

ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8712140108 DOC. DATE: 87/12/11 NOTARIZED: NO DOCKET #
 FACIL: 50-263 Monticello Nuclear Generating Plant, Northern States 05000263
 AUTH. NAME AUTHOR AFFILIATION
 ANTONY, D.D. Northern States Power Co.
 MUSOLF, D. Northern States Power Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 87-019-00: on 871111, ESF actuations occurred due to
 misplaced jumper during surveillance test. W/8 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED: LTR 1 ENCL 1 SIZE:
 TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

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	AEOD/DSP/ROAB	2	2	AEOD/DSP/TPAB	1	1
	ARM/DCTS/DAB	1	1	DEDRO	1	1
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	NRR/DREP/RAB	1	1	NRR/DREP/RPB	2	2
	NRR/DRIS/SIB	1	1	NRR/PMAS/ILRB	1	1
	<u>REG FILE</u> 02	1	1	RES DEPY GI	1	1
	RES TELFORD, J	1	1	RES/DE/EIB	1	1
	RGN3 FILE 01	1	1			
EXTERNAL:	EG&G GROH, M	5	5	FORD BLDG HOY, A	1	1
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LICENSEE EVENT REPORT (LER)

Expires 8/31/85

FACILITY NAME (1) | DOCKET NUMBER (2) | PAGE (3)
Monticello | 0 5 0 0 0 2 | 6 3 1 | OF 0 3

TITLE (4)
Engineered Safety Feature Actuations Due to Misplaced Jumper During Surveillance Test

EVENT DATE (5) | LER NUMBER (6) | REPORT DATE (7) | OTHER FACILITIES INVOLVED (8)

MONTH | DAY | YEAR | YEAR | SEQUENTIAL | REVISION | MONTH | DAY | YEAR | FACILITY NAMES | DOCKET NUMBERS
NUMBER | NUMBER | | | | | | |

1 | 1 | 1 | 1 | 8 | 7 | 8 | 7 | - | 0 | 1 | 9 | - | 0 | 0 | 1 | 2 | 1 | 1 | 8 | 7 | 0 | 5 | 0 | 0 | 0 |

OPERATING | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS
MODE (9) | N OF 10 CFR §: (Check one or more of the following)(11)

POWER | 20.402(b) | 20.405(c) | X | 50.73(a)(2)(iv) | 73.71(b)
LEVEL | 20.405(a)(1)(i) | 50.38(c)(1) | | 50.73(a)(2)(v) | 73.71(c)
(10) | 0 | 0 | 0 | 20.405(a)(1)(ii) | 50.36(c)(2) | | 50.73(a)(2)(vii) | OTHER
| 20.405(a)(1)(iii) | 50.73(a)(2)(i) | | 50.73(a)(2)(viii)(A) (Specify in
| 20.405(a)(1)(iv) | 50.73(a)(2)(ii) | | 50.73(a)(2)(viii)(B) Abstract
| 20.405(a)(1)(v) | 50.73(a)(2)(iii) | | 50.73(a)(2)(x) below and
in Text, NRC Form 366A)

LICENSEE CONTACT FOR THIS LER (12)

NAME | TELEPHONE NUMBER
Douglas D. Antony, General Superintendent, Operations | 6 1 2 2 9 5 - 5 1 5 1 |
AREA CODE |

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC	REPORTABLE	CAUSE	SYSTEM	COMPONENT	MANUFAC	REPORTABLE
TURER			TURER	TO NPRDS	TURER			TURER	TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

EXPECTED | MONTH | DAY | YEAR |
SUBMISSION |
DATE (15) | | | |

YES (If yes, complete EXPECTED SUBMISSION DATE) | X | NO

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines)
(16)

