

ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

REGULATOR INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9011050295 DOC. DATE: 90/10/29 NOTARIZED: NO DOCKET #
 FACIL: 50-263 Monticello Nuclear Generating Plant, Northern States 05000263
 AUTH. NAME AUTHOR AFFILIATION
 PENNINGTON, D. Northern States Power Co.
 PARKER, T.M. Northern States Power Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 90-015-00: on 900916, failed to perform required stroke timing of primary containment isolation valve. W/901029 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 5
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES: NRR/LONG, W. 05000263

	RECIPIENT		COPIES			RECIPIENT		COPIES	
	ID CODE/NAME		LTR	ENCL		ID CODE/NAME		LTR	ENCL
	PD3-1 LA		1	1		PD3-1 PD		1	1
	LONG, B.		1	1					
INTERNAL:	ACNW		2	2		AEOD/DOA		1	1
	AEOD/DSP/TPAB		1	1		AEOD/ROAB/DSP		2	2
	NRR/DET/ECMB 9H		1	1		NRR/DET/EMEB 7E		1	1
	NRR/DLPQ/LHFB11		1	1		NRR/DLPQ/LPEB10		1	1
	NRR/DREP/PRPB11		2	2		NRR/DST/SELB 8D		1	1
	NRR/DST/SICB 7E		1	1		NRR/DST/SPLB8D1		1	1
	NRR/DST/SRXB 8E		1	1		<u>REG FILE</u> 02		1	1
	RES/DSIR/EIB		1	1		RGN3 FILE 01		1	1
EXTERNAL:	EG&G BRYCE, J.H		3	3		L ST LOBBY WARD		1	1
	NRC PDR		1	1		NSIC MAYS, G		1	1
	NSIC MURPHY, G.A		1	1		NUDOCS FULL TXT		1	1

NOTE TO ALL "RIDS" RECIPIENTS:

PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK, ROOM P1-37 (EXT. 20079) TO ELIMINATE YOUR NAME FROM DISTRIBUTION LISTS FOR DOCUMENTS YOU DON'T NEED!

FULL TEXT CONVERSION REQUIRED
 TOTAL NUMBER OF COPIES REQUIRED: LTR 30 ENCL 30

Handwritten initials/signature



Northern States Power Company

414 Nicollet Mall
Minneapolis, Minnesota 55401-1927
Telephone (612) 330-5500

October 29, 1990

Report Required by
10 CFR Part 50, Section 50.73

Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Failure to Perform Required Stroke Timing of Primary
Containment Isolation Valve Following Valve Maintenance

The Licensee Event Report for this occurrence is attached.

Please contact us if you require additional information related to this event.

Thomas M Parker
Manager
Nuclear Support Services

c: Regional Administrator - III NRC
Sr Resident Inspector, NRC
NRR Project Manager, NRC
MPCA
Attn: Dr J W Ferman

Attachment

9011050295 901029
PDR ADOCK 05000263
S PNU

IE22
11

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)		DOCKET NUMBER (2)	PAGE (3)
MONTICELLO NUCLEAR GENERATING PLANT		0 1 5 0 0 0 2 1 6 1 3	1 OF 0 1 4

TITLE (4) Failure to Perform Required Stroke Timing of Primary Containment Isolation Valve Following Valve Maintenance

EVENT DATE (5)			LER NUMBER (6)		REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES											
0	9	1	6	9	0	9	0	0	0	1	5	0	0	0	0	0	0	0	0	0

OPERATING MODE (9) N

POWER LEVEL (10) 0 0 5

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(e)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(e)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(e)
<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(e)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	<input type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 365A)
<input type="checkbox"/> 20.406(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	
<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Dave Pennington	6 1 2 2 9 1 5 - 1 1 3 1 5 1 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Minor maintenance and subsequent post maintenance testing were performed on a primary containment automatic isolation valve. It was returned to service following this maintenance. Later, it was discovered that required stroke timing of the valve was not performed as a part of the post maintenance testing. This was a violation of Technical Specifications which require primary containment automatic isolation valves to be stroke timed prior to placing them back in service.

