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 Office of Nuclear Reactor Regulation, Director

SUBJECT: Submits revised info re implementation of Mark I
 containment long-term program mods at facility. All required
 Mark I containment program analyses & mods completed.

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September 17, 1985

Director
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MONTICELLO NUCLEAR GENERATING PLANT
DOCKET NO. 50-263 LICENSE NO. DPR-22

Revised Wetwell-to-Drywell Vacuum Breaker
Valve Evaluation and Completion of Mark I
Containment Program Improvements at Monticello

Reference: Letter dated April 15, 1983 from D M
Musolf, NSP, to Director, NRR, USNRC,
Evaluation of the Monticello Wetwell-
to-Drywell Vacuum Breaker Valves"

The purpose of this letter is to provide, for the information of the NRC Staff, revised information related to implementation of Mark I Containment Long Term Program modifications at the Monticello Nuclear Generating Plant. This information supplements our submittal of April 15, 1983 referenced above which responded to the NRC information request contained in Generic Letter 83-08, "Modification of Vacuum Breakers on Mark I Containments."

Subsequent to our April 15, 1983 submittal, Continuum Dynamics, Incorporated, reevaluated the wetwell-to-drywell vacuum breakers for the chugging and condensation oscillation phases of a loss of coolant accident (CDI Technical Note 84-21, January, 1985). As a result, the predicted maximum impact velocity was increased from 3.92 to 4.90 rad/sec. Nutech Engineers, Incorporated was requested to evaluate vacuum breaker component stresses which result from this increased impact velocity. Nutech concluded that the existing vacuum breaker design, with the upgrades described in our April 15, 1983 submittal, was acceptable when evaluated for the revised impact velocity of 4.90 rad/sec. Therefore, no further valve modifications were needed.

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Director of NRR
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All required Mark I Containment Program analyses and modifications have now been completed. Detailed descriptions of this work were included in the Monticello Long Term Program Plant Unique Analysis Report submitted to the NRC on December 15, 1982 and the Monticello Long Term Program Plant Unique Analysis Report for Torus Attached Piping submitted to the NRC on November 4, 1983. The requirements of the NRC Order for Modification of License and Extension of Exemption dated January 13, 1981 as modified on January 19, 1982 have been satisfied.

Following completion of major structural modifications in 1982, the wetwell-to-drywell differential pressure was removed. The blower system was left in place, however, for periodic vacuum breaker leakage surveillance testing required by the Technical Specifications.

Please contact us if you have any questions concerning the revised vacuum breaker analysis or any of the actions we have taken to complete the Mark I Containment Long Term Program at the Monticello Nuclear Generating Plant.



David Musolf
Manager - Nuclear Support Services

c: Regional Administrator-III, NRC
Resident Inspector, NRC
NRR Project Manager, NRC
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