While performing a Spent Fuel Pool Monitor monthly functional test, a primary containment group II isolation, a secondary containment isolation and a standby gas treatment system initiation occurred due to a high level trip of Fuel Pool Monitor channel B. The trip of the B channel is a planned part of the surveillance test; however, the procedure has a step for installing a jumper to prevent these Engineered Safety Feature (ESF) actuations. The licensed operator assisting with the test inadvertently placed the jumper across terminals B53 and B54 in the logic panel rather than terminals B53 and B55 as called for in the procedure. This allowed the ESF actuations to occur when the upscale trip check was performed. The procedure did not require independent verification of jumper placement. The ESF actuations were immediately reset and the test was completed. An administrative hold was placed on the procedure to assure independent verification of jumper placement. A modification was made that eliminated the need for using jumpers for this routine test.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)						PAGE (3)				
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER								
Monticello	05000263	87	-	0	1	9	-	0	0	02	OF	03

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION

On November 11, 1987, at 1050 hours central standard time, a partial primary containment group II isolation (JM), a secondary containment isolation and a standby gas treatment system (BH) initiation occurred while performing a Spent Fuel Pool Radiation Monitor functional test. The reactor was in the REFUEL mode and was shutdown at the time of the event. The Engineered Safety Feature (ESF) actuations occurred when the upscale trip check was performed on the B channel of the fuel pool monitor. The test procedure has a step for installing a jumper to prevent the ESF actuations; however, it did not require independent verification of jumper placement. The licensed operator assisting with the test inadvertently placed the jumper across terminals B53 and B54 in the logic panel rather than across terminals B53 and B55 as specified in the procedure. The ESF actuations were annunciated in the main control room when the upscale trip check was performed. The licensed operators determined the cause of the ESF actuations by 1051 hours on November 11, 1987. The spent fuel pool monitor upscale trip was reset and the ESF actuations were reset by 1055 hours on November 11, 1987. The monitor functional test was completed without further complications.

CAUSE

The root cause of this event is procedural inadequacy. A similar event occurred on March 7, 1987, (Reference LER 87-008) and a program was initiated to identify and revise routine surveillance procedures to require independent verification of jumper placement. The procedure involved in this event had been identified, but had not yet been revised. The lack of an independent verification allowed a single human mistake to result in the ESF actuations. A contributing cause of this event is poor human engineering. The Spent Fuel Pool Monitoring System was designed such that it required placement of jumpers for completion of routine surveillance tests.

ANALYSIS

This event had no affect on public health or safety since this was an inadvertent actuation and the safety function of the affected systems was initiated. The systems that were actuated functioned properly. This event could not have had more severe consequences regardless of initial conditions.

CORRECTIVE ACTION

An administrative hold was placed on the procedure to assure it was not used without a specific sign-off for jumper placement verification. Administrative holds were placed on other procedures which had been identified as having similar deficiencies. A modification was completed that eliminated the need for use of jumpers for routine surveillance testing of the spent fuel pool monitors. Similar modifications were completed on the Reactor Building Wide Range Gas Monitors and the Reactor Building Vent monitors.

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Monticello	05000263	87	019	00	03 OF 03

TEXT (If more space is required, use additional NRC Form 366A's) (17)

ADDITIONAL INFORMATION

Failed Component Identification

None.

Previous Similar Events

There was one previous similar event, however, it was associated with the Reactor Building Wide Range Gas Monitors - not the Fuel Pool Monitors. Reference LER 87-008.



Northern States Power Company

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Minneapolis, Minnesota 55401
Telephone (612) 330-5500

December 11, 1987

Report Required by
10 CFR Part 50, Section 50.73

U S Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Engineered Safety Feature Actuations Due to
Misplaced Jumper During Surveillance Test

The Licensee Event Report for this occurrence is attached.

This event was reported via the Emergency Notification System in accordance with 10 CFR Part 50, Section 50.72, on October 29, 1987.

for Monica Vick
David Musolf
Manager - Nuclear Support Services

c: Regional Administrator-III, NRC
NRR Project Manager, NRC
Resident Inspector, NRC
MPCA
Attn: J W Ferman

Attachment

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