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ACCELERATED

# DISTRIBUTION DEMONSTRATION

# SYSTEM

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

FACIL:50	AME AUTHOR ,E. Northern D. Northern	uclear Gener	r Co.	: NO rn States	DOCKET # 05000263
SUBJECT			e assumption of tes noperative process	monitor.	n R r. I
	UTION CODE: IE22D 50.73 Licensee Eve		IVED:LTR   ENCL   ER), Incident Rpt,	size:	D
NOTES:					S
INTERNAL:	RECIPIENT ID CODE/NAME PD3-3 LA SCALETTI,D ACRS MICHELSON AEOD/DOA AEOD/DSP/ROAB ARM/DCTS/DAB NRR/DEST/ADS NRR/DEST/ADS NRR/DEST/ELB NRR/DEST/PSB NRR/DEST/PSB NRR/DEST/SGB NRR/DEST/SGB NRR/DLPQ/QAB NRR/DLPQ/QAB NRR/DREP/RAB NRR/DREP/RAB NRR/DREP/RAB NRR/DRES/SIB REG FILE 02 RES TELFORD,J RGN3 FILE 01	COPIES LTTR ENCL 1 1 1 1 1 1 2 2 1 1 1 2 2 2 1 1 1 1 1 1	RECIPIENT ID CODE/NAME PD3-3 PD ACRS MOELLER AEOD/DSP/NAS AEOD/DSP/TPAB DEDRO NRR/DEST/CEB NRR/DEST/ICSB NRR/DEST/MTB NRR/DEST/RSB NRR/DEST/RSB NRR/DLPQ/HFB NRR/DLPQ/HFB NRR/DCEA/EAB NRR/DCEA/EAB NRR/DREP/RPB NRR/DREP/RPB NRR/PMAS/ILRB RES DEPY GI RES/DE/EIB	COPIES LTTR ENCI 1 1 2 2 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1	A D D S
EXTERNAL:	EG&G GROH,M H ST LOBBY WARD NRC PDR NSIC MAYS,G	5 5 1 1 1 1 1 1	FORD BLDG HOY,A LPDR NSIC HARRIS,J	1 1 1 1 1 1	R I D S

TOTAL NUMBER OF COPIES REQUIRED: LTTR 46 ENCL 45 D S

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NRC Form 366	U.S. NUCLEAR REGULATORY COMMISSION
(9-83)	Approved OMB NO. 3150-0104
	REPORT (LER) Expires 8/31/85
FACILITY NAME (1)	DOCKET NUMBER (2)   PAGE (3)
Monticello Nuclear Generating Plant	
TITLE (4) False Assumption of Test	
Causes Missed Sampling for an	
	REPORT DATE (7) OTHER FACILITIES INVOLVED (8)
	MONTH DAY YEAR FACILITY NAMES DOCKET NUMBERS
I I I NUMBER I NUMBER	
OPERATING    THIS REPORT IS SUBMITTED PURS	SUANT TO THE RÉQUIREMENTS
MODE (9)  N OF 10 CFR §: (Check one or n	nore of the following)(11)
	5(c)   50.73(a)(2)(iv)   73.71(b)
	(c)(1) $1 = 50.73(a)(2)(v)$ $1 = 73.71(c)$
$ (10) ^{0} ^{0} ^{0} ^{1} ^{20.405(a)(1)(ii)} ^{1} ^{50.366}$	
	(a)(2)(i)     50.73(a)(2)(vii)(A) (Specify in)
	(a)(2)(ii)     50.73(a)(2)(viii)(B) Abstract
$  \qquad   20.405(a)(1)(v) \qquad   50.73(a)(1)(v) \qquad   10000000000000000000000000000000000$	(a)(2)(iii)   50.73(a)(2)(x) below and
	in Text, NRC Form 366A)
	CT FOR THIS LER (12)
NAME	TELEPHONE NUMBER
	AREA CODE
Ellis Pfeffer, Radiation Protection Sp	Decialist   6  1 2   2  9  5 - 5 1 5 1
	F FAILURE DESCRIBED IN THIS REPORT (13)
MANUFAC   REPORTABLE	MANUFAC   REPORTABLE
CAUSE   SYSTEM   COMPONENT   TURER   TO NPRDS	CAUSE SYSTEM COMPONENT  TURER   TO NPRDS
SUPPLEMENTAL REPORT EXPECTED	
	SUBMISSION
YES (If yes, complete EXPECTED SUBMISSION	N DATE)    NO   DATE (15)
ABSTRACT (Limit to 1400 spaces, i.e., approx	ximately fifteen single-space typwritten lines)
(16)	
During a routine plant shutdown and refu	eling outage, the Service Water Discharge
	RM) was found to have been in an inoperable
condition without grab samples being col	lected for longer than 8 hours time.
The missed sample was due to personnel e	
Protection Specialist (RPS) requested co	ompletion of the monitor operability test, but
did not verify that a functional test re	equired to clear an inoperable condition on the
	, before he secured his sampling routine.
	, r0 , , , ,
After investigation of the circumstances	the RPS realized that informal
	Les to be missed. He then resumed 8 hour
sampling. The situation was discussed in	
counseled concerning proper shift turnov	
	for possible revision to assist in preventing
future occurrences of this nature.	
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NRC Form 366 (9-83)	n 366 U.S. NUCLEAR REGULATO Approved OMB NO.			
LICENSEE EV	ENT REPORT (LER) TEXT C	CONTINUATION Expires 8/31/8	5	
  FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)	
		YEAR   SEQUENTIAL   REVISION		
Monticello Nuclear	1	NUMBER   NUMBER		
Generating Plant	0500263	87-0024-00	0 2 0F  0 3	
TEXT (If more space is re	quired, use additional	NRC Form 366A's) (17)		

