

# ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8911150124      DOC. DATE: 89/11/09      NOTARIZED: NO      DOCKET #  
 FACIL: 50-263 Monticello Nuclear Generating Plant, Northern States      05000263  
 AUTH. NAME      AUTHOR AFFILIATION  
 JACOBSON, R.      Northern States Power Co.  
 PARKER, T.M.      Northern States Power Co.  
 RECIP. NAME      RECIPIENT AFFILIATION

SUBJECT: LER 89-027-00: on 891010, inadequate review of circuit  
 isolation results in RWCU isolation valve closure.  
W/8      ltr.

DISTRIBUTION CODE: IE22T      COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4  
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

**NOTES:**

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	PD3-1 LA LONG, B.	1 1 1 1	PD3-1 PD	1 1
INTERNAL:	ACRS MICHELSON	1 1	ACRS MOELLER	2 2
	ACRS WYLIE	1 1	AEOD/DOA	1 1
	AEOD/DSP/TPAB	1 1	AEOD/ROAB/DSP	2 2
	DEDRO	1 1	NRR/DEST/ESB 8D	1 1
	NRR/DEST/ICSB 7	1 1	NRR/DEST/MEB 9H	1 1
	NRR/DEST/MTB 9H	1 1	NRR/DEST/PSB 8D	1 1
	NRR/DEST/RSB 8E	1 1	NRR/DEST/SGB 8D	1 1
	NRR/DLPQ/HFB 10	1 1	NRR/DLPQ/PEB 10	1 1
	NRR/DOEA/EAB 11	1 1	NRR/DREP/RPB 10	2 2
	NUDOCS-ABSTRACT	1 1	<u>REG FILE</u> 02	1 1
	RES/DSIR/EIB	1 1	<del>RGN3</del> FILE 01	1 1
EXTERNAL:	EG&G WILLIAMS, S	4 4	L ST LOBBY WARD	1 1
	LPDR	1 1	NRC PDR	1 1
	NSIC MAYS, G	1 1	NSIC MURPHY, G.A	1 1
	NUDOCS FULL TXT	1 1		

*A0-4*

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Northern States Power Company

414 Nicollet Mall  
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Telephone (612) 330-5500

November 9, 1989

Report Required by:  
10 CFR Part 50  
Section 50.73

U S Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT  
Docket No. 50-263 License No. DPR-22

Inadequate Review of Circuit Isolation  
Results in Reactor Water Cleanup Isolation Valve Closure

The Licensee Event Report for this occurrence is attached.

This event was reported via the Emergency Notification System in accordance with 10 CFR Part 50 Section 50.72 on October 10, 1989.

Thomas M Parker  
Manager  
Nuclear Support Services

c: Regional Administrator - III, NRC  
NRR Project Manager, NRC  
Resident Inspector, NRC  
MPCA  
Attn: J W Ferman

Attachment

8911150124 891109  
PDR ADDOCK 05000263  
S FIC

IE22  
11

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST - 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Monticello Nuclear Generating Plant	DOCKET NUMBER (2) 0 5 0 0 0 2 6 3	PAGE (3) 1 OF 0 3
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TITLE (4) Inadequate Review of Circuit Isolation Results  
in Reactor Water Cleanup Isolation Valve Closure

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)										
1	0	1	0	8	9	8	9	0	2	7	0	0	1	1	0	9	8	9		0 5 0 0 0 0

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)										
POWER LEVEL (10) 0 0 0	20.402(b)			20.405(c)			<input checked="" type="checkbox"/> 50.73(a)(2)(iv)			73.71(b)	
	20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)			73.71(c)	
	20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
	20.405(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii)(A)				
	20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)				
	20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)				

LICENSEE CONTACT FOR THIS LER (12)

NAME Ronald Jacobson	TELEPHONE NUMBER
	AREA CODE: 6 1 2    2 9 5 - 1 3 0 2

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During performance of a modification procedure the Reactor Water Cleanup Hi Filter-Demineralizer Inlet Temperature circuit was de-energized due to momentarily lifting a wire lead from a terminal block to allow termination of a second wire lead. This resulted in closure of the Reactor Water Cleanup primary containment isolation valves. The wire leads were reterminated, the isolation was reset, and Reactor Water Cleanup returned to service. The plant was shutdown for refueling at the time of the event. The root cause of this event is a cognitive personnel error. Inadequate attention was given to other circuits that could be affected by the modification isolation. The need to review redundant drawings and adequately verify as-built configuration for circuit isolations involving lifted leads or jumpers will be emphasized during training.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  Monticello Nuclear Generating Plant	DOCKET NUMBER (2)  0   5   0   0   0   2   6   3	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8   9	-   0   2   7	-   0   0	0   2	OF	0   3

TEXT (If more space is required, use additional NRC Form 365A's) (17)

DESCRIPTION

During performance of modification procedure 89-90290 "Outage Related Electrical Work Required for Reactor Building Chemistry Sampling System" the Reactor Water Cleanup (RWCU) (EIIS System Code: CE) Hi Filter-Demineralizer Inlet Temperature circuit (EIIS Component Identifier Code: TC) was de-energized due to momentarily lifting a wire lead from a terminal block to allow termination of a second wire lead. This resulted in closure of the RWCU primary containment isolation valves. The wire leads were reterminated, the isolation was reset, and RWCU returned to service. The total elapsed time of the isolation was five minutes. The plant was in a refueling outage during the event.

CAUSE

During the restoration of circuits that had been isolated for a modification, a wire in the RWCU high temperature isolation circuit was momentarily lifted from one of the terminal points. This caused the isolation.

The root cause of this event is a cognitive personnel error. Inadequate attention was given to other circuits that could be affected by the modification isolation. The RWCU Instrument Connection and the RWCU External Connection drawings were reviewed by the Shift Supervisor and an engineer prior to approving the isolation. The drawings reviewed did not include internal connections shown on the RWCU Internal Connection drawing. This drawing shows the high temperature isolation circuit. Work Instruction 4AWI 4.5, Equipment Control Procedure, in Section 6.2 (Preparation, Review, Authorization of Isolations) requires As-built configuration verification for electrical isolations. Compliance is achieved either by (1) review of separate, independent drawings which are in the plant drawing files or (2) by doing a physical As-built verification. What was neglected for this additional isolation review was a more thorough review of separate drawings (i.e., the RWCU Internal Connection drawing) or a physical As-Built verification. There were no unusual characteristics of the work location that directly contributed to the error.

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8   9	—   0   2   7	—   0   0	0   3	OF	0   3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

ANALYSIS

The only safety implication is an unnecessary operation of ESF equipment. There was no impact on reactor water quality since the reactor was in cold shutdown and the RWCU system was returned to normal operation in five minutes.

It is not likely this event would have occurred during power operation as the need to lift the wires was directly related to a plant modification which could only be done with the plant in a cold shutdown condition. However, if the event had occurred at power operation, the impact would still have been negligible, resulting in loss of RWCU flow for five minutes or less.

CORRECTIVE ACTION

The need to review redundant drawings and adequately verify as-built configuration for circuit isolations involving lifted leads or jumpers will be emphasized during training.

ADDITIONAL INFORMATION

Failed Component Identification :

None.

Previous Similar Events :

None.