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SUBJECT: Forwards application for amend to License DPR-22,revising reactor protection sys specs to eliminate main steam line high radiation scram & associated vessel isolation function & description of APRM scram trip function.								I
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## Northern States Power Company

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February 14, 1992

U S Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555 10 CFR Part 50 Section 50.90

MONTICELLO NUCLEAR GENERATING PLANT Docket No. 50-263 License No. DPR-22

License Amendment Request Dated February 14, 1992 Revisions to Reactor Protection System Technical Specifications

Attached is a request for a change in the Technical Specifications, Appendix A of the Operating License for the Monticello Nuclear Generating Plant. This request is submitted in accordance with the provisions of 10 CFR Part 50, Section 50.90.

The first portion of this amendment involves revising the Reactor Protection System specifications to eliminate the main steam line high radiation scram and associated vessel isolation function. Due to the nature of the modifications involved, the best time to implement this change would be during the 1993 refueling outage. We therefore request that NRC review be completed by September 1, 1992 to allow time for preparation of the modification package and integration of the work into the outage schedule. In the interim period between the time the Technical Specification change is approved and the time the permanent plant modification is performed, we will likely retain the main steam line radiation monitor scram and vessel isolation functions as "extra" trips, although these functions could be bypassed if circumstances change.

The second portion of this change involves revising the requirements for the Intermediate Range Monitor scram functional test to clarify that the test is only required to be performed prior to each startup.

The third and final portion of this change involves revising the description of the Average Power Range Monitor scram trip function in the Bases section in order to clarify when bypasses are permissible.

Exhibit A contains a description of the proposed changes, the reasons for requesting the changes, a Safety Evaluation, and a Determination of Significant Hazards Consideration. Exhibit B contains the current Technical Specification pages marked up with the proposed changes. Exhibit C contains revised Monticello Technical Specification pages.

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Please contact us if you require further information related to this request.

musta

Thomas M Parker Manager Nuclear Support Services

c: Regional Administrator-III, NRC NRR Project Manager, NRC Resident Inspector, NRC MDH, Attn: Dr R Thron J Silberg

Attachments: Affidavit to the US Nuclear Regulatory Commission Exhibit A - Evaluation of Proposed Changes Figure A-1 - APRM Scram Logic (simplified diagram) Exhibit B - Technical Specification pages marked up with proposed wording changes Exhibit C - Revised Monticello Technical Specification Pages