
Mail Stop 08B3
Rockville, MD 20852-2738

NRC Senior Resident Inspector
Millstone Power Station

NRC FORM 366 (10/2010) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB: NO. 3150-0104 EXPIRES: 10/31/2013

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

1. FACILITY NAME	2. DOCKET NUMBER	3. PAGE
Millstone Power Station - Unit 3	05000423	1 OF 2

4. TITLE

Failure to Enter Technical Specification Action Statement

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
										05000
09	10	2011	2011	– 001	– 00	11	07	2011	FACILITY NAME	DOCKET NUMBER
										05000

9. OPERATING MODE

1

10. POWER LEVEL

100

11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: *(Check all that apply)*

☐ 20.2201(b)	☐ 20.2203(a)(3)(i)	☐ 50.73(a)(2)(i)(C)	☐ 50.73(a)(2)(vii)
☐ 20.2201(d)	☐ 20.2203(a)(3)(ii)	☐ 50.73(a)(2)(ii)(A)	☐ 50.73(a)(2)(viii)(A)
☐ 20.2203(a)(1)	☐ 20.2203(a)(4)	☐ 50.73(a)(2)(ii)(B)	☐ 50.73(a)(2)(viii)(B)
☐ 20.2203(a)(2)(i)	☐ 50.36(c)(1)(i)(A)	☐ 50.73(a)(2)(iii)	☐ 50.73(a)(2)(ix)(A)
☐ 20.2203(a)(2)(ii)	☐ 50.36(c)(1)(ii)(A)	☐ 50.73(a)(2)(iv)(A)	☐ 50.73(a)(2)(x)
☐ 20.2203(a)(2)(iii)	☐ 50.36(c)(2)	☐ 50.73(a)(2)(v)(A)	☐ 73.71(a)(4)
☐ 20.2203(a)(2)(iv)	☐ 50.46(a)(3)(ii)	☐ 50.73(a)(2)(v)(B)	☐ 73.71(a)(5)
☐ 20.2203(a)(2)(v)	☐ 50.73(a)(2)(i)(A)	☐ 50.73(a)(2)(v)(C)	☐ OTHER
☐ 20.2203(a)(2)(vi)	☒ 50.73(a)(2)(i)(B)	☐ 50.73(a)(2)(v)(D)	Specify in Abstract below or in NRC Form 366A

12. LICENSEE CONTACT FOR THIS LER

FACILITY NAME	TELEPHONE NUMBER (Include Area Code)
William D. Bartron, Supervisor, Nuclear Station Licensing	860-444-4301

13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX

14. SUPPLEMENTAL REPORT EXPECTED

☐ YES *(If yes, complete 15. EXPECTED SUBMISSION DATE)* ☒ NO

15. EXPECTED SUBMISSION DATE

MONTH	DAY	YEAR

ABSTRACT *(Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)*

On September 10, 2011, while operating in MODE 1 at 100% power, Millstone Power Station Unit 3 (MPS3) identified that a value recorded on August 11, 2011, for the Bandpass Filter Offset Voltage for one channel of MPS3's Gamma Metrics wide range nuclear instrumentation 3NME*AMPL2 was left outside of the acceptance criteria defined in Surveillance Procedure SP3441E01.

The condition was not noted on August 11, 2011, and the Technical Specification (TS) action statement was not entered. TS action statement 3.3.3.6 a (1) was entered upon discovery at 0025 on September 10, 2011. The surveillance procedure was successfully completed and the equipment was restored to operable status at 0425 on September 10, 2011.

This event is being reported pursuant to 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by the plant's Technical Specifications.

The cause of the failure was inadequate review of the calculated surveillance results. Subsequent to identification, further investigations revealed that the parameter was not necessary for operability of the equipment. The surveillance procedure has been revised to specify that measured offset voltage is not a TS acceptance criterion. Procedure changes being implemented will include independent reviews of calculated results on data sheets from I&C surveillances to ensure acceptance criteria are met prior to returning equipment to service.

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE
		YEAR	SEQUENTIAL NUMBER	REV NO.	
Millstone Power Station - Unit 3	05000423	2011	-- 001 --	00	2 OF 2

NARRATIVE

1. Event Description

On September 10, 2011, while operating in MODE 1 at 100% power, Millstone Power Station Unit 3 (MPS3) identified that a value recorded on August 11, 2011, for the Bandpass Filter Offset Voltage for one channel of MPS3 's Gamma Metrics wide range nuclear instrumentation (3NME*AMPL2) [MON] [IG] was left outside of the acceptance criteria defined in Surveillance Procedure SP3441E01 after the surveillance was completed. It was assessed at the time of discovery that this resulted in Accident Monitoring Instrumentation Instrument 19 "Neutron Flux" Channel 2 being inoperable per Plant Technical Specification (TS) 3.3.3.6.

The condition was not noted on August 11, 2011, and the action statement was not entered until September 10, 2011. Action statement 3.3.3.6.a was entered upon discovery at 0025 on September 10, 2011. The surveillance procedure was successfully completed and the equipment was restored to operable status at 0425 on September 10, 2011. TS action statement 3.3.3.6 a (1) requires with less than the required number of channels OPERABLE, restoration of the inoperable channel(s) to OPERABLE status within 7 days, or be in at least HOT STANDBY within the next 6 hours and in at least HOT SHUTDOWN within the following 6 hours.

This event is being reported pursuant to 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by the plant's Technical Specifications.

2. Cause

The condition of the recorded voltage being outside of the acceptance criteria was not noted until September 10, 2011 due to inadequate review of calculated surveillance results on data sheets.

3. Assessment of Safety Consequences

The safety consequences of this event were very low. Although the acceptance criteria was included in the station's Surveillance Procedure, the monitored parameter was subsequently found unnecessary to assure operability of the nuclear instrumentation.

4. Corrective Action

The surveillance procedure was successfully completed within four hours of identification of the condition, and the equipment was restored to operable status. The surveillance procedure has been revised to specify that measured offset voltage is not a TS acceptance criterion and extent of condition has been addressed.

Human performance aspects of this issue have been addressed through the corrective action program. Procedure changes being implemented will include independent reviews of calculated results on data sheets from I&C surveillances to ensure acceptance criteria are met prior to returning equipment to service.

Additional corrective actions are being taken in accordance with the station's corrective action program.

5. Previous Occurrences

No previous similar events were identified where the procedure was not followed when a Technical Specification surveillance acceptance criterion was not met, and prompt actions were not taken.

Energy Industry Identification System (EIIS) codes are identified in the text as [XX].