REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

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RECIP.NAME RECIPIENT AFFILIATION
Office of Nuclear Reactor Regulation, Director

SUBJECT: Provides info re implementation of requirements in NUREG-0619, "BWR Feedwater Nozzle & Control Rod Drive Return Line Nozzle Cracking." All implementation requirements are scheduled for completion by June 1982.

DISTRIBUTION CODE: A024S COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 3
TITLE: BWR Feedwater Nozzles/Spargers or Control Rod Drive Return (USI A-10)

NOTES:

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NSP

NORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401

January 28, 1981

Director of Nuclear Reactor Regulation
U S Nuclear Regulatory Commission
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

BWR Feedwater Nozzle and Control Rod Drive
Return Line Nozzle Cracking - NUREG-0619

On December 8, 1980 we received a letter from Mr D G Eisenhut, Director, Division of Licensing, Office of Nuclear Reactor Regulation dated November 13, 1980, which transmitted the revised and final edition of NUREG-0619, "BWR Feedwater Nozzle and Control Rod Drive Return Line Nozzle Cracking". We were requested to furnish confirmation that the implementation dates indicated in NUREG-0619 will be met or to furnish justification for delays.

Implementation requirements for feedwater nozzles specified in Section 4.4 of NUREG-0619 will be met in the following manner:

1) An analysis will be performed to determine if the existing low flow controller meets the intent of the requirements (i.e., limit crack growth to ASME Section XI allowables during the reactor vessel life time). If the existing controller is found satisfactory no further actions in this regard will be performed. If the analysis indicate that the existing system does not meet the intent of the requirements, appropriate modifications will be performed prior to June 30, 1983.

2) Welded-in thermal sleeves and spargers with top mounted elbows will be installed during a maintenance outage scheduled for Fall, 1981. This configuration has been reviewed and accepted by the NRC staff. An inspection plan for this configuration will be proposed in the post-modification report described in paragraph 4.4.3.1.

3) The RWCU system was modified during the Fall, 1980 refueling outage to distribute return flow evenly to all four feedwater nozzles. The return line was rerouted in the steam chase outside of primary containment to provide return flow to both feedwater injection lines. Return line flow resistance was controlled to assure an even split of flow to both feedwater injection lines. Test results were submitted with NSP's comments dated July 11, 1980 concerning draft NUREG-0619. Additional descriptive information concerning this modification is not believed to be necessary.

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4) Operating procedures have been reviewed and changes have been made, where appropriate, to enhance the feedwater nozzle environment.

5) An on-line thermal sleeve leakage monitoring system was installed during the Fall, 1978 refueling outage. This system was described in detail at a meeting with the NRC staff on February 21, 1980. It is not planned to submit additional descriptive information concerning this system.

6) Reports will be provided as otherwise described in paragraphs 4.4.3.1.(1) and 4.4.3.1.(2).

Implementation requirements for CRD return line nozzles specified in Section 8 of NUREG-0619 will be met with the exception of Section 8.1.4.c'. We believe replacement of carbon steel piping in the flow stabilizer loop is not necessary for the Monticello facility. As noted in the Monticello Annual Report of Changes, Tests, and experiments for 1979 (letter dated February 28, 1980 from L O Mayer, NSP, to J G Keppler, Director, Region III, Office of Inspection & Enforcement, USNRC), the CRD stabilizing assembly has been isolated. Tests conducted in April, 1978 verified that isolation of the stabilizing assembly has a negligible affect on normal CRD insertion and withdrawal. The stabilizing assembly is completely independent of the CRD scram subsystem and has no affect on the scram function. All other implementation requirements are scheduled for completion before startup following the 1982 refueling outage which is now scheduled for June, 1982.

Please contact us if you have any questions concerning our committments to resolve the issue of feedwater and CRD return line nozzle cracking.

L. O. Mayer

L O Mayer, PE
Manager of Nuclear Support Services

LOM/DMM/jh

cc: J G Keppler
NRC Resident Inspector
G Charnoff

UNITED STATES NUCLEAR REGULATORY COMMISSION

NORTHERN STATES POWER COMPANY

MONTICELLO NUCLEAR GENERATING PLANT

Docket No. 50-263

LETTER DATED JANUARY 28, 1981
RESPONDING TO NRC LETTER DATED NOVEMBER 13, 1980
BWR FEEDWATER NOZZLE AND CONTROL ROD DRIVE
RETURN LINE NOZZLE CRACKING - NUREG-0619

Northern States Power Company, a Minnesota corporation, by this letter dated January 28, 1981 hereby submits a response to an NRC letter dated November 13, 1980 which requested confirmation that implementation dates contained in NUREG-0619 will be met.

This request contains no restricted or other defense information.

NORTHERN STATES POWER COMPANY

By L. O. Mayer
L O Mayer
Manager of Nuclear Support Services

On this 28th day of January, 1981, before me a notary public in and for said County, personally appeared L O Mayer, Manager of Nuclear Support Services, and being first duly sworn acknowledged that he is authorized to execute this document on behalf of Northern States Power Company, that he knows the contents thereof and that to the best of his knowledge, information and belief, the statements made in it are true and that it is not interposed for delay.

Jeanne M Hacker

JEANNE M. HACKER
NOTARY PUBLIC - MINNESOTA
HENNEPIN COUNTY
My Commission Expires May 6, 1985