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 Office of Nuclear Reactor Regulation, Director

SUBJECT: Forwards Rev 1 to "Sys Description Safety Relief Valve
 Blowdown Control Sys for Monticello Nuclear Generating
 Plant." Rept describes control sys mods to be implemented
 during 1984 refueling outage.

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June 13, 1983

Director of Nuclear Reactor Regulation
U S Nuclear Regulatory Commission
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Modifications to Safety/Relief Valve Control
System to Limit Containment Hydrodynamic Loads

Our letter dated December 15, 1982 transmitted for NRC Staff review the Mark I Containment Long Term Program Plant Unique Analysis Report for the Monticello Nuclear Generating Plant. In that letter we stated that credit was taken for modifications to the safety/relief valve (SRV) control system to limit containment hydrodynamic loads resulting from clearing an elevated water leg from a safety/relief valve discharge line. The purpose of this letter is to transmit, for your information, a description of this modification and an evaluation of the effects of the modified system on existing plant systems.

Attached is a report prepared by NUTECH Engineers, Incorporated, entitled "System Description, SRV Blowdown Control System for Monticello Nuclear Generating Plant", NSP-48-1103, Revision 1, May 18, 1983. This report describes the control system modifications we plan to implement during the 1984 refueling outage as part of our Mark I Containment Long Term Program minor modification package. This schedule is consistent with the January 19, 1982 Order for Modification issued by the Commission.

As noted in the report, this modification does not affect existing Technical Specification limits or the safety related functions of the safety/relief valves. The modification will be accomplished in accordance with the provisions of 10 CFR Part 50, Section 50.59. Proposed Technical Specification changes to add limiting conditions for operation and surveillance requirements for this system will be proposed following implementation of this modification. Additional Technical Specification changes will also be proposed at that time to eliminate minor inconsistencies between the Plant Unique Analysis Report and the Technical Specifications.

Please contact us if you have any questions related to the planned SRV blowdown control system.

David Musolf
Manager - Nuclear Support Services

DMM/dab

cc: Regional Administrator - III
NRR Project Manager, NRC
Resident Inspector, NRC
G Charnoff

Attachment

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