### **REQUEST FOR ADDITIONAL INFORMATION 864-6150 REVISION 3**

### 11/8/2011

## **US-APWR** Design Certification

### Mitsubishi Heavy Industries

Docket No. 52-021

## SRP Section: 15 - Introduction - Transient and Accident Analyses Application Section: 15.0

# QUESTIONS for Reactor System, Nuclear Performance and Code Review (SRSB)

#### 15-35

RAI 786-5881, Question 15.0.0-30 (b) asked for justification of the Doppler feedback assumptions for 15.2 and 15.3 events. The response stated that the Doppler feedback is not a key parameter for any of these events based on sensitivity studies. The staff notes that this response did not include sensitivities on the departure from nucleate boiling ratio (DNBR) for the limiting Anticipated Operational Occurrence (15.2.2) or the limiting Postulated Accident (15.3.3). Please provide the DNBR values for the sensitivity case for 15.2.2 and compare the number of rod failures for the sensitivity study on 15.3.3 to the DCD case. If the sensitivity case is more limiting than the DCD case, either include it in the DCD or justify not including it in the DCD.

#### 15-36

The response to RAI 769-5797 Question 15-26 proposed adding text to DCD Sections 15.1.4.2 and 15.1.4.3.2 to explain that the main steam line pressure signal used to actuate the ECCS function is a lead/lag compensated signal. However, as described in Table 15.1.4-1, the ECCS function for this event is actuated on low pressurizer pressure (not the lead/lag compensated main steam line pressure). Please explain why these statements are being added to Section 15.1.4.