

UNITED STATES NUCLEAR REGULATORY COMMISSION

NORTHERN STATES POWER COMPANY

MONTICELLO NUCLEAR GENERATING PLANT

DOCKET NO. 50-263

REQUEST FOR AMENDMENT TO
OPERATING LICENSE DPR-22

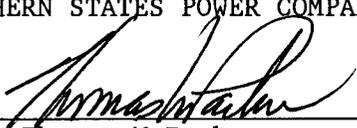
LICENSE AMENDMENT REQUEST DATED July 26, 1989

Northern States Power Company, a Minnesota corporation, requests authorization for changes to Appendix A of the Monticello Operating License as shown on the attachments labeled Exhibits A, B, C and D. Exhibit A describes the proposed changes, describes the reasons for the changes, and contains a significant hazards evaluation. Exhibits B and C are copies of the Monticello Technical Specifications incorporating the proposed changes. Exhibit D contains a sample Core Operating Limits Report for Cycle 14.

This letter contains no restricted or other defense information.

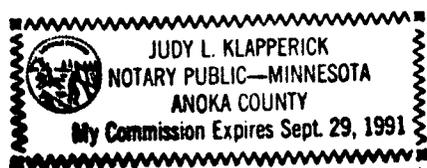
NORTHERN STATES POWER COMPANY

By


Thomas M Parker
Manager
Nuclear Support Services

On this 26 day of July 1989 before me a notary public in and for said County, personally appeared Thomas M Parker, Manager Nuclear Support Services, and being first duly sworn acknowledged that he is authorized to execute this document on behalf of Northern States Power Company, that he knows the contents thereof, and that to the best of his knowledge, information, and belief the statements made in it are true and that it is not interposed for delay.





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Exhibit A

Monticello Nuclear Generating Plant

License Amendment Request Dated July 26, 1989

Evaluation of Proposed changes to the Technical Specifications
Appendix A of Operating License DPR-22.

Pursuant to 10 CFR 50, Section 50.59 and 50.90, the holders of Operating License DPR-22 hereby propose the following changes to Appendix A Technical Specifications:

Proposed Changes

a. Core Operating Limit Report

A new definition, Core Operating Limit Report, has been added per the guidance provided in Generic Letter 88-16 (new page 5b).

A new reporting requirement has been added to require the submittal of the Core Operating Limit Report prior to the startup of each cycle (page 249b).

b. Definition of the Limiting Control Rod Pattern

The definition of a Limiting Control Rod Pattern has been deleted. The limits associated with this definition have been moved to the Core Operating Limits Report (page 2 and 46).

c. Rod Block Monitor Upscale Setpoints

The RBM upscale setpoints have been moved to the Core Operating Limit Report (pages are 57, 58 and 67).

d. Maximum Average Planar Linear Heat Generation Rate

The limits for Maximum Average Planar Linear Heat Generation Rate have been moved to the Core Operating Limits Report (pages v, vii, 211, 214, 215a, 215b and 216).

e. Linear Heat Generation Rate

These limits for Linear Heat Generation Rate have been moved to the Core Operating Limits Report (page 212).

f. Minimum Critical Power Ratio

The limits for Minimum Critical Power Ratio have been moved to the Core Operating Limits Report (pages v, 213, 215c, 215d, 216, 217 and 218).

g. Bases Modifications

The bases have been modified to remove references to General Electric Topicals (pages 14, 20, 67, 69a, 216, 217 and 218) and also update the critical heat flux correlation used for Cycle 14.

Reason for Changes

Changes a, c, d, e and f implement the actions suggested in Generic Letter 88-16.

Change a: This definition and reporting requirement are requested by Generic Letter 88-16.

Change c: The RBM setpoint is a function of MCPR. This was documented in Appendix D to the Licence Amendment Request dated February 4, 1987, where three different setpoints were provided for MCPRs of 1.25, 1.30 and 1.35. In order to have the flexibility to use different RBM setpoints without requiring NRC approval in advance, these setpoints are proposed to be placed in the Core Operating Limits Report.

Change d: Table 3.11.1, which specifically references fuel types, has been moved to the Core Operating Limits Report so that addition of NRC approved fuel types will not require NRC approval for the first cycle use at Monticello.

Change e: The Linear Heat Generation Rate limit is a function of fuel type. This change will eliminate the necessity of modifying the Technical Specifications when using new fuel types.

Change f: The Minimum Critical Power Ratio is calculated for each cycle. The change will eliminate the necessity of modifying the Technical Specifications for changes in MCPR.

The core design and transient analysis for Cycle 15 will be performed by Northern States Power using NRC approved methods. Change b and g will eliminate the necessity of another license amendment request prior to Cycle 15.

Safety Evaluation and Determination of Significant Hazards Considerations

The proposed change to the Operating License has been evaluated to determine whether it constitutes a significant hazards consideration as required by 10 CFR Part 50, Section 50.91 using the standards provided in Section 50.92. This analysis is provided below:

1. The proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated.

This change relocates several nuclear limits from the Technical Specifications to the Core Operating Limits Report and will not involve any increase in the probability or consequences of an accident previously evaluated.

2. The proposed amendment will not create the possibility of a new or different kind of accident from any accident previously analyzed.

The proposed amendment affects only the location of cycle-specific limits and cannot create the possibility of a new or different kind of accident.

3. The proposed amendment will not involve a significant reduction in the margin of safety.

The proposed amendment affects only the location of cycle-specific limits and will not involve a significant reduction in the margin of safety.

The Commission has provided guidance (March 6, 1986 Federal Register) concerning the application of the standards in 10 CFR 50.92 for determining whether a significant hazards consideration exists by providing certain examples of amendments that will likely be found to involve no significant hazards considerations. The changes to the Monticello Technical Specifications proposed in this amendment request are representative of NRC example (iii): because they involve a change resulting from a nuclear reactor core reloading. The acceptance criteria for the Technical Specifications have not changed and the analytical methods used to demonstrate conformance with regulations are not significantly changed from previously approved methods. Based on this guidance and the reasons discussed above, we have concluded that the proposed changes do not involve a significant hazards consideration.

Environmental Assessment

This license amendment request does not change effluent types or total effluent amounts nor does it involve an increase in power level. This change moves several nuclear limits from the Technical Specifications to the Core Operating Limits Report. Therefore, this change will not result in any significant environmental impact.