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Northern States Power Company

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November 1, 1989

Submitted pursuant to 10 CFR 50.71

Director Office of Nuclear Reactor Regulation US Nuclear Regulatory Commission Washington DC 20555

> MONTICELLO NUCLEAR GENERATING PLANT Docket No. 50-263 License No. DPR-22

> <u>Submittal of Revision No. 8 to the</u> <u>Updated Safety Analysis Report (USAR)</u>

In our letter entitled, "License Amendment Request Dated August 2, 1989 Detetion of Primary Containment Isolation Valve Table," we committed to make the necessary revisions to USAR Table 5.2-3b prior to plant startup following the 1989 refueling outage. Revision 8 to the Monticello Updated Safety Analysis Report (USAR) completes that commitment.

A description and summary of the safety evaluation for changes made under the provisions of 10 CFR 50.59 will be included in the "Annual Report of Changes, Tests and Experiments - December 1989" which will be submitted in July 1990.

Exhibit A contains the USAR page changes and instructions for entering the pages.

Thomas M Parker Manager Nuclear Support Services

c: Regional Administrator-III, NRC NRR Project Manager, NRC Resident Inspector, NRC G Charnoff

Attachment

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Exhibit A

MONTICELLO NUCLEAR GENERATING PLANT

Revision 8 to the Updated Safety Analysis Report

The attached instructions should be followed when making this revision to the Updated Safety Analysis Report. If you have any questions concerning this revision call:

Monica Vik (612) 337-2038

Volume	Tab	Remove <u>Page No.</u>	Rev <u>No.</u>	Insert <u>Page No.</u>	Rev <u>No.</u>
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2. 5.	Plant Containment Systems	5.2-3b (p 1 of 4)	/	5.2-36 (p 1 of 4)	8
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TBL 2.3-5	0	FTG 2.5-2	Õ
TBI. $2 3-6$	0	FIG 2 5-3	Õ
TBL 2 3-7	õ	FTC 2.5 - 5	ñ
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(2 of 2)	0	3.1-2	0
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TBL 2.3-23	5	3 2 - 3	6
TRL $2 - 3 - 24$	5	3.0.4	3
TBL 2.3 $2+$	5	3.2-4	ר ד
TBL $2.3-25$	5	3.2-5	7
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TDL 2.4-1	0	3.2-8	0
TBL 2.3-1	0	3.3-1	0
$(1 \circ f \circ 2)$	0	3.3-2	6
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	0	3.3-5	0
101 2.0-2	0	3.3-6	0
	0	3.3-7	0
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5.2-10	1	5.3-7	0
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DATED SAFETY ANALYSIS REPORT (USAR)

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TBL 5.2-3b	-	6-vii	5
(4 of 4)	1	6-viii (blank)	0
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		6.1-2	6

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Record of USAR Revisions

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ORIGINAL	10/81	5/1/82	· · · · ·
1	10/82	4/29/83	
2	10/83	4/30/84	
3	12/84	6/28/85	
4	12/85	6/30/86	Added Appendices A
5	12/86	6/30/87	chru i
6	12/87	6/30/88	
7	12/88	6/30/89	

<u>Annica</u> Vik Prepared by M M Vik

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Reviewed By Plant Manager or designee Mgr - Nuclear Support Serv



TABLE 5.2-3b (Page 1 of 4)

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Primary Containment Automatic Isolation Valves

Isolation Group		Isolation	Valves	Permissible Operating	Normal
<u>(Note 1)</u>	Application	<u>Inboard</u>	<u>Outboard</u>	<u>Time (sec)</u>	<u>Postion (Note 3)</u>
1	Main Steam Isolation	AO-2-80A AO-2-80B AO-2-80C AO-2-80D	AO - 2 - 86A AO - 2 - 86B AO - 2 - 86C AO - 2 - 86D	$3 \leq t \leq 5$	Open
	Main Steam Line Drain	MO-2373	MO-2374	60	Closed
	Reactor Water Sample	CV-2790	CV-2791	60	Closed
2	Drywell Equipment Sump	A0-2561A	A0-2561B	60	Open
	Drywell Floor Sump	A0-2541A	A0-2541B	60	Open
	Torus Vent Bypass	-	CV-2384	60	Closed
	Torus to N ₂ Recirculation	-	CV-7440	60	Closed
	Torus Vent	-	A0-2383	60	Closed
	Torus Vent	-	A0-2896	60	Closed
	Drywell Vent Bypass	-	CV-2385	60	Closed
	Drywell Vent	-	A0-2386	60	Closed
	Drywell Vent	-	A0-2387	60	Closed
	Torus Air Purge Air Supply	-	A0-2378	60	Closed
	Drywell Air Purge Supply	-	A0-2381	60	Closed
	Containment Air Purge Supply	-	A0-2377	60	Closed
	TIP Ball Valves (3)	-	-	Note 2	Closed

TABLE 5.2-3b

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TABLE 5.2-3b (Page 2 of 4)