The cause of this event was inadequate administrative controls.

The corrective action taken was to stroke time the valve and review similar work packages. Actions taken or planned to prevent recurrence are to implement improved administrative controls and to remind plant personnel of the stroke timing requirement.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BUREAU PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BUREAU ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Monticello Nuclear Generating Plant	DOCKET NUMBER (2) 0 5 0 0 0 2 6 3 9 0	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9 0	- 0 1 5	- 0 0	0 2	OF 0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION

On September 16, 1990, the plant was in the process of starting up following a planned maintenance outage. A 600 psig reactor pressure inspection of the drywell was made by the operators at approximately 0800. A packing leak was found during this inspection on the main steam drain inboard isolation valve, MO-2373 (EIIIS Component Code: ISV), which is a primary containment automatic isolation valve. A work package was issued and the valve packing was tightened. Post maintenance testing was then performed on the valve which included electrically stroking the valve and verifying that the valve operator torque switch settings were within the acceptance band. The valve was left in its normally closed position following the tests. The work package for valve MO-2373 was completed at 1100 and plant startup then continued.

During the review of the completed work package for valve MO-2373 on September 28, 1990, the System Engineer realized that the valve was not stroke timed during post maintenance testing as is required by Technical Specification 4.7.D.3. This Technical Specification states that every primary containment automatic isolation valve must be cycled and its operating time verified before returning the valve to service following maintenance. The event is therefore reportable per 10CFR50.73, paragraph (a)(2)(i)(B).

CAUSE

The failure to stroke time valve MO-2373 following maintenance as required by Technical Specifications was attributed to inadequate procedural guidance.

Adequate administrative controls were not in place in the post maintenance testing program to flag this testing requirement. Per the existing plant administrative directives, the post maintenance tests selected are to be based on the extent of the maintenance work performed and shall verify the ability of the affected component or system to perform its intended function after maintenance. These requirements were believed to have been met by the post maintenance testing that was initially performed on valve MO-2373.

A contributing cause was cognitive personnel error. Engineering and operations personnel involved in either writing or reviewing the post maintenance testing requirements for valve MO-2373 failed to perform an adequate verification of post maintenance testing requirements which are specified in the Technical Specifications and in plant administrative directives. Memory of what was contained in these documents was relied upon in specifying the post maintenance tests.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Monticello Nuclear Generating Plant	DOCKET NUMBER (2) 0 5 0 0 0 2 6 3	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9 0	- 0 1 5	- 0 0	0 3	OF 0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

ANALYSIS

There was no threat to the health or safety of the public as a result of this event. The safety function of valve MO-2373 is to close in the event of an automatic isolation signal. The initial post maintenance testing performed on valve MO-2373 demonstrated that it would close. The valve was also closed immediately following initial post maintenance testing and remained closed except for subsequent testing. Per plant operating procedures, the valve is normally left in the closed position and opened only during plant shutdown or startup or for surveillance testing. In addition, the main steam drain outboard valve, MO-2374, which is located immediately downstream of MO-2373, was closed during all maintenance and testing of valve MO-2373. Therefore, isolation of the main steam drain line would have been achieved if it was required.

CORRECTIVE ACTION

The following corrective actions were taken or planned as a result of this event:

1. Valve MO-2373 was stroke timed as required by Technical Specifications and found to be within the acceptance band.
2. A review of work packages on containment isolation valves was completed and no other missed stroke time surveillance tests were identified.
3. Standard preventative maintenance procedures normally performed following valve maintenance were changed to flag the Technical Specification requirement for primary containment automatic isolation valve stroke timing.
4. Other means of flagging this requirement in the post maintenance testing program are also being investigated.
5. A written communication was sent to applicable plant employees reminding them of the Technical Specification requirement for stroke timing valves.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Monticello Nuclear Generating Plant	DOCKET NUMBER (2) 0 5 0 0 0 2 6 3	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9 0	- 0 1 5	- 0 0	0 4	OF 0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

ADDITIONAL INFORMATION

Failed Component Identification

None

Previous Similar Events

None