#### DESCRIPTION

On November 21, 1987, at 2115 hours Central Standard Time (CST), it was discovered that the Service Water Discharge Pipe Gross Radioactivity Monitor (SWDPGRM) had been in an inoperable condition for eighteen and one-half hours without a grab sample being collected. The Reactor was in the shutdown mode and was shutdown at the time of the event. Technical Specifications Table 3.8.1 specifies that when the minimum channels are inoperable service water discharge may continue for up to 30 days provided that at least once every 8 hours a grab sample is collected and analyzed for gross beta and gamma radioactivity at an LLD of 1.0E-7 uci/ml.

Surveillance Test #346b, Liquid Process Monitor Grab Sampling, had been initiated at 0400 hours CST on November 10, 1987, due to loss of the Service Water Sample Pump. At 0300 hours CST on November 21, 1987, a Service Water Sample was collected and analyzed. Results were less than the LLD of 1.0E-7 uci/ml.

At 1030 hours CST, the 8 hour grab sample requirements had been discontinued. It was assumed that requested operability testing requirements had been successfully completed at this time and that the SWDPGRM was fully operable. The mistake was discovered at 2115 hours CST when the Shift Chemistry Radiation Protection Specialist (RPS), during the course of a discussion with a licensed operator, realized that the monitor was not fully operative. An investigation of the monitor status verified this condition. The Shift Chemistry RPS resumed 8 hour sampling requirements at 2130 hours CST that evening. A sample was collected and analyzed with the results less than the LLD of 1.0E-7 uci/ml. A review of discharge canal monitors verified that no discharge of radioactive liquids had occurred during the missed sample interval.

#### CAUSE

The root cause of this event was personnel error. There was no unusual characteristics of the work location that directly contributed to the error. It did not result from an error in an approved procedure or non-adherence. After maintenance activities had been completed on the 24 VDC system on November 21, 1987, the Plant Operations group completed Part I (the front panel checks) of Surveillance Test #0289a, the functional test for the Service Water Monitor. The Shift Chemistry RPS then performed Part II (radiation response check) of same test and returned the test to the Operations group with the understanding that Part III (sample flow verification) would be completed by Operations. Part III was not completed until the morning of November 24, 1987. It was assumed that completion of this test had returned the Service Water Monitor to an operable condition. This information was passed on at the Shift Chemistry RPS turnover that morning and the 8 hour grab sampling requirements were discontinued. This was a cognitive error in that the RPS failed to verify that the system was fully operable and was imprecise in communicating the details of the in-progress test to his relief.

NRC Form 366  (9-83)			U.S. NUCLEAR REGULATORY COMMISSION   Approved OMB NO. 3150-0104				
	LICENSEE	EVENT REPORT (LER) TEXT	CONTINUATION Expires 8/31/85				
FACILITY NAME	(1)	DOCKET NUMBER (2	)   LER NUMBER (6)	PAGE (3)			
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 Generating Plant
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 ITEXT (If more space is required, use additional NRC Form 366A's) (17)

#### ANALYSIS

Missing two required eight-hour samples while the service water monitor was inoperable potentially would have delayed recognition had a leakage of low level radioactivity into the service water system occurred. The Discharge Canal Liquid Process Monitor was operable and would have detected significant activity released to the discharge canal. The samples collected and analyzed immediately prior to and after the incident have been less than LLD of 1.0E-7 uci/ml. This event could not have had more severe consequences regardless of initial conditions.

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#### CORRECTIVE ACTION

The situation was discussed with the RPS who made the personnel error. Operations Daily Log Part D will be revised to include a requirement to perform a once per shift alarm check on Service Water sample flow (SWSF) to alert the Shift Chemistry RPS should the SWDPGRM become inoperable due to loss of sample flow. Surveillance Test #0289a will be revised so that the Operations group performs the front panel checks and sample flow checks as Part I and II. The RPS will then perform the radiation response check as Part III. Operations Manual Section B.5.11 will be revised to include an annuciator response procedure for loss of SWSF and guidance to operations personnel in the event operability requirements are not satisfied. This event, along with the proposed revisions, will be discussed with all members of the chemistry group at the next weekly group meeting and implemented as part of scheduled operations group training.

### ADDITIONAL INFORMATION

Failed Component Identification

None

Previous Similar Events

None

NRC FORM 366A (9-83)



#### Northern States Power Company

414 Nicollet Mall Minneapolis. Minnesota 55401 Telephone (612) 330-5500

December 21, 1987

Report Required by 10 CFR Part 50, Section 50.73

US Nuclear Regulatory Commission Attn: Document Control Desk Washington DC 20555

> MONTICELLO NUCLEAR GENERATING PLANT Docket No. 50-263 License No. DPR-22

False Assumption of Test Completion Causes Missed Sampling for an Inoperative Process Monitor

The Licensee Event Report for this occurence is attached.

Thouca Vike

David Musolf Manager - Nuclear Support Services

c: Regional Administrator-III, NRC NRR Project Manager, NRC Resident Inspector, NRC MPCA Attn: J W Ferman

Attachment

