

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:7905190033 DOC.DATE: 79/05/15 NOTARIZED: NO DOCKET #
 FACIL:50-263 MONTICELLO NUCLEAR GENERATING PLANT, NORTHERN STATES 05000263
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 MAYER,L.O. NORTHERN STATES POWER CO.
 RECIP.NAME RECIPIENT AFFILIATION
 COMMISSION

SUBJECT: RESPONDS TO STELLO 790312 LTR & SUBMITS SCHEDULE FOR
 IMPLEMENTATION OF MARK I CONTAINMENT LONG-TERM PROGRAM.

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 TITLE: MARK I CONTAINMENT (RELATED INFO ON INDIVIDUAL DOCKETS)

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NORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401

May 15, 1979

Director of Nuclear Reactor Regulation
U S Nuclear Regulatory Commission
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Schedule for the Implementation and Resolution
of the Mark I Containment Long Term Program

By letter of March 12, 1979, Mr. V. Stello, Jr. requested that Northern States Power Company provide information concerning the schedule for installation of major plant modifications at Monticello as necessary to conform to the acceptance criteria established by the Mark I Containment Long Term Program.

In the case of Monticello, much of the indicated modification work was completed during, or prior to, the fall 1978 refueling outage. Modifications completed during that outage included:

- a. Mark I T-Quenchers installed on the remaining five of the eight SRV discharge lines.
- b. Vent-header deflectors installed in all sixteen torus bays.
- c. Completed improved structural supports for HPCI and RCIC turbine submerged discharge piping.
- d. Truncated vent-downcomers to three-foot submergence at minimum water level, and installed lateral restraints.

Pending final review of the LDR and PUAAG by NRC Staff, additional modifications within containment identified at this time are:

1. Additional supports or support modifications for SRV piping in both drywell and wetwell.
2. Torus internal catwalk support modifications and grating holddown improvements.

The NSP schedule objective is to complete these additional modifications during the next refueling outage scheduled to commence in January 1980.

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The plant unique analysis for SRV piping and piping supports, torus structure, and torus internals is scheduled for completion in second-quarter 1980. We anticipate this analysis will confirm conformance with acceptance criteria rather than serve as a basis for identifying further major modifications.

L. O. Mayer

L O Mayer, PE
Manager of Nuclear Support Services

LOM/jh

cc J G Keppler
G Charnoff