Primary Containment Automatic Isolation Valves

Isolation Group <u>(Note 1)</u>	Application	Isolation <u>Inboard</u>	Valves <u>Outboard</u>	Permissible Operating <u>Time (sec)</u>	Normal <u>Postion (Note 3)</u>
2	RHR Supply	MO-2029	MO-2030	120	Closed
	RHR Head Cooling	MO-2027	MO-2026	120	Closed
	RHR Return to A Loop	-	M0-2014	120	Closed
	RHR Return to B Loop	-	MO-2015	120	Closed
	Containment Nitrogen Supply	-	CV-3269	60	Closed
	Torus Nitrogen Supply	-	CV-3267	60	Closed
	Drywell Nitrogen Supply	-	CV-3268	60	Closed
	Oxygen Analyzer Sample Point	-	CV-3311	60	Open
	Oxygen Analyzer Sample Point	-	CV-3312	60	Open
	Oxygen Analyzer Return	-	CV-3313	60	Open
	Oxygen Analyzer Return	-	CV-3314	60	Open
2	CAM Sample Line (Drywell)	-	sv-3307	30	Closed
2	PASS Sample Line (Drywell)	-	SV-4081	30	Closed
2	CAM Sample Line (Drywell)	-	SV-3308	30	Closed
2	PASS Sample Line (Drywell)		SV-4082	30	Closed
2	Drywell to CAM Analyzer A (Supply)	-	SV-4001A	30	Closed
2	Dr ywe ll to CAM Analyzer A (Supply)	-	SV-4020A	30	Closed
2	Dr ywell to CAM Analyzer B (Supply)	-	SV-4001B	30	Closed
2	Drywell to CAM Analyzer B (Supply)	-	SV-4020B	30	Closed
2	Torus to CAM Analyzer A (Supply)	-	SV-4002A	30	Closed

TABLE 5.2-3b

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TABLE 5.2-3b

(Page 3 of 4)

Primary Containment Automatic Isolation Valves

		Taolation	Values	Permissible	Normal
<u>(Note 1)</u>	Application	Inboard	<u>Outboard</u>	<u>Time (sec)</u>	Postion (Note 3)
2	Torus to CAM Analyzer A (Supply)	-	SV-4003A	30	Closed
2	Torus to CAM Analyzer B (Supply)	-	SV-4002B	30	Closed
2	Torus to CAM Analyzer B (Supply)	-	SV-4003B	30	Closed
2	CAM Analyzer A to Torus (Return)	-	SV-4004A	30	Closed
2	CAM Analyzer A to Torus (Return)	-	SV-4005A	30	Closed
2	CAM Analyzer B to Torus (Return)	-	SV-4004B	30	Closed
2	CAM Analyzer B to Torus (Return)	-	SV-4005B	30	Closed
2	CGCS A Drywell to Recombiner (Supply)		AO-7422A	20	Closed
2	CGCS A Drywell to Recombiner (Supply)		AO-7423A	20	Closed
2	CGCS A Recombiner to Torus (Return)		AO-7424A	20	Closed
2	CGCS A Recombiner to Torus (Return)		AO-7425A	20	Closed
2	CGCS B Drywell to Recombiner (Supply)		AO-7422B	20	Closed
2	CGCS B Drywell to Recombiner (Supply)		AO-4723B	20	Closed
2	CGCS B Recombiner to Torus (Return)		AO-7424B	20	Closed
2	CGCS B Recombiner to Torus (Return)		AO-7425B	20	Closed
3	Reactor Water Cleanup Supply	y MO-2397	MO-2398	40	Open
	Reactor Water Cleanup Return	n -	MO-2399	40	Open
	Reactor Water Sample	CV-2790	CV-2791	60	Open
4	HPCI Steam Supply	MO-2034	MO-2035	40	Open
5	RCIC Steam Supply	MO-2075	MO-2076	30	Open

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TABLE 5.2-3b (Page 4 of 4)

Primary Containment Automatic Isolation Valves

Note 1: Containment isolation groupings are as follows:

Group 1 The values in Group 1 are closed upon any one of the following conditions:

- 1. Reactor low low water level
- 2. Main steam line high radiation
- 3. Main steam line high flow
- 4. Main steam line tunnel high temperature
- 5. Main steam line low pressure (RUN mode only)
- Group 2 The valves in Group 2 are closed upon any one of the following conditions: 1. Reactor low water level
 - 1. Reactor fow water fever
 - 2. High Drywell Pressure
- Group 3 Same as Group 2.

Group 4 Isolation values in the HPCI System are closed upon any one of the following conditions:

- 1. HPCI steam line high flow
- 2. HPCI steam line low pressure
- 3. High temperature in the vicinity of the HPCI steam line.
- Group 5 Isolation values in the RCIC System are closed upon any one of the following conditions:
 - 1. RCIC steam line high flow
 - 2. RCIC steam line low pressure
 - 3. High temperature in the vicinty of the RCIC steam line.
- Note 2: Testing consists of verifying TIP automatic withdrawal and ball valve closure on a simulated Group 2 isolation signal.
- Note 3: The normal position of the valves during operation is indicated. However, valve positions may be changed as required to support plant operation, such as containment nitrogen addition and venting, or to allow for surveillance testing, such as valve stroke timing and CGCS operability testing.

TABLE 5.2-3b

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