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SUBJECT: Forwards Rev 1 to B60501 application for amend to License
 DPR-22, revising Tech Specs to require scheduling of Type B &
 C tests, including tests of MSIVs & feedwater isolation
 valves, for each refueling outage.

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 TITLE: OR Submittal: Append J Containment Leak Rate Testing

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Northern States Power Company

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July 15, 1987

Director of Nuclear Reactor Regulation
Attention: Document Control Desk
US Nuclear Regulatory Commission
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT
DOCKET NO. 50-263 LICENSE NO. DPR-22

Revision No. 1 to License Amendment Request
Dated May 1, 1986, Containment Leakage Testing

Attached are corrected Exhibit B, pages 157, 158, 160, and 161 for our License Amendment Request date May 1, 1986. The attached revised pages should be substituted directly for the Exhibit B pages in the original submittal. All other portions of our original submittal remain unaffected by these revisions.

These revised pages are intended to correct deficiencies identified during NRC Staff review of the original submittal. The changes include:

- a. Specification 4.7.A.2.b.4.c is changes to require that the rate of which gas is injected into containment or bled from containment during supplemental verification testing be 0.75 to 1.25 of La.
- b. Specification 4.7.A.2.b.5 is revised to specifically require Type B and C tests, including tests of main steam and feedwater isolation valves, to be scheduled for each refueling outage.
- c. Clarify the airlock leakage test requirements to more clearly describe the conditional approval granted by the Commission for testing the airlock at reduced pressure when the airlock has not been used. This approval was granted by the Commission on June 3, 1984 (Attachment 1).

The Monticello personnel airlock is not used during plant operation because of the inerted atmosphere in

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the containment. Our experience over 15 years has shown that the airlock has demonstrated good leak tightness. Because of the design of the airlock, only overall leakage testing can be performed (the space between the inner and outer doors is pressurized) since the airlock design does not incorporate double gasketed door seals. Bracing is installed inside the airlock to hold the inner door against its seat.

The original plant Technical Specifications required airlock tests to be conducted at 10 psig. The manufacturer has indicated that that inner volume testing at Pa is possible to meet the requirements of Appendix J to 10 CFR Part 50. However, because this testing puts a large force on the inner door which tends to unseat and open it, we believe that it is prudent to avoid full pressure testing during plant operation if it is not necessary. The revised NRC Staff position approved for Monticello on June 3, 1984 will permit low pressure testing provided the airlock has not been used since the last full pressure test.

d. A typographical error on page 157 is corrected (the word "operable" was improperly substituted for "inoperable" in the original submittal. The omission of a greater-than-or-equal-to symbol on page 158 is corrected.

Please contact us if you have additional questions related to this revision.

A license amendment fee for this revision was determined to be unnecessary because it is a correction to a License Amendment Request for which a fee has already been paid.



David Musolf
Manager Nuclear Support Services

c: Sr Project Manager, NRC
Sr Resident Inspector, NRC
Regional Administrator, Region III, NRC
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Attachment