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 Office of Nuclear Reactor Regulation, Director

SUBJECT: Forwards plant-unique analysis repts Vols 1-5 & App A, Repts describe analyses conducted for suppression chamber vent sys, suppression chamber internals & safety relief valve discharge piping. *566 RPTs*

DISTRIBUTION CODE: A0255 COPIES RECEIVED: LTR 1 ENCL 40 SIZE: 2+927
 TITLE: OR Submittal: USI A-7 Mark I Containment

NOTES:

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December 15, 1982

Director
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U S Nuclear Regulatory Commission
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Submittal of the Monticello Mark I Containment Long Term
Program Plant Unique Analysis Report

- References:
- (1) NRC Order for Modification of License and Extension of Exemption dated January 13, 1981 as modified January 19, 1982
 - (2) General Electric Letter, June 29, 1981, "Mark I Owners Group Status Summary Report"
 - (3) NUREG-0661, "Safety Evaluation Report, Mark I Containment Long-Term Program", July, 1980
 - (4) NRC Meeting Notes (October 12, 1982) for the September 9 and 10, 1982 Meeting with General Electric and the Mark I Owners Group

Attached are forty copies of Volumes I through V, including Appendix A, of the Monticello Long Term Program Plant Unique Analysis Report (LTP-PUAR). This report is intended to satisfy the requirements of Reference (1) in accordance with the schedule established in Reference (2).

The Monticello LTP-PUAR describes the analyses which have been conducted for the Monticello suppression chamber, vent system, suppression chamber internals and safety relief valve discharge piping. The Staff's criteria set forth in Reference (3) have been used in these analyses as a basis for acceptance of analysis methods and results.

In addition, the LTP-PUAR includes an appendix to address the licensing inquiries that were generated during the Staff's recent review of other plant unique analyses. This appendix was included in the LTP-PUAR in anticipation of the applicability of these issues to the Monticello plant.

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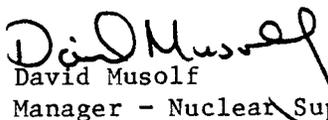
The LTP-PUAR takes credit for a modification to the safety relief valve (SRV) control system to limit the containment hydrodynamic loads due to SRV operations. NSP is currently finalizing the design of the modifications to the SRV control system and will submit a report to the Staff when it has been completed. The report will include a description of the modifications and an evaluation of the effects of the modified system on the over-pressure protection analysis. The results and conclusions of the enclosed LTP-PUAR will not be affected by the modification which will be completed by the end of the 1983 refueling outage.

There are additional requirements provided in NUREG-0661 that will also be addressed by NSP in separate reports. These requirements include the analyses of torus attached piping and the vacuum breaker valves between the torus and the drywell. The schedule for submittal of the report on the torus attached piping analysis and modifications is provided in Reference (2). This report will also address operability of pumps and valves. The report on the vacuum breaker analysis and modification will be submitted after the NRC Staff has completed its review of the additional information requested at the September 9-10, 1982 Mark I Owners Group Meeting (Reference 4). However, in a parallel effort, NSP has proceeded to modify the valves to ensure that the results of the final analysis will demonstrate the capability of the valves to perform their safety function under accident conditions. The modifications have included replacing selected valve components, including the disc, with components made from higher strength materials and securing vacuum breaker parts.

Requirements for limiting chugging duration following a Small-Break Accident to 10 minutes are being addressed in the BWR Owner's Group Emergency Procedure Guidelines (EPG's) and will be implemented following approval of the EPG's by the NRC Staff.

As required by NUREG-0661, the General Electric supplied suppression pool temperature transients for the various stuck open SRV events, isolation events, and small break accidents have been included in Section 5 of Volume 1 of the LTP-PUAR. Also as required by NUREG-0661, a description of the upgraded Suppression Pool Temperature Monitoring System (SPTMS) has been included. One channel of SPTMS is now operable and a fully redundant system will be available by the end of the 1983 refueling outage.

Please contact us if you have any questions concerning the Monticello LTP-PUAR.


David Musolf
Manager - Nuclear Support Services

DMM/bd

cc: Regional Administrator, Region III, NRC
NRR Project Manager, NRC
NRC Resident Inspector
G Charnoff

Attachments