

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8710160314 DOC. DATE: 87/10/07 NOTARIZED: NO DOCKET #  
 FACIL: 50-263 Monticello Nuclear Generating Plant, Northern States 05000263  
 AUTH. NAME AUTHOR AFFILIATION  
 MUSOLF, D. Northern States Power Co.  
 RECIP. NAME RECIPIENT AFFILIATION  
 Document Control Branch (Document Control Desk)

SUBJECT: Rev 2 to 860501 application for amend to License DPR-22,  
 revising Tech Specs 4.7.A.2.b.4.c & 4.7.A.2.c.1 to read as  
 listed re containment leakage testing.

DISTRIBUTION CODE: A001D COPIES RECEIVED: LTR 1 ENCL 0 SIZE: 2  
 TITLE: OR Submittal: General Distribution

NOTES:

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	PD3-3 LA	1	PD3-3 PD	5
	SCALETTI, D	1		
INTERNAL:	ARM/DAF/LFMB	1	NRR/DEST/ADS	1
	NRR/DEST/CEB	1	NRR/DEST/MTB	1
	NRR/DEST/RSB	1	NRR/DOEA/TSB	1
	NRR/RMAS/ILRB	1	OGC/HDS1	1
	<u>REG FILE</u> 01	1	RES/DE/EIB	1
EXTERNAL:	EG&G BRUSKE, S	1	LPDR	1
	NRC PDR	1	NSIC	1

TOTAL NUMBER OF COPIES REQUIRED: LTTR 21 ENCL 18



USNRC-DS

1987 OCT 17 A 10:10

Northern States Power Company

414 Nicollet Mall  
Minneapolis, Minnesota 55401  
Telephone (612) 330-5500

October 7, 1987

10 CFR Part 50  
Section 50.90

Director of Nuclear Reactor Regulation  
US Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT  
DOCKET NO. 50-263 LICENSE NO. DPR-22

Revision No. 2 to License Amendment Request  
Dated May 1, 1986, Containment Leakage Testing

Following discussions with the Monticello NRR Project Manager and members of the NRC Staff, two additional changes in our License Amendment Request Dated May 1, 1986 were identified. Northern States Power Company concurs in these additional changes as follows:

Specification 4.7.A.2.b.4.c

Change to read as follows:

- c. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be limited between 75 to 125% of La.

Specification 4.7.A.2.c.1

Change to read as follows:

1. At each refueling shutdown, and at six month intervals thereafter, by conducting an overall airlock leakage test at  $\geq Pa$  and demonstrating that overall airlock leakage rate is  $\leq 0.05La$ . This test interval may be extended up to the next refueling outage (up to a maximum interval between tests at Pa of 24 months) if there have been no air lock openings since the last successful test at Pa.

8710160314 871007  
PDR ADDCK 05000263  
P PDR

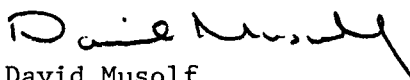
A001  
110

Director of NRR  
October 7, 1987  
Page 2

Northern States Power Company

These are minor changes consistent with our original application. They have no effect on information presented in Exhibit A of our original submittal, including the Determination of Significant Hazards Considerations. Please contact us if you have additional questions related to this revision.

A license amendment fee for this revision was determined to be unnecessary because it is a revision to a License Amendment Request for which a fee has already been paid.

  
David Musolf  
Manager Nuclear Support Services

c: Sr Project Manager, NRC  
Sr Resident Inspector, NRC  
Regional Administrator, Region III, NRC  
G Charnoff  
MPCA (State Contact) Attn: Dr J W Ferman